

December 7, 2001

Mr. J. A. Price
Vice President - Nuclear Technical Services - Millstone
Dominion Nuclear Connecticut, Inc.
c/o Mr. David A. Smith
Rope Ferry Road
Waterford, CT 06385

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2 - ISSUANCE OF
AMENDMENT RE: EMERGENCY DIESEL GENERATOR SURVEILLANCE
REQUIREMENT (TAC NO. MB1781)

Dear Mr. Price:

The Commission has issued the enclosed Amendment No. 259 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2, in response to your application dated April 23, 2001.

The amendment approves a change to Technical Specification (TS) 3.8.1.1, "Electrical Power System - A.C. Sources." The change removes Surveillance Requirement 4.8.1.1.2.c.1 regarding Emergency Diesel Generator (EDG) inspection at least once per 18 months during shutdown condition. Although removed from the TS, the EDG inspection will continue to be performed in accordance with procedures prepared in conjunction with the recommendations of the manufacturer.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

John T. Harrison, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 259 to DPR-65
2. Safety Evaluation

cc w/encls: See next page

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Unit 2

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Unit 2

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DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 259
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated April 23, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 259, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: December 7, 2001

ATTACHMENT TO LICENSE AMENDMENT NO. 259

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following page of the Appendix A, Technical Specifications, with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove

3/4 8-3

Insert

3/4 8-3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 259

TO FACILITY OPERATING LICENSE NO. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated April 23, 2001, Dominion Nuclear Connecticut, Inc. (DNC/the licensee), submitted a request to change the Millstone Nuclear Power Station, Unit No. 2 (MP2) Technical Specifications (TSs). The requested change would modify TS 3.8.1.1, "Electrical Power System - A.C. Sources." The change would remove Surveillance Requirement (SR) 4.8.1.1.2.c.1 which requires inspecting each Emergency Diesel Generator (EDG) at least once per 18 months during shutdown. The licensee states in their application that the inspection of the EDGs will continue to be performed in accordance with procedures prepared in conjunction with the recommendations of the manufacturer and will still be performed during shutdown since the inspection activities normally take 7 to 10 days to complete and the TSs allow only 72 hours for the EDG to be inoperable. However, by letter dated May 31, 2001, the licensee submitted a separate request to change the MP2 TSs to extend the EDG allowed outage time from 72 hours to 14 days which, when combined with this amendment, would allow the performance of EDG maintenance activities during plant operation. This request is currently being reviewed by the NRC staff under TAC No. MB2196.

2.0 BACKGROUND

A similar TS change request for Millstone Nuclear Power Station, Unit No. 3, Docket No. 50-423, was reviewed and approved by the Nuclear Regulatory Commission (NRC) as part of TS Amendment No. 194, dated February 2, 2001. The licensee states that the SR for Unit 2 is a maintenance activity since it does not verify operability or the EDG functions assumed in the safety analysis. The licensee states that, by removing the SR from the Unit 2 TS, DNC will be able to evaluate the manufacturer's maintenance recommendations and process changes to the EDG maintenance procedures utilizing the provisions of Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.59.

MP2 has two EDGs. Each EDG is capable of supplying power to the respective emergency 4160 V bus. Both EDGs are 12 cylinder 900 rpm opposed piston Fairbanks Morse diesel engines. During normal power operation, the EDGs are maintained in a standby mode. The EDGs may be manually started and will automatically start on a loss of power to the respective emergency bus, or a safety injection actuation signal. If the normal and alternate offsite power

sources are not available, the EDGs are then automatically connected to the respective emergency bus and sequentially loaded. Upon receiving an automatic start signal, the EDGs are accelerated to rated speed, frequency, and voltage within 15 seconds, and are ready to accept load in accordance with the unit's sequential loading schedule.

3.0 EVALUATION

The licensee proposes to remove SR 4.8.1.1.2.c.1 from the MP2 TS. This SR requires, at least once per 18 months during shutdown, "Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations..." in order to demonstrate operability. However, this activity does not actually demonstrate any operability functions of the EDGs assumed in the safety analysis and would be more accurately categorized as a maintenance procedure. Therefore, it is appropriate to include this activity as part of the plant maintenance procedures, as the licensee proposed in their application, which is consistent with NUREG-1432, Standard Technical Specifications, Combustion Engineering Plants, Rev. 2 (STS). In addition, the licensee stated in their application that EDG inspections and other maintenance activities that are recommended by the manufacturer will continue to be performed in accordance with procedures prepared in conjunction with the recommendations of the manufacturer. The licensee also stated that any changes to the plant maintenance procedures that contain the EDG maintenance requirements would be subject to the requirements of 10 CFR 50.59, "Changes, Tests, and Experiments."

Because SR 4.8.1.1.2.c.1 does not verify operability, and the licensee will continue to perform this maintenance activity in accordance with the manufacturer's recommendations which will be subject to 10 CFR 50.59 requirements, the staff finds that relocating the requirements of SR 4.8.1.1.2.c.1 from the MP2 TS to plant procedures is acceptable and is consistent with the STS.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (66 FR 31705). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: O. Chopra

Date: December 7, 2001