February 12, 1985

Docket No. 50-336

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Mr. W. G. Counsil, Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Counsil:

The Commission has issued the enclosed Amendment No. 98 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated December 10, 1984.

This amendment changes the Technical Specifications to authorize the use of an outage equipment door in place of the equipment hatch door during refueling operations.

A copy of our Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

Original signed by:

D. B. Osborne, Project Manager
Operating Reactors Branch #3
Division of Licensing

Enclosures:
1. Amendment No. 98 to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

ORB#3:DL	ORB#3:DL	ORB#3:DL	OELD	AD:OR:DL
PMKreutzer	DBOsborne:dd	JRMiller	Bachmann	GCLainas
2/6/84	2/6/84	2/6/84 for JRM Signed 2/12/85	2/8/84	2/12/84

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 98
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated December 10, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 98 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 12, 1985

ATTACHMENT TO LICENSE AMENDMENT NO.98

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Remove and replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is provided to maintain document completeness.

Remove	Insert
3/4 9-4	3/4 9-4

REFUELING OPERATIONS

CONTAINMENT PENETRATIONS

LIMITING CONDITION FOR OPERATION

3.9.4 The containment penetrations shall be in the following status:

a. The equipment door closed and held in place by a minimum of four bolts or the outage equipment door is installed,

b. A minimum of one door in each airlock is closed, and

c. Each penetration providing direct access from the containment atmosphere to the outside atmosphere shall be either:

1. Closed by an isolation valve, blind flange, manual valve, or special device, or

2. Be capable of being closed by an OPERABLE automatic containment purge valve.

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or movement of irradiated fuel in the containment.

SURVEILLANCE REQUIREMENTS

4.9.4 Each of the above required containment penetrations shall be determined to be either in its isolated condition or capable of being closed by an OPERABLE automatic containment purge valve within 72 hours prior to the start of and at least once per 31 days during CORE ALTERATIONS or movement of irradiated fuel in the containment by:

a. Verifying the penetrations are in their isolated condition, or

b. Testing the containment purge valves per the applicable portions of Specification 4.6.3.1.2.

REFUELING OPERATIONS

DECAY TIME

LIMITING CONDITION FOR OPERATION

3.9.3 The reactor shall be subcritical for a minimum of 72 hours prior to movement of irradiated fuel in the reactor pressure vessel.

APPLICABILITY: MODE 6.

ACTION:

With the reactor subcritical for less than 72 hours, suspend all operations involving movement of irradiated fuel in the reactor pressure vessel.

SURVEILLANCE REQUIREMENTS

4.9.3 The reactor shall be determined to have been subcritical for at least 72 hours by verification of the date and time of subcriticality prior to movement of irradiated fuel in the reactor pressure vessel.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 98 TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT 2

DOCKET NO. 50-336

INTRODUCTION

In a letter dated December 10, 1984, Northeast Nuclear Energy Company (NNECO) proposed an amendment to Operating License No. DPR-65 for Millstone Unit No. 2. The amendment would involve a change to Technical Specification 3.9.4(a) (Containment Penetrations during Refueling Operations) which would authorize use of a temporary, specially designed equipment hatch door (with penetrations) in place of the permanent door (which has no penetrations). The permanent equipment hatch door is described in Section 5.2.7.1.3 of the Millstone Unit No. 2 FSAR. The temporary outage equipment hatch door is required to facilitate steam generator maintenance activities while maintaining primary containment integrity during core alterations or fuel movement.

Evaluation

The Millstone Unit No. 2 containment building includes an equipment hatch, nineteen (19) feet in diameter, to permit transfer of large components into and out of the containment. It is fitted with a double gasket flange around the dished door to minimize leakage in the unlikely event of a loss-of-coolant-accident (LOCA).

Technical Specification 3.9.4 specifies the required status of certain containment penetrations during core alterations or movement of irradiated fuel within the containment. These requirements ensure that a release of radioactive material within the containment will be restricted from leakage to the environment. The radioactive material released from a postulated fuel element rupture would be retained within the building due to the lack of containment pressurization potential while in the refueling mode coupled with the penetration integrity requirements.

The temporary equipment hatch door proposed to be utilized during refueling outages will consist of a circular 1/4 inch thick steel plate with stiffeners. The door will be mounted to the 3/4 inch thick, 8 inch wide, embedded plate which circles the exterior end of the equipment hatch containment penetration.

It will be secured in place by thirteen (13) studs which will be welded to the embedded plate.

The O-ring gasket is located between the temporary door and the embedded plate to provide an "air tight" seal between the containment and the enclosure building. A doorway opening is located in the center of the hatch door for personnel access. During fuel movement the opening is blocked with a steel plate bolted to the hatch door.

Ten (10) six inch diameter penetrations through the temporary door are provided for various hoses and electrical cables. The temporary door is designed to maintain primary containment penetration integrity required by Technical Specification 3.9.4 during core alterations or movement of irradiated fuel within the containment while refueling. The penetrations will be isolated by blind flanges when not in use. Penetrations in which cables or hoses pass through will be sealed with silicone RTV fire-resistant foam to provide the required sealing.

The ten (10) penetrations will consist of one foot long segments of six inch diameter pipe penetrating the door and welded along their perimeters. The silicon foam will surround the hoses and cables and completely fill the one foot pipe segment. The silicon foam is quick drying and will harden to a rubber-like consistency. Testing of this foam shows that it will stay in place and maintain a vapor barrier. The hoses and cables will be supported by scaffolding on both sides of the temporary equipment hatch door. If one of the foam seal barriers fails during fueling operations, the amended Technical Specification will require that all refueling activities be immediately suspended.

The features incorporated into the design of this temporary equipment hatch door ensure that containment integrity will be maintained while core alterations or fuel movements are conducted within the containment considering the negligible pressure gradient which will exist across the equipment hatch penetration. The temporary equipment hatch door was inspected by staff members during a site visit on January 14, 1985.

Based on our review and discussions with the licensee and our inspection of the temporary hatch door, we conclude that the licensee's proposal will provide adequate containment integrity during refueling outages. Therefore, we find the proposed modifications and Technical Specification change acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment

involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 12, 1985

Principal Contributors:
Mark Caruso, ORAB/DL
Dee Osborne