

April 22, 1994

Docket No. 50-336

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

DISTRIBUTION:

Docket File	CGrimes
NRC & Local PDRs	ACRS (10)
PDI-4 Plant	OPA
SVarga	OC/LFDCB
JCalvo	LDoerflein, RI
SNorris	PCampbell
GVising	JNorberg
OGC	RJones
DHagan	CMcCracken
GHill (2)	

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M88973)

The Commission has issued the enclosed Amendment No. 173 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated March 14, 1994.

The amendment changes the Millstone Unit 2 Technical Specifications (TS) to provide a one-time extension of the surveillance frequency from the required 18-month to the next refueling outage but no later than September 30, 1994, of the power operated valves in the service water system (TS 4.7.4.1.b) and in the boron injection flowpath (TS 4.1.2.2.c). This extends the surveillance for these valves approximately 5 months.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

9405030066 940422
PDR ADOCK 05000336
P PDR

Enclosures:

1. Amendment No. 173 to DPR-65
2. Safety Evaluation

cc w/enclosures:

See next page

*per G5
3/24*

OFFICE:	LA:PDI-4	PM:PDI-4	D:PDI-4	BC:EMEB	BC:SRXB
NAME:	SNorris	GVising	JNorberg	JNorberg	RJones for Collins
DATE:	3/24/94	3/24/94	4/20/94	3/28/94	3/31/94

OFFICE:	BC:SPLB	OGC			
NAME:	CMcCracken	EHollen			
DATE:	4/4/94	4/11/94			

DFOI

OFFICIAL RECORD COPY
DOCUMENT NAME: G:\VISSING\M88973.AMD

NRC FILE ORIGINAL COPY

200002

CP

April 22, 1994

Docket No. 50-336

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

DISTRIBUTION:

Docket File	CGrimes
NRC & Local PDRs	ACRS (10)
PDI-4 Plant	OPA
SVarga	OC/LFDCB
JCalvo	LDoerflein, RI
SNorris	PCampbell
GVising	JNorberg
OGC	RJones
DHagan	CMcCracken
Gill (2)	

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M88973)

The Commission has issued the enclosed Amendment No. 173 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated March 14, 1994.

The amendment changes the Millstone Unit 2 Technical Specifications (TS) to provide a one-time extension of the surveillance frequency from the required 18-month to the next refueling outage but no later than September 30, 1994, of the power operated valves in the service water system (TS 4.7.4.1.b) and in the boron injection flowpath (TS 4.1.2.2.c). This extends the surveillance for these valves approximately 5 months.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 173 to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

OFFICE:	LA:PDI-4	PM:PDI-4	D:PDI-4	BC:EMEB	BC:SRXB
NAME:	SNorris	GVising	JStoriz	JNorberg	RJones Collins
DATE:	3/24/94	3/28/94	4/20/94	3/28/94	3/31/94

OFFICE:	BC:SPLB	OGC			
NAME:	CMcCracken	EHollen			
DATE:	4/4/94	4/11/94			



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 22, 1994

Docket No. 50-336

Mr. John F. Opeka
Executive Vice President, Nuclear
Connecticut Yankee Atomic Power Company
Northeast Nuclear Energy Company
Post Office Box 270
Hartford, Connecticut 06141-0270

Dear Mr. Opeka:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M88973)

The Commission has issued the enclosed Amendment No. 173 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2, in response to your application dated March 14, 1994.

The amendment changes the Millstone Unit 2 Technical Specifications (TS) to provide a one-time extension of the surveillance frequency from the required 18-month to the next refueling outage but no later than September 30, 1994, of the power operated valves in the service water system (TS 4.7.4.1.b) and in the boron injection flowpath (TS 4.1.2.2.c). This extends the surveillance for these valves approximately 5 months.

A copy of the related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Guy S. Vissing".

Guy S. Vissing, Senior Project Manager
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 173 to DPR-65
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. John F. Opeka
Northeast Nuclear Energy Company

Millstone Nuclear Power Station
Unit 2

cc:

Gerald Garfield, Esquire
Day, Berry and Howard
Counselors at Law
City Place
Hartford, Connecticut 06103-3499

R. M. Kacich, Director
Nuclear Planning, Licensing & Budgeting
Northeast Utilities Service Company
Post Office Box 270
Hartford, Connecticut 06141-0270

J. M. Solymossy, Director
Nuclear Quality and Assessment Services
Northeast Utilities Service Company
Post Office Box 270
Hartford, Connecticut 06141-0270

J. P. Stetz, Vice President
Haddam Neck Plant
Connecticut Yankee Atomic Power Company
362 Injun Hollow Road
East Hampton, Connecticut 06424-3099

Kevin T. A. McCarthy, Director
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm Street
Hartford, Connecticut 06106-5127

Regional Administrator
Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Allan Johanson, Assistant Director
Office of Policy and Development
Policy Development & Planning Division
80 Washington Street
Hartford, Connecticut 06106

First Selectmen
Town of Waterford
Hall of Records
200 Boston Post Road
Waterford, Connecticut 06385

S. E. Scace, Vice President
Nuclear Operations Services
Northeast Utilities Service Company c/o
Post Office Box 270
Hartford, Connecticut 06141-0270

P. D. Swetland, Resident Inspector
Millstone Nuclear Power Station
U.S. Nuclear Regulatory Commission
Post Office Box 513
Niantic, Connecticut 06357

G. H. Bouchard, Nuclear Unit Director
Millstone Unit No. 2
Northeast Nuclear Energy Company
Post Office Box 128
Waterford, Connecticut 06385

Charles Brinkman, Manager
Washington Nuclear Operations
ABB Combustion Engineering
Nuclear Power
12300 Twinbrook Pkwy, Suite 330
Rockville, Maryland 20852

Nicholas S. Reynolds
Winston & Strawn
11400 L Street, NW
Washington, DC 20005-3502

Donald B. Miller, Jr.
Senior Vice President
Millstone Station
Northeast Nuclear Energy Company
Post Office Box 128
Waterford, Connecticut 06385



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 173
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee), dated March 14, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

9405030070 940422
PDR ADDCK 05000336
P PDR

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 173, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director
Project Directorate I-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 22, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 173

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove

3/4 1-10
3/4 7-12

Insert

3/4 1-10
3/4 7-12

SURVEILLANCE REQUIREMENT

- 4.1.2.2 The above required flow paths shall be demonstrated OPERABLE:
- a. At least once per 7 days by exercising all testable power operated valves in each flow path through at least one complete cycle,
 - b. At least once per 31 days by verifying the correct position of all manually operated valves in the boron injection flow path not locked, sealed or otherwise secured in position, and
 - c. At least once per 18 months*, during shutdown, by exercising all power operated valves in each flow path through at least one complete cycle.
 - d. At least once per 24 hours by verifying that the boric acid piping temperature is greater than 55°F. This may be accomplished by verifying that the ambient temperature in the vicinity of the boric acid piping on elevations (-)5'-0" and (-)25'-6" is greater than 55°F.

*Except that the surveillance requirement due no later than April 23, 1994, may be deferred until the next refueling outage, but no later than September 30, 1994, whichever is earlier.

PLANT SYSTEMS

3/4.7.4 SERVICE WATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.4.1 Two independent service water loops shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one service water loop inoperable, restore the inoperable loop to OPERABLE status within 48 hours or be in COLD SHUTDOWN within the next 36 hours.

SURVEILLANCE REQUIREMENTS

4.7.4.1 Each service water loop shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
 1. Starting (unless already operating) each pump from the control room,
 2. Verifying that each pump develops at least 93% of the discharge pressure for the applicable flow rate as determined from the manufacturer's Pump Performance Curve.
 3. Verifying that each pump operates for at least 15 minutes,
 4. Verifying that each loop is aligned to receive electrical power from separate OPERABLE emergency busses.
 5. Verifying correct position of all valves servicing safety related equipment that are not locked, sealed or otherwise secured in position, and
 6. Exercising all automatically operated valves servicing safety related equipment and testable during plant operation.
- b. At least once per 18 months* by exercising all power operated valves through one complete cycle of full travel.

*Except that the surveillance requirement due no later than May 5, 1994, may be deferred until the next refueling outage, but no later than September 30, 1994, whichever is earlier.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 173

TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated March 14, 1994, the Northeast Nuclear Energy Company (the licensee) submitted a request for changes to the Millstone Nuclear Power Station, Unit No. 2 Technical Specifications (TS). The requested changes would provide a one-time extension of the surveillance frequency from the required 18-month to the next refueling outage but no later than September 30, 1994, of the power-operated valves in the service water system (TS 4.7.4.1.b) and in the boron injection flowpaths (TS 4.1.2.2.c). This would extend the surveillance for these valves approximately 5 months.

Technical Specification 4.1.2.2.c requires that each power-operated valve in each boron injection flowpath be exercised through at least one complete cycle at least once per 18-months, during shutdown. This TS requirement is particularly applicable to valve 2-CH-508 since this valve is the only one in the system that has not satisfied the requirement. Technical Specification 4.7.4.1.b requires each power-operated valve in each of the service water system loops be exercised through one full cycle at least once per 18-months. This TS requirement is particularly applicable to valves 2-SW-3.1A and 2-SW-3.1B since these valves are the only valves in the system that cannot be cycled during operation and have not been cycled since the last refueling outage. Currently, TS 4.1.2.2.c is required to be performed no later than April 23, 1994, for valve 2-CH-508 and TS 4.7.4.1.b is required to be performed no later than May 5, 1994, for valves 2-SW-3.1A and 2-SW-3.1B (these due dates include the 25 percent allowance per Millstone Unit No. 2 TS 4.02).

As a result of an extended Millstone Unit No. 2 Steam Generator Replacement Outage, the start of the 1994 refueling outage was rescheduled from March 1994 to July 1994. As a result, the 18-month surveillance for the power-operated valves in the boron injection flowpath and the power-operated valves of the service water system has been impacted. The proposed change would preclude the necessity of a plant shutdown specifically to perform the surveillance and allow the surveillance to be completed during the next shutdown or no later than September 30, 1994.

9405030072 940422
PDR ADDCK 05000336
P PDR

2.0 EVALUATION

The licensee has indicated that valve 2-CH-508 in the boron injection flowpath was exercised through a complete cycle on March 7, 1994, during a weekly surveillance procedure SP 2601A, "Borated Water Source and Flow Path Verification" while the plant was operating. This surveillance verified the valve's operability; however, the performance of this surveillance did not satisfy the literal compliance with TS 4.1.2.2.c, because it was not performed while the unit was shut down. For all practical purposes, the valve has been verified to be operable and it is acceptable to extend the formal 18-month surveillance to the next refueling or not later than September 30, 1994.

Power-operated valves 2-SW-3.1A and 2-SW-3.1B located on the 'A' and 'B' service water headers, immediately upstream of the service water supply piping to the reactor building closed cooling water heat exchangers, provide isolation of the upstream piping in the event of a significant leak in the piping. The valves are normally open, and are designed to fail in the "as is" condition. The valves do not perform any safety function, nor do they provide isolation between the two service water headers. Their primary function is to isolate the downstream portion of the header for maintenance activities. Past operational history of the service water system valves did not identify any previous problems with the ability of the valves to open or close or to meet any other design requirements. In addition, the licensee has reviewed the proposed one-time extensions for these valves utilizing probabilistic risk assessment techniques and have determined that the one-time increase to the surveillance frequencies would have negligible impact on the overall public risk. The intent of the Technical Specification requirement to cycle the power-operated valves in the service water system is to ensure that the valves actuate on the appropriate safety signal; however, these valves receive no safety signal and would not be subject to the surveillance except that the requirement specifies "all" power-operated valves.

The licensee notes that a short duration shutdown to replace a 'D' reactor coolant pump seal may occur. In discussions with the licensee on March 17, 1994, the staff asked if the two service water valves could be stroked during power operations. Stroking these valves would create a temperature transient on the component cooling water system which cools the reactor coolant pump seals. Any temperature fluctuation could result in seal failure on the 'D' reactor coolant pump seal. Additionally, stroking the valves, even one at a time, removes a service water header from service. It is not the policy to operate the plant at full power conditions with one service water header out of service, even for the short period of time that would be required to stroke the valves. The risk significance of stroking the valves appears to be greater than extending the surveillance.

Based on the above, we determine it acceptable to extend the surveillance frequency of these valves to the next refueling outage or not later than September 30, 1994.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (59 FR 13751). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: G. Vissing
P. Campbell

Date: April 22, 1994