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ASLBP

SEP 3 1975

Docket No. 50-336

Northeast Nuclear Energy Company
ATTN: Mr. Donald C. Switzer
President
P. O. Box 270
Hartford, Connecticut 06101

Amendment No. 1
Change No. 1
License No. DPR-65

Gentlemen:

In response to your requests of August 21, 1975, and August 25, 1975, the Commission has issued Amendment No. 1 to Facility Operating License DPR-65 to change certain of the Technical Specifications, Appendix A of DPR-65. By telephone conversation with Mr. G. L. Johnson of your staff on August 25, 1975, we approved your request. This letter confirms that oral authorization. A signed copy of the amendment is enclosed. A copy of a related notice, which has been forwarded to the Office of the Federal Register for filing and publication, is also enclosed.

Amendment No. 1 to DPR-65 consists of Change No. 1 to the Technical Specifications. The specifications that have been changed are listed in Change No. 1. A copy of the staff's safety evaluation is also enclosed.

Change No. 1 consists of (1) the correction of certain proofreading errors and (2) a special test exception that suspends the specification which limits the rate of recovery of the water level in the steam generator. This special test exception permits the performance of tests during hot functional testing to verify the adequacy of the feedwater sparger modifications that were installed to prevent the occurrence of water hammer in the feedwater system piping. This special test exception was granted since, as issued August 1, 1975, Facility Operating License DPR-65 restricts you to load fuel and to perform post-core hot functional testing (subcritical only).

The staff has concluded that Change No. 1 will not present any danger to the health and safety of the public nor will it result in (i) any significant increase in the probability of an accident or (ii)

g/p
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[Signature]

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Northeast Nuclear Energy
Company

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a significant increase in the consequences of an accident or (iii) a significant decrease in a safety margin.

Sincerely,

Original Signed By
O. D. Parr

Olan D. Parr, Chief
Light Water Reactors
Project Branch 1-3
Division of Reactor Licensing

Enclosures:

1. Amendment No. 1 to DPR-65
2. Staff Safety Evaluation
3. Federal Register Notice

cc: See page 3

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DATE	8/29/75	8/29/75	9/3/75			

Northeast Nuclear Energy
Company

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cc: William H. Cuddy
Day, Berry & Howard
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Hartford, Connecticut 06103

Mr. Anthony E. Wallace, President
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Mr. J. R. McCormick, President
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Hartford, Connecticut 06101

Mr. Leon F. Maglathlin, Vice President
and Chief Administrative Officer
Western Massachusetts Electric Company
174 Brush Hill Avenue
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Mr. Horace H. Brown
Director of Planning
Federal-State Relations
Department of Finance & Control
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Mr. Herbert J. Davis
First Selectman
Town of Waterford, Hall of Records
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Mr. David Burack, Study Manager
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270 Orange Street
New Haven, Connecticut 06511

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Northeast Nuclear Energy
Company

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cc: Mr. Wallace Stickney
Environmental Protection Agency
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Mr. Bruce Blanchard
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Chief, TIRB (2)
Technology Assessment Division
Office of Radiation Programs
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Mr. Sheldon Myers
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Colonel Howard Sargent
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

THE CONNECTICUT LIGHT AND POWER COMPANY,
THE HARTFORD ELECTRIC LIGHT COMPANY,
WESTERN MASSACHUSETTS ELECTRIC COMPANY, AND
NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-336

(Millstone Nuclear Power Station, Unit 2)

FACILITY OPERATING LICENSE

License No. DPR-65
Amendment No. 1

1. The Nuclear Regulatory Commission (the Commission) having found that:
 - A. The application for the amendment filed by The Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, and Northeast Nuclear Energy Company (the licensees) dated August 21, 1975 and supplement thereto dated August 25, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
 - B. The facility will operate in conformity with the application, as supplemented, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amended operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-65 is hereby amended to read as follows:

2.C(2) Technical Specifications

The Technical Specifications contained in Appendices A and B issued on August 1, 1975, as revised, are hereby incorporated in this amended license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 1. A copy of Change No. 1 is enclosed with this amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by
D. B. Vassallo

R. C. DeYoung, Assistant Director
for Light Water Reactors Group 1
Division of Reactor Licensing

Enclosure:
Change No. 1 to the
Technical Specifications

Date of Issuance: SEP 3 1975

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CHANGE NO. 1
TO
TECHNICAL SPECIFICATIONS
APPENDIX A TO
LICENSE DPR-65

1. In Table 3.3-5, change the RESPONSE TIME IN SECONDS for CSAS/ Containment Spray (Item 1.b) from ≤ 6.5 to ≤ 18.5 .
2. In Table 3.3-5, change the RESPONSE TIME IN SECONDS for the Containment Pressure -- High - High (Item 4.a) from ≤ 8.5 to ≤ 20.5 .
3. In specification 4.6.5.1b.1, change "the DOP" to "a halogenated hydrocarbon refrigerant test gas".
4. Add the following Special Test Exception 3/4 10.6:

SPECIAL TEST EXCEPTIONS

STEAM GENERATOR WATER ADDITION

LIMITING CONDITION FOR OPERATION

3.10.6 The requirement of 3.7.1.6 relating to a recovery rate limit may be suspended during the performance of tests to determine additional data to verify the adequacy of the feedwater sparger modifications provided:

- a. No sustained water hammer in the feedwater system piping is allowed to exist.

Applicability: Whenever the secondary water level in a steam generator is below the elevation of the feedwater sparger.

Action: a. If water hammer begins, immediately terminate feed-water flow to the steam generator.

SURVEILLANCE REQUIREMENTS

4.10.6 The absence of sustained water hammer shall be verified by the following whenever the steam generator water level is below the elevation of the sparger:

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- a. Continuous aural monitoring in the vicinity of the containment side feedwater piping to the affected steam generator.
- b. Continuous visual monitoring of the output of temporary instrumentation installed for monitoring of the feedwater piping. Water hammer is indicated by oscillations in the piping.

Add the following Bases for Special Test Exception 3.10.6/4.10.6:

SPECIAL TEST EXCEPTIONS

BASES

3/4 10.6 STEAM GENERATOR WATER ADDITION

This special test exception permits the water level recovery rate in the steam generator to exceed 168 gpm. This special test exception is required to demonstrate the effectiveness of feedwater system modifications which have been accomplished to minimize the effects of "feedwater hammer".

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SAFETY EVALUATION BY OFFICE OF
NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 1 TO LICENSE NO. DPR-65
CHANGE NO. 1 TO TECHNICAL SPECIFICATIONS
THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
THE NORTHEAST NUCLEAR ENERGY COMPANY
MILLSTONE NUCLEAR POWER STATION UNIT 2
DOCKET NO. 50-336

INTRODUCTION

By letter dated August 21, 1975, the Licensee requested certain changes to the Technical Specifications for the Millstone Nuclear Power Station, Unit 2. The changes, which apply to Appendix A to the license, would correct proof reading errors, and permit verification of the adequacy of the feedwater sparger modifications that were installed to prevent the occurrence of water hammer in the feedwater system piping. By letter dated August 25, 1975, the Licensee modified the proposed special test exception.

The staff's evaluation of the changes is discussed below. The staff's conclusions follow the discussion.

DISCUSSION

The Licensee has proposed changes in Specification 3.3.2.1, Table 3.3-5, governing the response time testing of the containment spray actuation system/containment spray (Item 1.b) and the containment pressure high-high/containment spray functions to correct errors made in the development of the original specification. The proposed revised response times of 18.5 seconds and 20.5 seconds for Items 1.b and 4.a,

respectively, are consistent with the assumptions of the loss-of-coolant accident analysis for Millstone Unit 2 submitted on April 21, 1975. We find that the proposed changes to Table 3.3-5 do not create a significant hazards consideration and would not endanger the health and safety of the public. Therefore, we conclude that the requested change in Specification 3.3.2.1, Table 3.3-5, should be authorized.

The Licensee has requested a change in the type of test performed on the charcoal adsorbers required by Specification 4.6.5.1b.1 in order to correct a typographical error made in the original specification. The Licensee has requested that this requirement be changed to require the performance of halogenated hydrocarbon tests on the charcoal adsorbers. This proposed change is consistent with the recommendations of Regulatory Guide 1.52. We find that the proposed change to Specification 4.6.5.1b.1 does not create a significant hazards consideration and would not endanger the health and safety of the public. Therefore, we conclude that the requested change in Specification 4.6.5.1b.1 should be authorized.

Specification 3.7.1.6 presently limits the steam generator water level recovery rate to 168 gallons per minute above and beyond the flow necessary to maintain a water level in steam generator. The Licensee proposed a special test exception 3/4 10.6 to permit the performance of tests during hot functional testing to verify the adequacy of the feedwater sparger modifications to prevent the occurrence of water

hammer. Subsequently, the Licensee modified the proposed test exception to allow the feedwater addition rate during testing to be equal to the auxiliary feedwater pump capacity.

We have reviewed the proposed test exception. We believe that the test criteria and required action provide adequate assurance that, should water hammer be encountered during the test, no hazard to the public health and safety will result. Therefore, we have no objections to the Licensee's proposed test exception. However, we believe that this test, if performed during post-core hot functional testing, will not meet the requirements of specification 3.7.1.6 regarding the basis for a revision of the limit on steam generator water level recovery rate. The test which meets this requirement should be performed with adequate decay heat to simulate operation conditions.

We find that the proposed special test exception 3/4 10.6 does not create a significant hazards consideration and would not endanger the health and safety of the public. Therefore, we conclude that the proposed test exception 3/4 10.6 should be authorized.

CONCLUSIONS

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the

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change does not involve a significant hazards consideration; (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

The changes requested by the Licensee in the August 21, 1975 letter, and as modified by the August 25, 1975 letter, are hereby granted by changing the Technical Specifications as provided in the attached "Change No. 1 to Technical Specifications DPR-65."



R. C. DeYoung, Assistant Director
for Light Water Reactors Group 1
Division of Reactor Licensing

August 25, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-336

THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY, AND
NORTHEAST NUCLEAR ENERGY COMPANY
(Millstone Nuclear Power Station, Unit 2)

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

Notice is hereby given that the Nuclear Regulatory Commission (the Commission) has issued Amendment No. 1 to Facility Operating License No. DPR-65 issued to The Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, and Northeast Nuclear Energy Company (licensees). This amendment is effective as of the date of issuance.

The amendment corrects certain proofreading errors and grants a special test exception to permit the performance of tests during hot functional testing to verify the adequacy of the feedwater sparger modifications that were installed to prevent the occurrence of water hammer in the feedwater system piping.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings required by the Act and the Commission's rules and regulations in 10 CFR Chapter 1, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated August 21, 1975 and supplement thereto dated August 25, 1975, (2) Amendment No. 1 to License No. DPR-65, with any attachments, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Waterford Public Library, Rope Ferry Road, Waterford, Connecticut. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 3rd day of September 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By

O. D. Parr

Olan D. Parr, Chief
Light Water Reactors
Project Branch 1-3
Division of Reactor Licensing