

August 13, 1999

Mr. R. P. Necci - Vice President
Nuclear Oversight and Regulatory Affairs
c/o Mr. David A. Smith
Northeast Nuclear Energy Company
P. O. Box 128
Waterford, CT 06385-0128

SUBJECT: MILLSTONE NUCLEAR POWER STATION, UNIT NOS. 2 AND 3 - ISSUANCE
OF AMENDMENT RE: ADMINISTRATIVE CONTROLS
(TAC NOS. MA4937 AND MA4938)

Dear Mr. Necci:

The Commission has issued the enclosed Amendment Nos. 239 and 173 to Facility Operating License Nos. DPR-65 and NPF-49 for the Millstone Nuclear Power Station, Unit Nos. 2 and 3, respectively, in response to your application dated March 5, 1999.

The amendments relocate certain Technical Specification (TS) Section 6.0 administrative controls to the NRC-approved Northeast Utilities Quality Assurance Program (NUQAP) Topical Report in accordance with NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specifications Administrative Controls Related to Quality Assurance," dated December 12, 1995. Specifically, Sections 6.2.3, Independent Safety Engineering Group (Unit 3 only); 6.5, Review and Audit; 6.6, Reportable Event Action (partial); 6.7, Safety Limit Violation (partial); and 6.10, Record Retention. The proposed amendments also delete parts of Section 6.6 and 6.7 because their requirements were duplicated in existing regulations or elsewhere in the TSs. In addition, the proposed amendment would modify the table of contents and other TS sections to incorporate the aforementioned changes (e.g., correct references).

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
ORIGINAL SIGNED BY:
John A. Nakoski, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-336 and 50-423

190019

- Enclosures: 1. Amendment No. 239 to DPR-65
2. Amendment No. 173 to NPF-49
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 13, 1999

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Sincerely,

A handwritten signature in cursive script that reads "John A. Nakoski".

John A. Nakoski, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-336 and 50-423

Enclosures: 1. Amendment No. 239 to DPR-65
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UNITED STATES
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WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 239
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated March 5, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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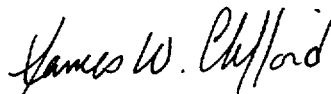
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 239 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 13, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 239

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

Insert

XVI	XVI
XVII	XVII
XVIII	--
3/4 6-11	3/4 6-11
3/4 7-22a	3/4 7-22a
6-5	6-5
6-6	6-6
6-7	--
6-8	--
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6-11	--
6-12	--
6-13	--
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CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

- b. Removing one wire from a dome, a vertical and a hoop tendon checked for lift off force pursuant to Specification 4.6.1.6.1.a and determining that over the entire length of the removed wire that:
1. The tendon wires are free of corrosion.
 2. There are no changes in physical appearance of the sheathing filler grease.
 3. A minimum tensile strength of 11,760 pounds for at least three wire samples (one from each end and one at mid-length) cut from each removed wire. Failure of any one of the wire samples to meet the minimum tensile strength test is evidence of abnormal degradation of the containment structure.

4.6.1.6.2 End Anchorages and Adjacent Concrete Surfaces The structural integrity of the end anchorages and adjacent concrete surfaces shall be demonstrated by determining through inspection that no apparent changes or degradation has occurred in the visual appearance of the end anchorage concrete exterior surfaces or as indicated by the concrete crack patterns adjacent to the end anchorages. Inspections of the concrete shall be performed concurrent with the containment tendon surveillance (reference Specification 4.6.1.6.1).

4.6.1.6.3 Liner Plate The structural integrity of the containment liner plate shall be determined in accordance with the Containment Leakage Rate Testing Program.

4.6.1.6.4 Reports In lieu of any other report required by Section 50.73 to 10 CFR Part 50, an initial report of any abnormal degradation of the containment structure detected during the above required tests and inspections shall be made within 10 days after completion of the surveillance requirements of this specification and the detailed report shall be submitted pursuant to Specification 6.9.2 within 90 days after completion. This report shall include a description of the condition of the concrete (especially at tendon anchorages), the inspection procedure, the tolerances on cracking, and the corrective actions taken.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

If any snubber selected for testing either fails to lock-up or fails to move (i.e., frozen in place), the cause will be evaluated and if caused by manufacturer design deficiency, all snubbers of the same design subject to the same defect shall be tested regardless of location or difficulty or removal. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber (s) were adversely affected by the inoperability of the snubber (s) in order to ensure that the supported component remains capable of meeting the designed service.

e. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension.

f. Mechanical Snubbers Functional Test Acceptance Criteria*

The mechanical snubber functional test shall verify that:

1. The force that initiates free movement of the snubber rod in either tension or compression is less than the specified maximum drag force.
2. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.

g. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service life is based shall be maintained as required by Quality Assurance Program Topical Report.

*Mechanical snubber functional test acceptance criteria shall become effective upon installation of snubber testing equipment but not later than June 30, 1985.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

A retraining and replacement training program for the facility staff that meets or exceeds the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.59 shall be maintained.

6.5 Deleted.

PAGES 6-6 THROUGH 6-13 HAVE BEEN INTENTIONALLY DELETED.

ADMINISTRATIVE CONTROLS

6.6 Deleted.

6.7 Deleted.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, February, 1978.
- b. Refueling operations.
- c. Surveillance activities of safety related equipment.
- d. Not used.
- e. Not used.

ADMINISTRATIVE CONTROLS

- f. Fire Protection Program implementation.
 - g. Quality Control for effluent monitoring using the guidance in Regulatory Guide 1.21 Rev. 1, June 1974.
 - h. Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM) implementation, except for Section I.E., Radiological Environmental Monitoring.
- 6.8.2
- a. The designated manager or designated officer or designated senior officer may designate specific procedures and programs, or classes of procedures and programs to be reviewed in accordance with the Quality Assurance Program Topical Report.
 - b. Procedures and programs listed in Specification 6.8.1, and changes thereto, shall be approved by the designated manager or designated officer or by cognizant Manager or Directors who are designated as the Approval Authority by the designated manager or designated officer, as specified in administrative procedures. The Approval Authority for each procedure and program or class of procedure and program shall be specified in administrative procedures.
 - c. Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved in accordance with the Quality Assurance Program Topical Report, prior to implementation. Each procedure of Specification 6.8.1 shall be reviewed periodically as set forth in administrative procedures.
- 6.8.3
- Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
 - c. The change is documented, reviewed and approved in accordance with the Quality Assurance Program Topical Report within 14 days of implementation.
- 6.8.4
- Written procedures shall be established, implemented and maintained covering Section I.E, Radiological Environmental Monitoring, of the REMDCM.

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS (CONT.)

- b. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- c. Safety Class 1 Inservice Inspection Program Review, Specification 4.4.10.1.
- d. ECCS Actuation, Specifications 3.5.2 and 3.5.3.
- e. Deleted
- f. Deleted
- g. RCS Overpressure Mitigation, Specification 3.4.9.3.
- h. Radiological Effluent Reports required by Specifications 3.11.1.2, 3.11.2.2, 3.11.2.3 and 3.11.4.
- i. Degradation of containment structure, Specification 4.6.1.6.4.
- j. Steam Generator Tube Inspection, Specification 4.4.5.1.5.
- k. Accident Monitoring Instrumentation, Specification 3.3.3.8.
- l. Radiation Monitoring Instrumentation, Specification 3.3.3.1.
- m. Reactor Coolant System Vents, Specification 3.4.11.

6.10 Deleted.

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ADMINISTRATIVE CONTROLS

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 Pursuant to paragraph 20.203(c)(5) of 10 CFR Part 20, in lieu of the "control device" or "alarm signal" required by paragraph 20.203(c), each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mR/h at 45 cm (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Health Physics Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mR/h, provided they are otherwise following plant radiation protection procedures for entry into such high radiation areas. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area; or
- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate levels in the area have been established and personnel have been made knowledgeable of them; or
- c. An individual qualified in radiation protection procedures with a radiation dose rate monitoring device, who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the Health Physics Manager in the RWP.

ADMINISTRATIVE CONTROLS

6.15 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)

Section I, Radiological Effluents Monitoring Manual (REMM), shall outline the sampling and analysis programs to determine the concentration of radioactive materials released offsite as well as dose commitments to individuals in those exposure pathways and for those radionuclides released as a result of station operation. It shall also specify operating guidelines for radioactive waste treatment systems and report content.

Section II, the Offsite Dose Calculation Manual (ODCM), shall describe the methodology and parameters to be used in the calculation of offsite doses due to radioactive gaseous and liquid effluents and in the calculations of gaseous and liquid effluent monitoring instrumentation Alarm/Trip Setpoints consistent with the applicable LCO's contained in these Technical Specifications.

Changes to the REMODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by the Quality Assurance Program Topical Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s), and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance in accordance with the Quality Assurance Program Topical Report.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire REMM or ODCM, as appropriate, as a part of or concurrent with the Annual Radioactive Effluent Report for the period of the report in which any change was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-423

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 173
License No. NPF-49

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated March 5, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

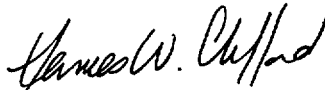
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-49 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 173, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James W. Clifford, Chief, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: August 13, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 173

FACILITY OPERATING LICENSE NO. NPF-49

DOCKET NO. 50-423

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

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2-1

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2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

2.1 SAFETY LIMITS

REACTOR CORE

2.1.1 The combination of THERMAL POWER, pressurizer pressure, and the highest operating loop coolant temperature (T_{avg}) shall not exceed the limits shown in Figures 2.1-1 and 2.1-2 for four and three loop operation, respectively.

APPLICABILITY: MODES 1 and 2.

ACTION:

Whenever the point defined by the combination of the highest operating loop average temperature and THERMAL POWER has exceeded the appropriate pressurizer pressure line, be in HOT STANDBY within 1 hour.

REACTOR COOLANT SYSTEM PRESSURE

2.1.2 The Reactor Coolant System pressure shall not exceed 2750 psia.

APPLICABILITY: MODES 1, 2, 3, 4, and 5.

ACTION:

MODES 1 and 2:

Whenever the Reactor Coolant System pressure has exceeded 2750 psia be in HOT STANDBY with the Reactor Coolant System pressure within its limit within 1 hour.

MODES 3, 4 and 5:

Whenever the Reactor Coolant System pressure has exceeded 2750 psia, reduce the Reactor Coolant System pressure to within its limit within 5 minutes.

INSTRUMENTATION

3/4.3.4 TURBINE OVERSPEED PROTECTION

LIMITING CONDITION FOR OPERATION

3.3.4 At least one Turbine Overspeed Protection System shall be OPERABLE.

APPLICABILITY: MODES 1, 2,* and 3.*

ACTION:

- a. With one stop valve or one governor valve per high pressure turbine steam line inoperable and/or with one reheat stop valve or one reheat intercept valve per low pressure turbine steam line inoperable, restore the inoperable valve(s) to OPERABLE status within 72 hours, or close at least one valve in the affected steam line(s) or isolate the turbine from the steam supply within the next 6 hours.
- b. With the above required Turbine Overspeed Protection System otherwise inoperable, within 6 hours isolate the turbine from the steam supply.

SURVEILLANCE REQUIREMENTS

4.3.4.1 The provisions of Specification 4.0.4 are not applicable.

4.3.4.2 The above required Turbine Overspeed Protection System shall be maintained, calibrated, tested, and inspected in accordance with the Millstone Unit No. 3 Turbine Overspeed Protection Maintenance and Testing Program. Adherence to this program shall demonstrate OPERABILITY of this system. The program and any revisions should be reviewed and approved in accordance with the Quality Assurance Program Topical Report. Revisions shall be made in accordance with the provisions of 10CFR50.59.

*Not applicable in MODE 2 or 3 with all main steam line isolation valves and associated bypass valves in the closed position and all other steam flow paths to the turbine isolated.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

h. Functional Testing of Repaired and Replaced Snubbers

Snubbers which fail the visual inspection or the functional test acceptance criteria shall be repaired or replaced. Replacement snubbers and snubbers which have repairs which might affect the functional test results shall be tested to meet the functional test criteria before installation in the unit. Mechanical snubbers shall have met the acceptance criteria subsequent to their most recent service, and the freedom-of-motion test must have been performed within 12 months before being installed in the unit.

i. Snubber Service Life Program

The service life of hydraulic and mechanical snubbers shall be monitored to ensure that the service life is not exceeded between surveillance inspections. The maximum expected service life for various seals, springs, and other critical parts shall be determined and established based on engineering information and shall be extended or shortened based on monitored test results and failure history. Critical parts shall be replaced so that the maximum service life will not be exceeded during a period when the snubber is required to be OPERABLE. The parts replacements shall be documented and the documentation shall be retained in accordance with Quality Assurance Program Topical Report.

1

ADMINISTRATIVE CONTROLS

6.2.3 Deleted.

6.2.4 SHIFT TECHNICAL ADVISOR

6.2.4.1 The Shift Technical Advisor shall provide advisory technical support to the Shift Supervisor in the areas of thermal hydraulics, reactor engineering, and plant analysis with regard to the safe operation of the unit. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline and shall have received specific training in the response and analysis of the unit for transients and accidents, and in unit design and layout, including the capabilities of instrumentation and controls in the control room.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for:

- a. If the Operations Manager does not hold a senior reactor operator license for Millstone Unit No. 3, then the Operations Manager shall have held a senior reactor operator license at a pressurized water reactor, and the Assistant Operations Manager shall hold a senior reactor operator license for Millstone Unit No. 3.
- b. The Health Physics Manager shall meet or exceed the qualifications of Regulatory Guide 1.8, Revision 1, May 1977.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff that meets or exceeds the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and 10 CFR Part 55.59 shall be maintained.

6.4.2 Deleted.

6.5 Deleted.

PAGES 6-6 THROUGH 6-13 HAVE BEEN INTENTIONALLY DELETED.

ADMINISTRATIVE CONTROLS

6.6 Deleted.

6.7 Deleted.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The applicable procedures required to implement the requirements of NUREG-0737 and supplements thereto;
- c. Refueling operations;
- d. Surveillance activities of safety related equipment;
- e. Not used.

ADMINISTRATIVE CONTROLS

- f. Not used.
 - g. Fire Protection Program implementation;
 - h. Quality controls for effluent monitoring, using the guidance in Regulatory Guide 1.21, Rev. 1, June 1974; and
 - i. Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMDCM) implementation except for Section I.E, Radiological Environmental Monitoring.
- 6.8.2
- a. The designated manager or designated officer or designated senior officer may designate specific procedures and programs, or classes of procedures and programs to be reviewed in accordance with the Quality Assurance Program Topical Report.
 - b. Procedures and programs listed in Specification 6.8.1, and changes thereto, shall be approved by the designated manager or designated officer or by cognizant Manager or Directors who are designated as the Approval Authority by designated manager or designated officer as specified in administrative procedures. The Approval Authority for each procedure and program or class of procedure and program shall be specified in administrative procedures.
 - c. Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved in accordance with the Quality Assurance Program Topical Report, prior to implementation. Each procedure of Specification 6.8.1 shall be reviewed periodically as set forth in administrative procedures.
- 6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:
- a. The intent of the original procedure is not altered;
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and
 - c. The change is documented, reviewed and approved in accordance with the Quality Assurance Program Topical Report within 14 days of implementation.

ADMINISTRATIVE CONTROLS

6.9.1.6.c The core operating limits shall be determined so that all applicable limits (e.g. fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6.d The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the U.S. Nuclear Regulatory Commission, Document Control Desk, Washington, D.C. 20555, one copy to the Regional Administrator Region I, and one copy to the NRC Resident Inspector, within the time period specified for each report.

6.10 Deleted.

ADMINISTRATIVE CONTROLS

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained, and adhered to for all operations involving personnel radiation exposure.

ADMINISTRATIVE CONTROLS

6.13 RADIOLOGICAL EFFLUENT MONITORING AND OFFSITE DOSE CALCULATION MANUAL (REMODCM)

Section I, Radiological Effluent Monitoring Manual (REMM), shall outline the sampling and analysis programs to determine the concentration of radioactive materials released offsite as well as dose commitments to individuals in those exposure pathways and for those radionuclides released as a result of station operation. It shall also specify operating guidelines for radioactive waste treatment systems and report content.

Section II, the Offsite Dose Calculation Manual (ODCM), shall describe the methodology and parameters to be used in the calculation of offsite doses due to radioactive gaseous and liquid effluents and in the calculations of gaseous and liquid effluent monitoring instrumentation alarm/trip setpoints consistent with the applicable LCOs contained in these technical specifications.

Changes to the REMODCM:

- a. Shall be documented and records of reviews performed shall be retained as required by the Quality Assurance Program Topical Report. This documentation shall contain:
 - 1) Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s), and
 - 2) A determination that the change will maintain the level of radioactive effluent control required by 10 CFR 20.106, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose, or setpoint calculations.
- b. Shall become effective after review and acceptance in accordance with the Quality Assurance Program Topical Report.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire REMM or ODCM, as appropriate, as a part of or concurrent with the Annual Radioactive Effluent Report for the period of the report in which any change was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

6.14 RADIOACTIVE WASTE TREATMENT

Procedures for liquid and gaseous radioactive effluent discharges from the Unit shall be prepared, approved, maintained and adhered to for all operations involving offsite releases of radioactive effluents. These procedures shall specify the use of appropriate waste treatment systems utilizing the guidance provided in the REMODCM.

The Solid Radioactive Waste Treatment System shall be operated in accordance with the Process Control Program to process wet radioactive wastes to meet shipping and burial ground requirements.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 239 AND 173

TO FACILITY OPERATING LICENSE NOS. DPR-65 AND NPF-49

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NOS. 2 AND 3

DOCKET NOS. 50-336 AND 50-423

1.0 INTRODUCTION

By letter dated March 5, 1999, the Northeast Nuclear Energy Company, et al. (the licensee), submitted a request for changes to the Millstone Nuclear Power Station, Unit Nos. 2 and 3 Technical Specifications (TSs). The requested changes would relocate certain TS Section 6.0 administrative controls to the NRC-approved Northeast Utilities Quality Assurance Program (NUQAP) Topical Report in accordance with NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specifications Administrative Controls Related to Quality Assurance," dated December 12, 1995. Specifically, Sections 6.2.3, Independent Safety Engineering Group (Unit 3 only); 6.5, Review and Audit; 6.6, Reportable Event Action (partial); 6.7, Safety Limit Violation (partial); and 6.10, Record Retention. The amendments would also delete parts of Section 6.6 and 6.7 because their requirements are duplicated in existing regulations or elsewhere in the TSs. In addition, the amendment would modify the table of contents and other TS sections to incorporate the aforementioned changes (e.g., correct references).

2.0 EVALUATION

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36. That regulation requires that the TSs include items in specific categories, including: (1) safety limits, limiting safety system settings and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TSs.

With respect to limiting conditions for operations (LCOs), 10 CFR 50.36 provides four criteria to be used in determining whether particular safety functions are required to be included in the TSs. In adopting the revision to the Rule, the Commission indicated that the intent of these criteria can be utilized to identify the optimum set of administrative controls in the TSs (60 FR 36957). Addressing administrative controls, 10 CFR 50.36 states that they "are the provisions

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relating to organization and management, procedures, record keeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner." The specific content of the administrative controls section of the TSs is, therefore, that information the Commission deems essential for the safe operation of the facility that is not already adequately covered by other regulations. Accordingly, the staff has determined that requirements that are not specifically required under 10 CFR 50.36(c)(5) and which are not otherwise necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety, can be removed from administrative controls. Existing TS requirements, therefore, may be relocated to more appropriate documents (e.g., Security Plan, QA Plan, and the Emergency Plan) and controlled by the applicable regulatory requirement. Similarly, while the required content of the TSs administrative controls is specified in 10 CFR 50.36(c)(5), particular details of administrative controls may be relocated to licensee-controlled documents where 10 CFR 50.54, 10 CFR 50.59, or other regulations provide adequate regulatory control.

2.1 Independent Safety Engineering Group

The proposed amendment would relocate the Independent Safety Engineering Group (ISEG) functions specified in the existing TS Section 6.2.3 for Millstone Unit 3 only to the NUQAP Topical Report. ISEG functions, responsibilities, and authority, as specified in TS Section 6.2.3, have been relocated intact to Appendix F of the NUQAP Topical Report.

Changes to the NUQAP Topical Report are controlled in accordance with 10 CFR 50.54(a) and include requirements for prior NRC review and approval if a change constitutes a reduction in the NUQAP Topical Report commitment. The staff finds it is not necessary to include redundant or additional requirements in the TSs Administrative Controls section. Therefore, the staff finds that the relocation of the ISEG function from the Millstone Unit 3 TS Section 6.2.3 to the NUQAP Topical Report is consistent with the guidance provided in AL 95-06 and is acceptable.

2.2 Review and Audit

The proposed amendment would relocate the review and audit function specified in existing TS Section 6.5 to the NUQAP Topical Report. The requirements as currently delineated in Section 6.5 of the TSs have been relocated essentially intact (only minor editorial changes were made that do not impact the requirements) to Appendix F of the NUQAP Topical Report.

Changes to the NUQAP Topical Report are controlled in accordance with 10 CFR 50.54(a) and include requirements for prior NRC review and approval if a change constitutes a reduction in the NUQAP Topical Report commitment. Therefore, the staff finds that the relocation of the Review and Audit functions from Section 6.5 of the TS to the NUQAP Topical Report is consistent with the guidance provided in AL 95-06 and is acceptable.

2.3 Record Retention

The licensee proposed to relocate the record retention requirements in Section 6.10 of the TS to the NUQAP Topical Report. All the existing TS requirements from Section 6.10 have been relocated essentially intact (only minor editorial changes were made that do not impact the requirements) to Appendix F of the NUQAP Topical Report.

The staff finds that the regulatory requirements of 10 CFR Part 50, Appendix B provide sufficient control of plant records and sufficient regulatory controls for future changes to the NUQAP Topical Report pursuant to 10 CFR 50.54(a). In addition, other regulations such as 10 CFR 20, Subpart L and 10 CFR 50.71 require the retention of records related to operation of the nuclear power plant. The requirements in the NUQAP Topical Report along with the other regulatory requirements provide sufficient control of record keeping provisions. Therefore, the staff finds the relocation of the Record Retention requirements from Section 6.10 to the NUQAP Topical Report is consistent with the guidance provided in AL 95-06 and is acceptable.

2.4 Other Changes to the Technical Specifications

The licensee proposed other changes to the TSs to: (1) address requirements that are duplicated elsewhere within the TSs, in existing regulations, or portions of which have been relocated to the NUQAP Topical Report, and (2) to correct references to be consistent with the relocation of the administrative requirements to the NUQAP Topical Report.

For Unit 2 only, these changes include:

1. In specification 4.6.1.6.4, the licensee changed the reference to "Specification 6.6.1" to "Section 50.73 to 10 CFR Part 50." Specification 6.6.1 was deleted as discussed below.
2. In specification 4.7.8.g, the licensee changed the reference to "Specification 6.10.2.h" to "Quality Assurance Program Topical Report," for consistency with the relocation of specification 6.10 requirements to the NUQAP Topical Report.

For Unit 3 only, these changes include:

1. In specification 2.1.1, the licensee deleted "and comply with the requirements of Specification 6.7.1," in three locations. Specification 6.7.1 was deleted as discussed below.
2. In specification 4.3.4.2, the licensee changed the reference to "Specification 6.5.1.6.j" to "the Quality Assurance Program Topical Report," for consistency with the relocation of specification 6.5 requirements to the NUQAP Topical Report.
3. In specification 4.7.10.i, the licensee changed the reference to "Specification 6.10.3" to "the Quality Assurance Program Topical Report," for consistency with the relocation of specification 6.10 requirements to the NUQAP Topical Report.

Changes applicable to both Units 2 and 3 include:

1. The licensee deleted specification 6.6, "Reportable Event Action." Specification 6.6.1 referenced 10 CFR 50.73 for report requirements. Specification 6.6.1.a was deleted since it duplicated the NRC's requirement for the licensee to adhere to 10 CFR 50.73. Specification 6.6.1.b was deleted since it duplicated the requirements in Specifications 6.5.1.6.f and 6.5.1.8 that were relocated intact to the NUQAP Topical Report.
2. The licensee deleted specification 6.7, "Safety Limit Violation." Specification 6.7.1.a duplicates requirements in specification 2.1.1. Specification 6.7.1.b, in part, duplicates requirements of 10 CFR 50.36 and 10 CFR 50.72, with the remaining internal licensee notification requirements relocated intact to the NUQAP Topical Report. Specification

6.7.1.c duplicates requirements of 10 CFR 50.36 and 10 CFR 50.73, or specification 6.5.1.6.f that was relocated to the NUQAP Topical Report. Specification 6.7.1.d was relocated intact to the NUQAP Topical Report. Specification 6.7.1.e duplicates the requirements of 10 CFR 50.36.

3. The licensee changed specification 6.8, "Procedures," to reflect that the review and approval of procedures or changes thereto, are conducted in accordance with the requirements of the NUQAP Topical Report for consistency with the relocation of specifications 6.5 requirements.
4. The licensee changed specification 6.15 (Unit 2) and 6.13 (Unit 3), "Radiological Effluent Monitoring and Offsite Dose Calculation Manual (REMODOCM)," to reflect that the review and approval of changes to the REMODOCM and record retention, are conducted in accordance with the requirements of the NUQAP Topical Report for consistency with the relocation of specifications 6.5 and 6.10 requirements.

The staff has reviewed the changes discussed in 2.4 above and determined that these requirements are adequately covered by other regulations, were duplicated in other sections of the TSs, or were appropriately relocated to Appendix F of the NUQAP Topical Report. Changes to the NUQAP Topical Report are controlled in accordance with 10 CFR 50.54(a) and include requirements for prior NRC review and approval if a change constitutes a reduction in the NUQAP Topical Report commitment. The staff finds it is not necessary to include redundant or additional requirements in the TSs Administrative Controls section. Therefore, the staff finds that the TS changes discussed in 2.4 above are acceptable.

3.0 SUMMARY

The staff has evaluated the licensee's proposal related to the revision of the TSs administrative controls associated with sections 6.2.3, Independent Safety Engineering Group (Unit 3 only); 6.5, Review and Audit; 6.6, Reportable Event Action (partial); 6.7, Safety Limit Violation (partial); 6.10, Record Retention; and the table of contents and other TS sections to incorporate the aforementioned changes (e.g., correct references). Based on the above evaluation, the staff concludes that: (1) the proposed QA-related administrative control provisions, as relocated from the current TSs to the NUQAP Topical Report constitute the bases for the licensee's continued compliance with the requirements of Appendix B to 10 CFR Part 50, and (2) the TS sections being relocated to the NUQAP Topical Report are not required to be retained in the TSs pursuant to 10 CFR 50.36. In addition, the staff has reviewed the NUQAP Topical Report and confirmed that the TS Sections have been relocated appropriately. Therefore, the staff finds the proposed changes to the TSs to be acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no

significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (64 FR 17027). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John A. Nakoski

Date: August 13, 1999