April 8, 1992

REVISED PER AMENDMENT NO.

Docket No. 50-336

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Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M82742)

The Commission has issued the enclosed Amendment No.156 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated February 7, 1992.

The amendment changes the Technical Specifications by extending the surveillance requirements of Technical Specification 4.6.1.2.a to allow the second Type A Containment Integrated Leakage Rate test, within the second 10-year service period, to be conducted during the Cycle 11 refueling outage scheduled to commence in May 1992. The Technical Specification change is a one time extension for Cycle 11 only. Consistent with this change, the amendment also includes the addition of Surveillance Requirement 4.6.1.2.h which states the provisions of Technical Specification 4.0.2 are not applicable.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No.156to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

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OFFICIAL RECORD COPY Document Name:AMENDCTM

OFC	:LA:PDI-4	:PM:PDI-4	:BC:SPLB	:D:PDI-4	:OGC	:	:
NAME	:SNorris	:GVissing:cn	:CMcCracken	:JStolz	:GHolahan	:	:
DATE	:3/19/92	:3/19/92	:3/19/92	:4/9/92	:4/3/92	:	:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.156
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated February 7, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No156 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Albert W. DeAgazio for

John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 8, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 156

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove	Insert
3/4 6-2	3/4 6-2
3/4 6-4	3/4 6-4

CONTAINMENT LEAKAGE

LIMITING CONDITION FOR OPERATION

3.6.1.2 Containment leakage rates shall be limited to:

a. An overall integrated leakage rate of $\leq L_a$, 0.50 percent by weight of the containment air per 24 hours at P_a, 54 psig.

b. A combined leakage rate of $\leq 0.60\ L_a$ for all penetrations and valves subject to Type B and C tests when pressurized to P_a.

c. A combined leakage rate of $\leq 0.017\ L_a$ for all penetrations identified in Table 3.6-1 as secondary containment bypass leakage paths when pressurized to P_a.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With either (a) the measured overall integrated containment leakage rate exceeding $0.75\ L_a$, or (b) with the measured combined leakage rate for all penetrations and valves subject to Types B and C tests exceeding $0.60\ L_a$, or (c) with the combined bypass leakage rate exceeding $0.017\ L_a$, restore the leakage rate(s) to within the limit(s) prior to increasing the Reactor Coolant System temperature above 200°F.

SURVEILLANCE REQUIREMENTS

4.6.1.2 The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50.

a. Three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10 month intervals* during shutdown at P_a (54 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection.

*The test interval for conducting a Type A test shall be extended to allow the second Type A test, within the second ten-year service period, to be conducted during Cycle 11 refueling outage. This extension expires upon completion of Cycle 11 refueling outage.

SURVEILLANCE REQUIREMENTS (Continued)

g. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select an acceptable integrated leakage measurement system.

h. The provisions of Specification 4.0.2 are not applicable.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO156 TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By application for license amendment dated February 7, 1992, Northeast Nuclear Energy Company (the licensee) requested changes to the Technical Specifications (TS) for Millstone Nuclear Power Station, Unit 2. The amendment would change the TS Sections by extending the surveillance requirements of TS 4.6.1.2.a to allow the second Type A Containment Integrated Leakage Rate test (ILRT), within the second 10-year service period, to be conducted during the Cycle 11 refueling outage scheduled to commence in May 1992. The TS change is a one time extension for Cycle 11 only. Consistent with this change, the amendment would also add a Surveillance Requirement 4.6.1.2.h which states the provisions of Technical Specification 4.0.2 are not applicable.

2.0 EVALUATION

The existing TS 4.6.1.2.a, "Containment Leakage Surveillance Requirements," states that three Type A tests (Overall Integrated Containment Leakage Rate) shall be conducted at 40 ± 10-month intervals during shutdown at Pa (54 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection. The 50-month time limit (April 8, 1992) for the second Type A test within the second 10-year service period would be extended to approximately 56 months due to the number of in-cycle shutdowns which have exended the time period for the next refueling outage to begin in May 1992, and continue until a projected startup in October 1992.

On February 8, 1988, Millstone Unit No. 2 successfully conducted a Type A test, which was the First Type A test in the second 10-year service period, and passed both the "as-found" and "as-left " ILRTs. Since then, there have not been any modifications made to the plant which could adversely affect the test results. Type B and C tests have also been completed during the 1989 and 1990 refueling outages and are scheduled to be performed during the upcoming 1992 refueling outage. The upcoming fuel outage will be an extended outage to accommodate the replacement of steam generators. Demonstrated operability of the components and penetrations, with the local leak rate test program, provides additional assurance that containment integrity has been maintained. The leakage condition of the containment determined from the ILRT of

February 8, 1988, was 36.8% of the TS limit and, as indicated above, no operations are known to have occurred which would suggest a significant degradation of this value.

Since the licensee has planned to conduct the second ILRT during the scheduled shutdown for next refueling and because the licensee has justified the leaktight integrity of the containment based on previous leakage test results, the staff concludes that a one time delay of approximately 6 to 7 months beyond the maximum permitted test interval will not have a significant safety impact. The staff, therefore, concludes that the licensee's requested test interval TS changes for delay in conducting the second ILRT of the second 10-year service period are acceptable.

Technical Specification 4.0.2 allows a 25% increase of a surveillance interval. Since the existing and proposed Millstone Unit 2 Technical Specifications provide for an explicit time interval for Type A test, it is not necessary for TS 4.0.2 to apply. Therefore, we find it acceptable to provide the proposed TS 4.6.1.2.h which states that the provisions of Specification 4.0.2 are not applicable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (57 FR 7812). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. S. Vissing

Date: April 8, 1992