



Duke Energy

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W. R. McCollum, Jr.
Vice President

October 8, 2001

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station
Docket 50-269, -270, -287
Selected Licensee Commitments Manual (SLC)

Gentlemen:

Pursuant to 10CFR 50.4 and 50.71, please find attached 7 copies of the latest revisions to the Oconee Selected Licensee Commitments Manual (SLC). The SLC Manual is Chapter 16.0 of the Oconee Updated Final Safety Analysis Report (UFSAR). This manual is intended to contain commitments and other station issues that warrant higher control, but are not appropriate for inclusion into the Technical Specifications (TS). Instead of being updated with the annual UFSAR Update, the SLC Manual will be updated as necessary throughout the year.

Very truly yours,

W. R. McCollum, Jr.
Vice President
Oconee Nuclear Station

CMB/cmb
Attachment

xc: Luis A. Reyes
Regional Administrator, Region II

L. N. Olshan, ONRR

M. C. Shannon
Oconee Senior Resident Inspector

ADD1

October 8, 2001

To: Manual Holders

Subject: Oconee Selected Licensee Commitments Manual (SLC)
Revision

On September 19, 2001, Station Management approved revisions to SLC 16.5.3, "Loss of Decay Heat Removal". This change adds the Containment Isolation design basis document to the reference section of the SLC bases. It also adds information from Generic Letter 88-17.

Remove these pages

SLC LOEP Pages 1-11
SLC Page 16.5.3-1
SLC Page 16.5.3-2
SLC Page 16.5.3-3

Insert these pages

SLC LOEP Pages 1-11
SLC Page 16.5.3-1
SLC Page 16.5.3-2
SLC page 16.5.3-3

Any questions concerning these revisions may be directed to Reese Gambrell at ext. 3364.

Regulatory Compliance
By: Conice Breazeale
Regulatory Compliance

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16.5 REACTOR COOLANT SYSTEM (RCS)

16.5.3 Loss of Decay Heat Removal

COMMITMENT The following conditions shall be met:

- a. Conduct a containment closure survey to identify containment penetrations that would need to be closed in the event of a loss of decay heat removal capability and to ensure that containment closure can be achieved within 2 1/2 hours,
- b. Two operable core exit thermocouple indications and alarm shall be available. The core exit temperature shall be monitored and recorded at least once every two hours,
- c. The LT-5 Reactor vessel level indication system shall be available and operable.
- d. An ultrasonic Reactor vessel level detection system, or other backup level indicating system, shall be available and operable in addition to LT-5.
- e. Both Main Feeder Buses (MFB) shall be energized.
- f. Two sources of power shall be available to supply the Main Feeder buses.
- g. Two of the following means of adding inventory to the RCS are available and operable:
 - 1. A gravity flow path from the BWST
 - 2. One Bleed Transfer Pump (BTP) and connecting piping
 - 3. A High Pressure Injection (HPI) pump
- h. Both steam generators upper primary side handhole covers shall be removed or equivalent RCS vent path opened.

- i. Testing and maintenance activities shall be reviewed to ensure no adverse effects on systems and components required for decay heat removal. Those activities which pose a substantial threat to decay heat removal capability will be prohibited.

-----NOTES-----

1. Commitment b is required to be met only when the reactor vessel head is in place.
 2. Commitment d is a Duke Power internal commitment, not a NRC commitment.
-

APPLICABILITY: MODE 5 and 6 with RCS level < 50 inches above centerline of the reactor vessel hot leg and with irradiated fuel in the reactor vessel.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more commitment not met.	A.1 As determined by Station Manager or Responsible Group Superintendent.	NA

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 16.5.3.1 NA	NA

BASES

Generic Letter 88-17, Loss of Decay Heat Removal, was issued October 17, 1988 by the Nuclear Regulatory Commission (NRC) due to numerous events in the industry involving a loss of decay heat removal capability while the reactor vessel was in a drained down condition. The major concern is that substantial core decay heat may pose a significant likelihood of a release due to a severe core damage accident.

Babcock and Wilcox (B&W) designed plants are not as sensitive to the loss of decay heat removal problems, but the additional actions of this commitment provide added assurance of avoiding possible problems while the reactor vessel is in a reduced inventory condition.

As documented within Commitment (a), a containment closure survey must be performed to identify containment penetrations that would need to be closed in the event of a loss of decay heat removal capability and to ensure that containment closure can be achieved within 2.5 hours. Based on the Duke Energy responses for ONS (Ref. 2) and subsequent NRC review of Duke's responses for ONS to GL 88-17, the following guidance describes the qualifications associated with the containment boundaries to be credited for containment closure in the event of a loss of decay heat removal:

- With the exception of the hatches, the containment boundaries that must be established in the event of a Loss of DHR event are similar to those credited during fuel movement. Thus, with the exception of the hatches, the qualifications of the containment closure boundaries for a loss of decay heat removal event are described within the TS 3.9.3 bases which pertain to containment closure during fuel movement.
- With respect to the equipment hatch, the equipment hatch must be capable of being installed with a minimum number of bolts to establish a boundary seal (i.e. four bolts equally spaced).
- With respect to the airlocks, at least one door of the personnel and emergency hatches must be capable of being closed. Credit is not taken for any temporary covers associated with the personnel or emergency hatches for containment closure during a loss of decay heat removal event.

REFERENCES:

1. Generic Letter 88-17
2. ONS responses to Generic Letter 88-17, dated January 3, 1989 and February 2, 1989.
3. OSS-0254.00-00-4001