

November 16, 1992

Docket No. 50-336

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
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Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M84307)

The Commission has issued the enclosed Amendment No. 165 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated August 17, 1992, as supplemented by letter dated October 30, 1992.

The amendment revises the Technical Specifications (TS) 4.6.1.6.1 to 4.6.1.6.4 related to the prestressed concrete containment surveillance.

We note your commitment, as indicated in your letter of October 30, 1992, to incorporate the applicable portions of the standard technical specifications on containment structural integrity, prior to the next containment tendon surveillance, presently scheduled for 1997.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed
by

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 165 to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

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*See previous concurrence

OFFICE	LA:PDI-4	PM:PDI-4	D:PD I-4	OGC*	
NAME	SNorris	GSVissing:cn	JStolz	CPW	
DATE	11/10/92	11/11/92	11/16/92	11/13/92	1/1

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Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY
DOCKET NO. 50-336
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 165
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated August 17, 1992, as supplemented by letter dated October 30, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

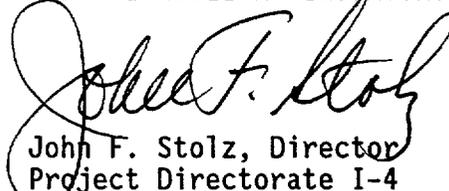
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 165, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 16, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 165

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 6-10
3/4 6-11

Insert

3/4 6-10
3/4 6-11

CONTAINMENT SYSTEMS

CONTAINMENT STRUCTURAL INTEGRITY

LIMITING CONDITION FOR OPERATION

3.6.1.6 The structural integrity of the containment shall be maintained at a level consistent with the acceptance criteria in Specification 4.6.1.6.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the structural integrity of the containment not conforming to the above requirements, restore the structural integrity to within the limits prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.6.1 Containment Tendons The containment tendons' structural integrity shall be demonstrated at the end of one,* three and five years following the initial containment structural integrity test and at five year intervals thereafter. The tendons' structural integrity shall be demonstrated by:

- a. Determining that a representative sample of at least 21 tendons (6 dome, 5 vertical, and 10 hoop) each have a lift off force of between 7030 (minimum) and 8940 (maximum) pounds per tendon wire. If the lift off force of any one tendon in the total sample population is out of the predicted bounds (less than minimum or greater than maximum), an adjacent tendon on each side of the defective tendon shall also be checked for lift off force. If both of these tendons are found acceptable, the surveillance program may proceed considering the single deficiency as unique and acceptable. More than one defective tendon out of the original sample population is evidence of abnormal degradation of the containment structure. Unless there is evidence of abnormal degradation of the containment structure during the first three tests of the tendons, the number of tendons checked for lift off force during subsequent tests may be reduced to a representative random sample of at least 9 tendons (3 dome, 3 vertical and 3 hoop).

*May be extended to no later than midnight, July 15, 1976.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- b. Removing one wire from a dome, a vertical and a hoop tendon checked for lift off force pursuant to Specification 4.6.1.6.1.a and determining that over the entire length of the removed wire that:
1. The tendon wires are free of corrosion.
 2. There are no changes in physical appearance of the sheathing filler grease.
 3. A minimum tensile strength of 11,760 pounds for at least three wire samples (one from each end and one at mid-length) cut from each removed wire. Failure of any one of the wire samples to meet the minimum tensile strength test is evidence of abnormal degradation of the containment structure.

4.6.1.6.2 End Anchorages and Adjacent Concrete Surfaces The structural integrity of the end anchorages and adjacent concrete surfaces shall be demonstrated by determining through inspection that no apparent changes or degradation has occurred in the visual appearance of the end anchorage concrete exterior surfaces or as indicated by the concrete crack patterns adjacent to the end anchorages. Inspections of the concrete shall be performed concurrent with the containment tendon surveillance (reference Specification 4.6.1.6.1).

4.6.1.6.3 Liner Plate The structural integrity of the containment liner plate shall be determined during the shutdown for each Type A containment leakage rate test (reference Specification 4.6.1.2) by a visual inspection of the plate and verifying no apparent changes in appearance or other abnormal degradation.

4.6.1.6.4 Reports In lieu of any other report required by Specification 6.6.1, an initial report of any abnormal degradation of the containment structure detected during the above required tests and inspections shall be made within 10 days after completion of the surveillance requirements of this specification and the detailed report shall be submitted pursuant to Specification 6.9.2 within 90 days after completion. This report shall include a description of the condition of the concrete (especially at tendon anchorages), the inspection procedure, the tolerances on cracking, and the corrective actions taken.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 165

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated August 17, 1992, supplemented by letter dated October 30, 1992, Northeast Nuclear Energy Company (NNECO/the licensee) submitted proposed Technical Specification (TS) changes to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit 2 (Millstone 2). The proposed changes would change Technical Specifications (TS) 4.6.1.6.1 to 4.6.1.6.4 related to prestressed concrete containment surveillance. The October 30, 1992 letter provided information that did not change the initial proposed no significant hazards consideration determination.

The current TS is based on Regulatory Guide (RG) 1.35 Revision 0. The proposed TS revision is to be based on Revision 3 of RG 1.35. However, the licensee's proposed revision adopted only some portions of RG 1.35 Rev.3, not in its entirety. In view of this fact, the staff advised the licensee (Reference 2) to adopt RG 1.35, Rev. 3, in its entirety and provided to the licensee a sample standard TS. In Reference 3, the licensee indicated its commitment to revise its TS in accordance with staff's requirement before the next tendon surveillance scheduled for 1997.

2.0 EVALUATION

The licensee proposed and the staff agreed to the following changes to the current TS:

- a) In section 4.6.1.6.1 "...a representative sample" is changed to ".... a representative random sample...."
- b) In section 4.6.1.6.2 "...adjacent concrete surfaces shall be demonstrated by determining through inspection that no apparent changes have occurred in the visual appearance of the end anchorage concrete exterior surfaces or concrete crack patterns

adjacent to the end anchorages" is changed to "...adjacent concrete surfaces shall be demonstrated by determining through inspection that no apparent changes or degradation has occurred in the visual appearance of the end anchorage concrete exterior surfaces or as indicated by the concrete crack patterns adjacent to the end anchorages"

The staff agreed to the change in a) because it conforms to the same term used in RG 1.35, Revision 3. Because of the change in a) the tendons inspected will not always be the same tendons except for one in each group and, therefore, the word "changes" may not be applicable to all the tendons so the words "or degradation" are added to cover all the tendon conditions. This explanation, as given by the licensee, appeared to be reasonable and the staff accepted the change.

The licensee proposed further revision to section 4.6.1.6.2 of the TS from "Inspections of the concrete shall be performed during the Type A containment leakage rate tests while the containment is at its maximum test pressure" to "Inspections of the concrete shall be performed concurrent with the containment tendon surveillance." This change is in accordance with regulatory position 3.4 of RG 1.35, Rev. 3, and limits only to the inspection of concrete surrounding visually inspected tendon anchorages. The pretest requirements of inspecting the interior and exterior surfaces of the containment as a whole are to be in accordance with 10 CFR Part 50 Appendix J. On the basis of this understanding, the staff agreed to the change. In view of the fact that the licensee has adopted the RG 1.35, Rev. 3, piece-meal, which is against the intent of the regulatory guide, the staff requested the licensee to adopt it in its entirety. The staff provided to the licensee a copy of the standard technical specification for its adoption. The licensee has committed to revise the TS in accordance with the staff's request prior to next tendon surveillance scheduled for 1997.

The staff has reviewed and evaluated the licensee's revision of the TS on containment tendon surveillance in accordance with only some portions of RG 1.35, Rev. 3. It is staff's position that the licensee must adopt RG 1.35, Rev. 3, in its entirety. The licensee has agreed and is committed to revise the TS accordingly prior to next tendon surveillance scheduled for 1997. On the basis of this condition, the staff accepts the licensee's revision to the current TS, which will have impact on actions to be taken during the containment leakage rate tests to be performed before the next tendon surveillance scheduled for 1997.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 40217). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCES

1. Letter to NRC from J. F. Opeka of Northeast Utilities(NU) dated August 17, 1992.
2. Letter to J. F. Opeka of NU from G. S. Vissing of NRC dated October 22, 1992.
3. Letter to NRC from J. F. Opeka of NU dated October 30, 1992.

Principal Contributor: C. P. Tan

Date: November 16, 1992