

December 20, 1994

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, CT 06141-0270

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M89379)

Dear Mr. Opeka:

The Commission has issued the enclosed Amendment No. 183 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Unit No. 2, in response to your application dated April 22, 1994.

The amendment changes Table 3.3-9 of the Technical Specifications by modifying the indicated measurement range for the neutron flux monitor on the remote shutdown panel. The amendment also includes some corrections of typographical errors in the Technical Specifications.

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures: 1. Amendment No. 183 to DPR-65
2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-336

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2. Safety Evaluation

cc w/encls: See next page

Mr. John F. Opeka
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Millstone Nuclear Power Station
Unit 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY
DOCKET NO. 50-336
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated April 22, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 183, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, to be implemented within 30 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Phillip F. McKee, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: December 20, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 183

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

V
3/4 3-40

Insert

V
3/4 3-40

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

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3/4.2.2	Deleted
3/4.2.3	TOTAL INTEGRATED RADIAL PEAKING FACTOR - F_r^T 3/4 2-9
3/4.2.4	AZIMUTHAL POWER TILT 3/4 2-10
3/4.2.5	Deleted
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 <u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1	REACTOR PROTECTIVE INSTRUMENTATION 3/4 3-1
3/4.3.2	ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION 3/4 3-10
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 <u>3/4.4 REACTOR COOLANT SYSTEM</u>	
3/4.4.1	COOLANT LOOPS AND COOLANT CIRCULATION 3/4 4-1
	Startup and Power Operation 3/4 4-1
	Hot Standby 3/4 4-1a
	Shutdown 3/4 4-1b

TABLE 3.3-9
REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Wide Range Logarithmic Neutron Flux Monitor	Hot Shutdown Panel (C-21)	10 ⁻⁸ % - 100%	1
2. Reactor Trip Breaker Indication	Reactor Trip Switchgear (Q03)	OPEN-CLOSE	1/trip breaker
3. Reactor Cold Leg Temperature	Hot Shutdown Panel (C-21)	0-600°F	1
4. Pressurizer Pressure	Hot Shutdown Panel (C-21)	0-1600 psia	1
a. Low Range		1500-2500 psia	1
b. High Range			
5. Pressurizer Level	Hot Shutdown Panel (C-21)	0-100%	1
6. Steam Generator Pressure	Hot Shutdown Panel (C-21)	0-1200 psia	1/steam generator
7. Steam Generator Level	Hot Shutdown Panel (C-21)	0-100%	1/steam generator



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 183

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated April 22, 1994, the Northeast Nuclear Energy Company (the licensee) submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 2 Technical Specifications (TS). The requested changes would change Table 3.3-9 of the TS by modifying the indicated measurement range for the neutron flux monitor on the remote shutdown panel. The proposed changes would also include some corrections of typographical errors.

2.0 EVALUATION

Panel C-21 is the remote hot shutdown panel that is located outside of the control room. In the event of a forced evacuation the operators are able to maintain the plant in hot shutdown condition with panel C-21, and bring it to cold shutdown with the local operation of certain equipment.

Currently, one of the variables that can be monitored at the C-21 panel is wide range logarithmic neutron flux. The two wide range logarithmic neutron flux monitors are dual scale meters which, when wired to do so, are capable of displaying the neutron source power level in 10^{-1} to 10^4 counts per second (CPS) and the operating power level in $10^{-8}\%$ to 100% power.

The licensee discovered, during a channel check, that panel C-21 did not meet the channel check criterion when placed in the CPS position. An investigation by the licensee, including a review of the original design, indicated that the panel C-21 was never designed or installed with the capability of receiving the CPS input. Therefore, it is the licensee's contention that the CPS scale on the panel C-21 is unnecessary because the range of the percent power can provide the information necessary for safe shutdown; and the entry in TS Table 3.3-9 for the "Wide Range Logarithmic Neutron Flux Monitor" for the CPS scale can be removed. The staff agrees that, with the range of the operating neutron flux monitor, the additional CPS monitor is unnecessary.

The licensee is requesting permission to remove the CPS measurement range requirements for panel C-21. Based on the indication by the licensee that (1) the range of the percent power scale can provide the information necessary for safe plant shutdown and (2) the staff does not have a requirement for CPS input for panel C-21, the staff approves of the proposed change.

With regards to the proposed corrections of typographical errors, the licensee has proposed corrections to the index which the staff has determined to be acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 27059). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: S. Brewer

Date: December 20, 1994