

December 14, 1992

Docket No. 50-336

Mr. John F. Opeka  
Executive Vice President, Nuclear  
Connecticut Yankee Atomic Power Company  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, Connecticut 06141-0270

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Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M84198)

The Commission has issued the enclosed Amendment No.166 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated August 12, 1992.

The amendment incorporates into the Technical Specifications the addition of two containment isolation valves to the auxiliary feedwater system and changes to Technical Specification Table 3.6-2, "Containment Isolation Valves."

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed  
by

Guy S. Vissing, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No.166 to DPR-65
2. Safety Evaluation

cc w/enclosures:  
See next page

OFFICE	LA:PDI-4	PM:PDI-4	D:PDI-4	OGC <i>gfb</i>	
NAME	<i>SNorris</i>	<i>GSVissing:cn</i>	<i>JStolz</i>	<i>E HOLLER</i>	
DATE	<i>11/30/92</i>	<i>11/30/92</i>	<i>12/11/92</i>	<i>12/17/92</i>	<i>1/1</i>

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Mr. John F. Opeka  
Northeast Nuclear Energy Company

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Unit 2

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY  
THE CONNECTICUT LIGHT AND POWER COMPANY  
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY  
DOCKET NO. 50-336  
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 166  
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated August 5, 1992, as supplemented by letter dated October 20, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

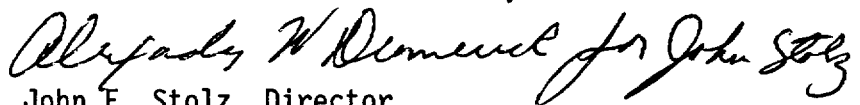
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 166, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: December 14, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 166

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the areas of change.

Remove

3/4 6-18

Insert

3/4 6-18

TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

<u>VALVE NUMBER</u>	<u>FUNCTION</u>	<u>TESTABLE DURING PLANT OPERATION</u>	<u>MAXIMUM ISOLATION TIME</u>
<b>A. CONTAINMENT ISOLATION VALVES</b>			
2-AC-12	Containment Air Sample	Yes	5 seconds
2-AC-15	Containment Air Sample	Yes	5 seconds
2-AC-20	Containment Air Sample	Yes	5 seconds
2-AC-47	Containment Air Sample	Yes	5 seconds
HV-8150	Containment Air Sample	Yes	5 seconds
HV-8151	Containment Air Sample	Yes	5 seconds
2-MS-191A	Steam Generator Sample	Yes	5 seconds
2-MS-191B	Steam Generator Sample	Yes	5 seconds
2-EB-91	Hydrogen Purge	Yes	5 seconds
2-EB-92	Hydrogen Purge	Yes	5 seconds
2-EB-99	Hydrogen Purge	Yes	5 seconds
2-EB-100	Hydrogen Purge	Yes	5 seconds
<b>B. MANUAL</b>			
2-SI-709*	Shutdown Cooling	Yes	Not Applicable
2-SI-463*	Safety Injection Tank Test Line	Yes	Not Applicable
2-SA-19*	Station Air	Yes	Not Applicable
2-RW-21*	Refueling Water Purification	Yes	Not Applicable
2-RW-63*	Refueling Water Purification	Yes	Not Applicable
2-RW-154*	Refueling Water Purification	Yes	Not Applicable
2-RW-232*	Refueling Water Purification	Yes	Not Applicable
2-AC-46*	Hydrogen Monitoring	Yes	Not Applicable
2-AC-51*	Hydrogen Monitoring	Yes	Not Applicable
2-MS-458*	AFW Condensate Drain	Yes	Not Applicable
2-MS-459*	AFW Condensate Drain	Yes	Not Applicable
<b>C. OTHER - NOT APPLICABLE</b>			

\*May be opened on an intermittent basis under administrative control.

MILLSTONE - UNIT 2  
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Amendment No. 166



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 166

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated August 5, 1992, supplemented by letter dated October 20, 1992, Northeast Nuclear Energy Company (NNECO/the licensee) submitted proposed Technical Specification (TS) changes to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit 2 (Millstone 2). The proposed changes would provide the addition of two containment isolation valves to the auxiliary feedwater system and to Technical Specification Table 3.6-2, "Containment Isolation Valves." The October 20, 1992, letter provided information that did not change the initial proposed no significant hazards consideration determination.

The auxiliary feedwater (AFW) system is being modified to include the addition of bypass lines and associated containment isolation valves 2-MS-458 and 2-MS-459. The bypass lines and valves are located upstream of the AFW pump Steam Supply Isolation Valves (SSIV) 2-MS-201 and 2-MS-202. The steam supply lines supply steam to the AFW pump turbine. The bypass lines will provide a drainage path for condensate which collects in the lines during plant start-up and normal plant operation when an SSIV is shut.

During normal plant operation, the bypass lines will be isolated by locking closed the associated isolation valves. Operation of the bypass lines will require operator action under controlled or monitored conditions; i.e., inservice testing or intermittent draining of condensate. In addition, the operating procedures for these valves will require that an operator be present at the applicable bypass line isolation valve during the entire period the valve is not locked closed and will also require that the operator be in constant communication with the control room so that the operator will be instructed to close the valve prior to leaving the area in an accident situation.

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Valves 2-MS-458 and 2-MS-459 and the piping upstream of these valves to the steam supply line for the auxiliary feedwater turbine have been classified ASME Section III, Class 2, and Seismic Class (or Category) I. The piping downstream of these valves is ANSI B31.1 and Seismic Class 2.

The bypass lines and isolation valves are being added to improve the reliability of the main steam and AFW system by reducing the potential for water hammer damage in the main steam piping and the AFW pump turbine during the intermittent operation of the SSIVs. The bypass lines will be isolated during normal plant operation and will experience intermittent service only under controlled conditions.

## 2.0 EVALUATION

The secondary side of the steam generator, along with the steam and feedwater piping inside containment, is considered to form a closed system inside containment. In accordance with General Design Criterion (GDC) 57, a single locked-closed manual valve located outside containment constitutes an acceptable containment isolation provision for a containment penetration associated with a closed system inside containment. Therefore, bypass valves 2-MS-458 and 2-MS-459, which each serve separate containment penetrations, comply with the requirements of GDC 57 in terms of valve type and number of valves required. Considering the anticipated usage of these valves during normal operation to drain condensate from the steam lines, the staff considers remote-manual valves to be a more prudent choice for the containment isolation valves in these lines. However, the administrative controls contained in the licensee's procedure for opening the locked-closed valves during plant operation are consistent with the staff's position concerning such openings, as expressed in Section 1 of Enclosure 1 to Generic Letter 91-08, "Removal of Component Lists from Technical Specifications," dated May 6, 1991. Therefore, the staff finds the type and number of valves to be acceptable.

Further, Regulatory Guide 1.141, "Containment Isolation Provisions for Fluid Systems," and Standard Review Plan 6.2.4, "Containment Isolation System," Revision 2, provide that containment isolation valves, and the piping outside containment and between the containment and the valves, shall meet Quality Group B Standards and Seismic Category I requirements ("Safety Class 2" and "ASME Section III, Class 2" are terms that are, in this case, interchangeable with "Quality Group B"). Therefore, the staff finds the design of valves 2-MS-458 and 2-MS-459 to be acceptable in terms of these design parameters. Thus, these valves satisfy the applicable regulatory requirements.

Based on the above evaluation, the staff finds the proposed addition of containment isolation valves 2-MS-458 and 2-MS-459 to the AFW system and to Technical Specification Table 3.6-2 to be acceptable.

## 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.



#### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 40216). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: J. Pulsipher

Date: December 14, 1992