

November 8, 1994

Mr. John F. Opeka  
Executive Vice President, Nuclear  
Connecticut Yankee Atomic Power Company  
Northeast Nuclear Energy Company  
Post Office Box 270  
Hartford, CT 06141-0270

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M90344)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 182 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2, in response to your application dated September 1, 1994.

The amendment revises the Technical Specifications concerning the Reactor Coolant System Volume.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Guy S. Vissing, Senior Project Manager Project  
Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 182 to DPR-65  
2. Safety Evaluation

cc w/encls: See next page

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DATE	10/25/94	10/25/94	10/26/94	10/8/94	10/28/94

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 182 to DPR-65  
2. Safety Evaluation

cc w/encls: See next page

Mr. John F. Opeka  
Northeast Nuclear Energy Company

Millstone Nuclear Power Station  
Unit 2

cc:

Ms. L. M. Cuoco, Senior Nuclear Counsel  
Northeast Utilities Service Company  
Post Office Box 270  
Hartford, Connecticut 06141-0270

Regional Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
475 Allendale Road  
King of Prussia, Pennsylvania 19406

J. J. LaPlatney  
Haddam Neck Unit Director  
Connecticut Yankee Atomic Power Company  
362 Injun Hollow Road  
East Hampton, Connecticut 06424-3099

First Selectmen  
Town of Waterford  
Hall of Records  
200 Boston Post Road  
Waterford, Connecticut 06385

Kevin T. A. McCarthy, Director  
Monitoring and Radiation Division  
Department of Environmental Protection  
79 Elm Street  
Hartford, Connecticut 06106-5127

P. D. Swetland, Resident Inspector  
Millstone Nuclear Power Station  
c/o U.S. Nuclear Regulatory Commission  
Post Office Box 513  
Niantic, Connecticut 06357

Allan Johanson, Assistant Director  
Office of Policy and Management  
Policy Development and Planning Division  
80 Washington Street  
Hartford, Connecticut 06106

Donald B. Miller, Jr.  
Senior Vice President  
Millstone Station  
Northeast Nuclear Energy Company  
Post Office Box 128  
Waterford, Connecticut 06385

S. E. Scace, Vice President  
Nuclear Operations Services  
Northeast Utilities Service Company  
Post Office Box 128  
Waterford, Connecticut 06385

G. H. Bouchard, Nuclear Unit Director  
Millstone Unit No. 2  
Northeast Nuclear Energy Company  
Post Office Box 128  
Waterford, Connecticut 06385

Nicholas S. Reynolds  
Winston & Strawn  
1400 L Street, NW  
Washington, DC 20005-3502

Charles Brinkman, Manager  
Washington Nuclear Operations  
ABB Combustion Engineering  
Nuclear Power  
12300 Twinbrook Pkwy, Suite 330  
Rockville, Maryland 20852

R. M. Kacich, Director  
Nuclear Planning, Licensing & Budgeting  
Northeast Utilities Service Company  
Post Office Box 128  
Waterford, Connecticut 06385

J. M. Solymosy, Director  
Nuclear Quality and Assessment Services  
Northeast Utilities Service Company  
Post Office Box 128  
Waterford, Connecticut 06385



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY  
THE CONNECTICUT LIGHT AND POWER COMPANY  
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY  
DOCKET NO. 50-336  
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 182  
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated September 1, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 182, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director  
Project Directorate I-4  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: November 8, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 182

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contain a vertical line indicating the area of change.

Remove

5-5

Insert

5-5

## DESIGN FEATURES

### VOLUME

5.4.2 The total water and steam volume of the reactor coolant system is a nominal 10,981 ft<sup>3</sup>.

### 5.5 EMERGENCY CORE COOLING SYSTEMS

5.5.1 The emergency core cooling systems are designed and shall be maintained in accordance with the original design provisions contained in Section 6.3 of the FSAR with allowance for normal degradation pursuant to the applicable Surveillance Requirements.

### 5.6 FUEL STORAGE

#### CRITICALITY

5.6.1 a) The new fuel (dry) storage racks are designed and shall be maintained with sufficient center to center distance between assemblies to ensure a  $k_{\text{eff}} \leq .95$ . The maximum nominal fuel enrichment to be stored in these racks is 4.50 weight percent of U-235.

b) Region A of the spent fuel storage pool is designed and shall be maintained with a nominal 9.8 inch center to center distance between storage locations to ensure a  $K_{\text{eff}} \leq .95$  with the storage pool filled with unborated water. Fuel assemblies stored in this region must comply with Figure 3.9-4 to ensure that the design burnup has been sustained.

c) Region B of the spent fuel storage pool is designed and shall be maintained with a nominal 9.8 inch center-to-center distance between storage locations to ensure  $K_{\text{eff}} \leq .95$  with a storage pool filled with unborated water. Fuel assemblies stored in this region may have a maximum nominal enrichment of 4.5 weight percent U-235. Fuel assemblies stored in this region are placed in a 3 out of 4 STORAGE PATTERN for reactivity control.

d) Region C of the spent fuel storage pool is designed and shall be maintained with a 9.0 inch center to center distance between storage locations to ensure a  $K_{\text{eff}} \leq .95$  with the storage pool filled with unborated water. Fuel assemblies stored in this region must comply with Figures 3.9-1a or 3.9-1b to ensure that the design burn-up has been sustained. Additionally, fuel assemblies utilizing Figure 3.9-1b require that borated stainless steel poison pins are installed in the fuel assembly's center guide tube and in two diagonally opposite guide tubes. The poison pins are solid 0.87 inch O.D. borated stainless steel, with a boron content of 2 weight percent boron.

e) Region C of the spent fuel storage pool is designed to permit storage of consolidated fuel and ensure a  $K_{\text{eff}} \leq 0.95$ . The contents of consolidated fuel storage boxes to be stored in this region must comply with Figure 3.9-3.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 182

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated September 1, 1994, the Northeast Nuclear Energy Company (the licensee) submitted a request for a change to the Millstone Nuclear Power Station, Unit No. 2 Technical Specifications (TS). The requested change would change Section 5.4.2 of the TS to reflect a change in the total reactor coolant system (RCS) volume.

2.0 EVALUATION

The steam generators at Millstone Unit No. 2 were replaced during the 1992 refueling outage. This modification resulted in a slight change in the total RCS water and steam volume. The total RCS water and steam volume has been determined to be 10,981 cubic feet, based on nominal conditions with the configuration of the new steam generators, zero tube plugging, and control element assemblies withdrawn.

The proposed change would simply change the volume represented in Section 5.4.2 of the TS. The steam generator replacements were evaluated as a plant design change request in accordance with the requirements of 10 CFR 50.59. The licensee determined that the replacement of the steam generators did not involve an unreviewed safety question. The NRC staff conducted several inspections and reviews regarding the steam generator replacement project and agreed with the licensee that the modification did not constitute an unreviewed safety question.

Based on the above determinations and the minor significance of the slight change in RCS volume, the staff has determined the proposed change to the TS to be acceptable.

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### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 49432). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Vissing

Date: November 8, 1994