

January 8, 1991

Docket No. 50-336

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Mr. Edward J. Mroczka
 Senior Vice President
 Nuclear Engineering and Operations
 Connecticut Yankee Atomic Power Company
 Northeast Nuclear Energy Company
 P. O. Box 270
 Hartford, Connecticut 06141-0270

Dear Mr. Mroczka:

SUBJECT: ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT -
 EXEMPTION FROM THE REQUIREMENTS OF 10 CFR PART 50, APPENDIX J,
 SECTIONS III.A AND III.C FOR THE MILLSTONE NUCLEAR POWER STATION,
 UNIT NO. 2 (TAC NO. 75970)

Enclosed is a copy of the Environmental Assessment and Finding of No
 Significant Impact which relates to your request for exemption from certain
 requirements of Appendix J to 10 CFR 50, Sections III.A and III.C for
 Millstone Unit No. 2. The application for exemption from the rule was
 dated June 8, 1990.

This assessment has been forwarded to the Office of the Federal Register
 for publication.

Sincerely,

151

John F. Stolz, Director
 Project Directorate I-4
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Enclosure:
 As stated

cc w/enclosure:
 See next page

OFC	:PDI-4:LA	:PDI-4:PM	:PDI-4:D	:OGC*	:
NAME	:SNorris	:GVising/Bah	:JStolz	:BMB	:
DATE	:12/17/90	:12/17/90	:12/18/90	:12/26/90	:

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* OGC concurrence
 subject to noted
 changes discussed
 with and approved
 by the PM

DFD
 1/11

Mr. Edward J. Mroczka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit No. 2

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UNITED STATES NUCLEAR REGULATORY COMMISSION
NORTHEAST NUCLEAR ENERGY COMPANY
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2
DOCKET NO. 50-336
ENVIRONMENTAL ASSESSMENT AND FINDING OF
NO SIGNIFICANT IMPACT

The U. S. Nuclear Regulatory Commission (the Commission) is considering issuance of an exemption from the requirements of Appendix J to 10 CFR Part 50 to Northeast Nuclear Energy Company, et al. (the licensee) for the Millstone Nuclear Power Station, Unit No. 2 located at the licensee's site in New London County, Connecticut.

ENVIRONMENTAL ASSESSMENT

Identification of Proposed Action:

The licensee in a letter dated June 8, 1990, requested an exemption from the requirements of 10 CFR 50, Section III.A and Section III.C of Appendix J to the extent that it requires Type C (local leak rate) testing of containment isolation valves in the reactor building closed cooling water (RBCCW) system.

One of the conditions of all operating licenses for water-cooled power reactors as specified in 10 CFR 50.54(o) is that primary reactor containments shall meet the containment leakage test requirements set forth in 10 CFR 50, Appendix J. Section III of Appendix J contains three subsections, lettered A through C, each of which specifies requirements for a particular aspect of containment leak testing. Sections III.A and III.C are the subjects of the proposed exemption request. Specifically, Section III.A identifies certain components subject to the requirements of Section III.C and Section III.C of

Appendix J identifies leakage testing requirements (Type C Tests) for containment isolation valves that can provide a direct connection between the inside and outside atmospheres of the primary reactor containment under normal operating conditions. The licensee has stated that the exemption would not present an undue risk to the public health and safety.

The Need for the Proposed Action:

The exemption is needed to relieve the licensee of the requirement to provide Type C testing of the 12 containment isolation valves of the RBCCW system at Unit No. 2 of the Millstone Nuclear Power Station.

Environmental Impacts of the Proposed Action:

The proposed exemption from the requirements of Appendix J To 10 CFR Part 50, Sections III.A and III.C involves no changes in plant operation or any accident and thus involves no changes in plant effluents or any changes in the use of resources. The valves in question are required to be opened in case of an accident and thus their leak tight integrity is irrelevant. Since the minimum design pressure of the RBCCW system is 60 psig and since the maximum design pressure of the containment during an accident is 54 psig, in the event a break of the RBCCW inside the containment, the leakage through the closed isolation valves would be inside the containment and thus provide no escape to the environment. If during an accident no RBCCW system were operational, a minimum pressure of 42 psig would be maintained. It is unlikely that the containment pressure would be maintained above 42 psig very long and, therefore, leakage from the containment into the RBCCW would be minimal and escape would be into the enclosure building where it would be collected. Accordingly, the Commission concludes that the proposed action would have no impact on the environment.

Alternatives to the Proposed Action:

The only alternative to the proposed action would be to implement the modifications to assure leak-tight integrity of the RBCCW containment isolation valves in the closed position within the acceptance criteria specified in Appendix J. As discussed above, since there are no environment impacts associated with the proposed action utilizing the only alternative would save any environmental costs.

Alternative use of Resources:

This action does not involve the use of any resources different from or beyond the scope of resources used during plant operation, which were assessed in the Final Environmental Statement relating to plant operation, dated June 1973.

Agencies and Persons Consulted:

The Commission's staff reviewed the licensee's request that supports the proposed exemption. The staff did not consult other agencies or persons.

FINDING OF NO SIGNIFICANT IMPACT:

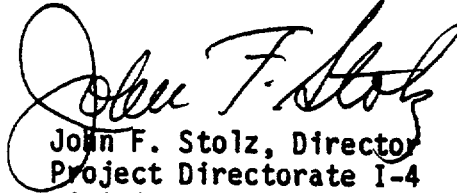
Based upon the foregoing environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed exemption.

For further details with respect to this action, see the request for exemption dated June 8, 1990. A copy is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW,

Washington, D.C. 20555, and at the local public document room located at the Learning Resources Center, Thames Valley State Technical College, 574 New London Turnpike, Norwich, Connecticut 06360.

Dated at Rockville, Maryland this 8th day of January, 1991.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "John F. Stolz". The signature is written in black ink and is positioned above the typed name and title.

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation