

DO NOT REMOVE

Docket No. 50-336

P O S T E D

50-336
MILLSTONE 2
AMENDMENT NO. 140
TO DPR-65

Mr. Edward J. Mrocza
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06141-0270

GVISSING
OGC
DHagan
EJordan
BGrimes

Dear Mr. Mrocza:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. 71854)

The Commission has issued the enclosed Amendment No. 140 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your applications dated October 14, 1986, July 21, 1987 and January 12, 1989.

The amendment incorporates Limiting Conditions for Operation and Surveillance Requirements for the Reactor Vessel Coolant Level instrumentation in Technical Specification 3/4.3.3.8, "Instrumentation - Accident Monitoring."

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's bi-weekly Federal Register notice.

Sincerely,

/s/

Guy S. Vissing, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 140 to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

[AMENDMENT TAC NO. 71854]

LA:PDI-4
SN:PTS
4/5/89

PM:PDI-4
GVISSING:cb
4/5/89

PD:PDI-4
JStolz
4/6/89

OGC
E Bachmann
4/10/89
Change as noted
to SE, p. 2



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 21, 1989

Docket No. 50-336

Mr. Edward J. Mrocza
Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06141-0270

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing", is written above the typed name.

Guy S. Vissing, Project Manager
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

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cc w/enclosures:
See next page

Mr. Edward J. Mrocza
Northeast Nuclear Energy Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated October 14, 1986, July 21, 1987 and January 12, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.


2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 21, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 140

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change. The corresponding overleaf pages are provided to maintain document completeness.

Remove

3/4 3-47

3/4 3-48a

B3/4 3-49

Insert

3/4 3-47

3/4 3-48a

B3/4 3-49

TABLE 3.3-11**ACCIDENT MONITORING INSTRUMENTATION**

<u>Instrument</u>	<u>Total No. of Channels</u>	<u>Minimum Channels Operable</u>	<u>Action</u>
1. Pressurizer Water Level	2	1	1
2. Auxiliary Feedwater Flow Rate	2/S.G.	1/S.G.	1
3. RCS Subcooled/Superheat Monitor	2	1	2
4. PORV Position Indicator Acoustic Monitor	1/valve	1/valve	3
5. PORV Block Valve Position Indicator	1/valve	1/valve	3
6. Safety Valve Position Indicator Acoustic Monitor	1/valve	1/valve	3
7. Containment Pressure (Wide Range)	2	1	4
8. Containment Water Level (Narrow Range)	1	1	7##
9. Containment Water Level (Wide Range)	2	1	4
10. Core Exit Thermocouples	4 CETs/core quadrant	2 CETs in any of 2 core quadrants	5
11. Main Steam Line Radiation Monitor	3	3	6
12. Reactor Vessel Coolant Level	2*	1*	8

*A channel is eight (8) sensors in a probe. A channel is operable if four (4) or more sensors, two (2) or more in the upper four and two (2) or more in the lower four, are operable.

##Refer to ACTION statement in Technical Specification 3.4.6.1.

- ACTION 5 -** With the number of OPERABLE accident monitoring instrumentation channels less than the MINIMUM CHANNELS OPERABLE requirements of Table 3.3-11, restore the inoperable channel(s) to OPERABLE status within 48 hours, or begin at least HOT SHUTDOWN within the next 12 hours.
- ACTION 6 -** With any channel of radiation monitoring instrumentation inoperable, portable hand-held radiation detection equipment will be used to assess radiation releases from the atmospheric dump valves and steam generator safeties subsequent to a steam generator tube rupture.
- ACTION 7 -** Restore the inoperable system to OPERABLE status within 7 days or be in COLD SHUTDOWN within the next 36 hours. (See the ACTION statement in Technical Specification 3.4.6.1.):
- ACTION 8 -** With the number of OPERABLE Channels one less than the MINIMUM CHANNELS OPERABLE in Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 48 hours if repairs are feasible without shutting down or:
1. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status; and
 2. Restore the system to OPERABLE status at the next scheduled refueling; and
 3. Initiate an alternate method of monitoring the Reactor Vessel inventory.

TABLE 4.3-7ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Pressurizer Water Level	M	R
2. Auxiliary Feedwater Flow Rate	M	R
3. Reactor Coolant System Subcooled/Superheat Monitor	M	R
4. PORV Position Indicator (Acoustic Monitor)	M	R
5. PORV Block Valve Position Indicator	N.A.	R
6. Safety Valve Position Indicator (Acoustic Monitor)	M	R
7. Containment Pressure	M	R
8. Containment Water Level (Narrow Range)	M	R
9. Containment Water Level (Wide Range)	M	R
10. Core Exit Thermocouples	M	R*
11. Main Steam Line Radiation Monitor	M	R
12. Reactor Vessel Coolant Level	M	R*

*Electronic calibration from the ICC cabinets only.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 140

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

INTRODUCTION

By applications for license amendments dated October 14, 1986, July 21, 1987 and January 12, 1989, Northeast Nuclear Energy Company (the licensee) requested changes to the Technical Specifications (TS) for Millstone Unit 2 to address recommendations of Generic Letter 83-37. The proposed change to the TS would incorporate Limiting Conditions for Operation (LCO) and Surveillance Requirements (SRs) for the Reactor Vessel Coolant Level (RVCL) instrumentation into TS 3/4.3.3.8, "Instrumentation - Accident Monitoring."

DISCUSSION AND EVALUATION

The RVCL instrumentation for Millstone Unit 2 is based upon the heated junction thermocouple technology for post-accident determination of reactor pressure vessel water inventory. In our safety evaluations dated April 18, 1985 and August 28, 1986, the NRC staff addressed the adequacy of the RVCL instrumentation for Millstone Unit 2. The need for RVCL instrumentation and associated TS was one of a number of post-TMI initiatives that had been established by the NRC staff. Based upon discussions with the NRC staff, and applications for license amendments dated October 14, 1986 and July 21, 1987, the licensee has submitted revised proposed LCOs and SRs for the RVCL instrumentation in a letter dated January 12, 1989.

The proposed LCO for the RVCL instrumentation would require at least one of the two channels to be operable. In the event that no channel is operable either restore the inoperable channel(s) to operable status in 48 hours or:

1. prepare and submit a special report to the Commission pursuant to Specification 6.9.2 within 30 days following the event outlining the action taken, the cause of the inoperability, and the plans and schedule for restoring the system to operable status; and

2. Restore the system to operable status at the next scheduled refueling; and
3. Initiate an alternate method of monitoring the reactor vessel inventory.

The SRs for the RVCL instrumentation include monthly channel checks (a determination of operability) and calibration of the instrumentation (from the electronic cabinets only) during refueling. The approval of similar, generic, requirements is contained in a letter from Mr. D. Crutchfield, NRC, to Mr. R. W. Wells, Chairman, Combustion Engineering Owners Group, dated October 28, 1986. The LCOs and SRs contained in this amendment are new limitations imposed on the new post-accident monitoring instrumentation, the RVCL instrument, and are not changes to previous requirements.

Based upon the above, the proposed change to TS 3/4.3.3.8 is acceptable.

ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The staff has previously published a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 21, 1989

Principal Contributor: G. S. Vissing