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 DFoster (w/OL only)  
 JRBuchanan, ORNL  
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Docket No. 50-336

SEP 19 1975

Northeast Nuclear Energy Company  
 ATTN: Mr. Donald C. Switzer  
 President  
 P. O. Box 270  
 Hartford, Connecticut 06101

Amendment No. 3  
 Change No. 3  
 License No. DPR-65

Gentlemen:

In response to your requests of September 9, 1975 and September 12, 1975, the Commission has issued Amendment No. 3 to Facility Operating License No. DPR-65 to change certain of the Technical Specifications, Appendix A of License No. DPR-65. A signed copy of the amendment is enclosed. A copy of a related notice, which has been forwarded to the Office of the Federal Register for publication, is also enclosed.

Amendment No. 3 to License No. DPR-65 consists of Change No. 3 to the Technical Specifications. The specifications that have been changed are listed in Change No. 3. A copy of the staff's safety evaluation is also enclosed.

Change No. 3 consists of (1) an increase in the flow rate required for operation of the enclosure building filtration system during certain modes of plant operation, (2) a revision of the specification regarding main steam line safety valve operability to delete the action statement allowing plant operation with inoperable steam generator safety valves, and (3) the correction of certain proofreading errors.

The staff has concluded that Change No. 3 does not represent a significant hazards consideration, and therefore will not present any danger to the health and safety of the public.

Sincerely,

Original Signed By  
 O. D. Parr  
 Olan D. Parr, Chief  
 Light Water Reactors  
 Project Branch 1-3  
 Division of Reactor Licensing

- Enclosures:  
 1. Amendment No. 3 to DPR-65  
 2. Staff Safety Evaluation

OFFICE	3. FEDERAL REGISTER Notice	RL:LWR 1-3	OELD <i>MC</i>	RL:LWR 1-3
SURNAME	cc: See page 2	PO"Reilly:mb	J.S. COHEN	ODParr
DATE		9/18/75	9/19/75	9/19/75

cc: William H. Cuddy  
Day, Berry & Howard  
Counselors at Law  
One Constitution Plaza  
Hartford, Connecticut 06103

Mr. Anthony E. Wallace, President  
The Connecticut Light & Power Company  
P. O. Box 2010  
Hartford, Connecticut 06101

Mr. J. R. McCormick, President  
The Hartford Electric Light Company  
P. O. Box 2370  
Hartford, Connecticut 06101

Mr. Leon F. Maglathlin, Vice President  
and Chief Administrative Officer  
Western Massachusetts Electric Company  
174 Brush Hill Avenue  
West Springfield, Massachusetts 01089

Mr. Horace H. Brown  
Director of Planning  
Federal-State Relations  
Department of Finance & Control  
340 Capitol Avenue  
Hartford, Connecticut 06115

Mr. Herbert J. Davis  
First Selectman  
Town of Waterford, Hall of Records  
200 Boston Post Road  
Waterford, Connecticut 06385

Mr. David Burack, Study Manager  
Long Island Sound Regional Study  
270 Orange Street  
New Haven, Connecticut 06511

Mr. Wallace Stickney  
Environmental Protection Agency  
JFK Federal Building  
Boston, Massachusetts 02203

OFFICE						
SURNAME						
DATE						

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

THE CONNECTICUT LIGHT AND POWER COMPANY,  
THE HARTFORD ELECTRIC LIGHT COMPANY,  
WESTERN MASSACHUSETTS ELECTRIC COMPANY, AND  
NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-336

(Millstone Nuclear Power Station, Unit 2)

FACILITY OPERATING LICENSE

License No. DPR-65  
Amendment No. 3

1. The Nuclear Regulatory Commission (the Commission) having found that:
  - A. The applications for amendment filed by The Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, and Northeast Nuclear Energy Company (the licensees) dated September 9, 1975 and September 12, 1975 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I and all required notifications to other agencies or bodies have been duly made;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance: (i) that the activities authorized by this amended operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the rules and regulations of the Commission;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the enclosure to this license amendment and Paragraph 2.C(2) of Facility License No. DPR-65 is hereby amended to read as follows:

2.C(2) Technical Specifications

The Technical Specifications contained in Appendices A and B issued on August 1, 1975, as revised, are hereby incorporated in this amended license. The licensees shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 3. A copy of Change No. 3 is enclosed with this amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By

O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors  
Project Branch 1-3  
Division of Reactor Licensing

Enclosure:

Change No. 3 to the  
Technical Specifications

Date of Issuance: SEP 19 1975

SEP 19 1975

CHANGE NO. 3 TO TECHNICAL SPECIFICATIONS  
APPENDIX A TO LICENSE NO. DPR-65

1. In Specification 3.7.1.1 change the Action Statement to read as follows:  
"With one main steam line code safety valve inoperable, restore the inoperable valve to OPERABLE status within 4 hours or be in COLD SHUTDOWN within the next 36 hours."

In Specification 4.7.1.1, delete Table 3.7-1.

In Basis 3/4.7.1.1, delete the remainder of the basis following the sentence which ends with the phrase "at 100% RATED THERMAL POWER."

2. In Specification 4.6.5, Items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$  10%" to "9000 cfm  $\pm$  10%".

In Specification 4.9.15, items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$  10%" to "9000 cfm  $\pm$  10%".

3. Change the specifications identified in the following table as indicated to correct proofreading errors.

SPECIFICATION NO.	TITLE	PAGE NO.	PAGE REV.	CHANGE
-	Index	VII	0	Add "Steam Generator Water Addition ... ... 3/4 7-5" to index
-	Index	VIII	0	Remove date from lower right corner of page
-	Index	XIII	0	Remove date from lower right corner of page
3.1.2.8	Reactivity Control Systems, Borated Water Sources	3/4 1-18	0	Change "Stroage" to "Storage" in 3.1.2.8.a and change "Operabel" to "Operable" in the action statement
3.1.3.2	Reactivity Control Systems, Part Length	3/4 1-23	0	In footnote, change "Exceptions" to "Exception"
3.1.3.5	Reactivity Control Systems, Shutdown CEA Insertion Limit	3/4 1-27	0	In footnote, change "Exceptions" to "Exception"
3.1.3.6	Reactivity Control Systems, Regulating CEA Insertion Limits	3/4 1-28	0	In Action Statement b.1, remove comma after "of Figure 3.1-2" and change "Exceptions" to "Exception" in footnote.
3.7.1.6	Steam Generator Water Addition	3/4 7-5	0	In Action Statement change "Throtte" to "Throttle"
3.7.7.1	Plant Systems - Sealed Source Contamination	3/4 7-19	0	In first line of 3.7.7.1 insert "either" between "material" and "in"

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SPECIFICATION  
NO.

TITLE

PAGE  
NO.

PAGE  
REV.

CHANGE

3.8.1.1

Electrical Power  
Systems - AC Sources

3/4  
8-1

0

In fourth line of  
Action a, change  
referenced sections  
to "4.8.1.1.1 and  
4.8.1.1.2.a.3" (to  
correct section num-  
ber).

3.8.1.1

Electrical Power  
Systems - AC Sources

3/4  
8-2

0

In third line of  
Action c, change  
referenced section  
to "4.8.1.1.2.a.3"  
and in third line of  
Action d, change  
referenced section to  
"4.8.1.1.1".

SEP 19 1975

CHANGE NO. 3 TO TECHNICAL SPECIFICATIONS  
APPENDIX A TO LICENSE NO. DPR-65

1. In Specification 3.7.1.1 change the Action Statement to read as follows:  
"With one main steam line code safety valve inoperable, restore the inoperable valve to OPERABLE status within 4 hours or be in COLD SHUTDOWN within the next 36 hours."

In Specification 4.7.1.1, delete Table 3.7-1.

In Basis 3/4.7.1.1, delete the remainder of the basis following the sentence which ends with the phrase "at 100% RATED THERMAL POWER."

2. In Specification 4.6.5, Items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$  10%" to "9000 cfm  $\pm$  10%".

In Specification 4.9.15, items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$  10%" to "9000 cfm  $\pm$  10%".

3. Change the specifications identified in the following table as indicated to correct proofreading errors.



SPECIFICATION NO.	TITLE	PAGE NO.	PAGE REV.	CHANGE
-	Index	VII	0	Add "Steam Generator Water Addition ... ... 3/4 7-5" to index
-	Index	VIII	0	Remove date from lower right corner of page
-	Index	XIII	0	Remove date from lower right corner of page
3.1.2.8	Reactivity Control Systems, Borated Water Sources	3/4 1-18	0	Change "Stroage" to "Storage" in 3.1.2.8.a and change "Operabel" to "Operable" in the action statement
3.1.3.2	Reactivity Control Systems, Part Length	3/4 1-23	0	In footnote, change "Exceptions" to "Exception"
3.1.3.5	Reactivity Control Systems, Shutdown CEA Insertion Limit	3/4 1-27	0	In footnote, change "Exceptions" to "Exception"
3.1.3.6	Reactivity Control Systems, Regulating CEA Insertion Limits	3/4 1-28	0	In Action Statement b.1, remove comma after "of Figure 3.1-2" and change "Exceptions" to "Exception" in footnote.
3.7.1.6	Steam Generator Water Addition	3/4 7-5	0	In Action Statement change "Throtte" to "Throttle"
3.7.7.1	Plant Systems - Sealed Source Contamination	3/4 7-19	0	In first line of 3.7.7.1 insert "either" between "material" and "in"

SEP 19 1975

PECIFICATION  
NO.

TITLE

PAGE  
NO.

PAGE  
REV.

CHANGE

3.8.1.1

Electrical Power  
Systems - AC Sources

3/4  
8-1

0

In fourth line of  
Action a, change  
referenced sections  
to "4.8.1.1.1 and  
4.8.1.1.2.a.3" (to  
correct section num-  
ber).

3.8.1.1

Electrical Power  
Systems - AC Sources

3/4  
8-2

0

In third line of  
Action c, change  
referenced section  
to "4.8.1.1.2.a.3"  
and in third line of  
Action d, change  
referenced section to  
"4.8.1.1.1".

For further details with respect to this action, see (1) the applications for amendment dated September 9, 1975 and September 12, 1975, (2) Amendment No. 3 to License No. DPR-65, with any attachments, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Waterford Public Library, Rope Ferry Road, Waterford, Connecticut. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 19<sup>th</sup> day of September 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By  
O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors  
Project Branch 1-3  
Division of Reactor Licensing

SEP 19 1975

SAFETY EVALUATION BY OFFICE OF  
NUCLEAR REACTOR REGULATION  
SUPPORTING AMENDMENT NO. 1 TO LICENSE NO. DPR-65  
CHANGE NO. 1 TO TECHNICAL SPECIFICATIONS  
THE CONNECTICUT LIGHT AND POWER COMPANY  
THE HARTFORD ELECTRIC LIGHT COMPANY  
WESTERN MASSACHUSETTS ELECTRIC COMPANY  
THE NORTHEAST NUCLEAR ENERGY COMPANY  
MILLSTONE NUCLEAR POWER STATION UNIT 2  
DOCKET NO. 50-336

INTRODUCTION

By letters dated September 9, 1975 and September 12, 1975, the Licensee requested certain changes to the Technical Specifications for the Millstone Nuclear Power Station, Unit 2. The changes, which apply to Appendix A to the license, would increase the flow rate required for operation of the enclosure building filtration system during certain modes of plant operation, revise the specification regarding main steam line safety valve operability to delete the action statement allowing plant operation with inoperable steam generator safety valves, and correct certain proofreading errors.

The Staff's evaluation of the changes is discussed below. The staff's conclusions follow the discussion.

DISCUSSION

By letter dated September 9, 1975, the Licensee requested changes in Specifications 3/4.7.1.1. These changes would delete any reference in Specifications 3/4.7.1.1 regarding plant operation with inoperable steam generator safety valves. The changes in these specifications were made necessary because of calculational errors which occurred in the

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development of Table 3.7-1. This resulted in the calculation of incorrect valves for the maximum allowable power level-high trip set-points with inoperable steam line safety valves during operation with both steam generators. At the present time, discussions between the staff and the Licensee are continuing concerning the exact valves for allowable core thermal power as a function of the number of inoperable steam generator safety valves. In order to avoid any delay in the staff's review of an amendment to the operating license which would authorize power operation, the Licensee proposes to delete any reference in Specifications 3/4.7.1.1 to operation with inoperable steam generator safety valves.

The concept of allowing plant operation with inoperable steam generator safety valves at appropriately reduced power levels was introduced by the staff during the development of the Millstone Unit 2 Technical Specifications. The Licensee has chosen not to exercise this operational option at this time. Instead, Millstone Unit 2 will not be allowed to operate with an inoperable steam generator safety valve. The proposed change is conservative with respect to the safety analysis. Therefore, we find that the proposed change to Specifications 3/4.7.1.1 does not create a significant hazard consideration and would not endanger the health and safety of the public. On this basis, we conclude that the requested change to Specifications 3/4.7.1.1 should be authorized.

By letter dated September 12, 1975, the Licensee requested changes in Specifications 4.6.5.1 and 4.9.15. These changes would increase the flow rate required for operation of the enclosure building filtration system to be consistent with the current operational capacity of this

system.

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Originally, the enclosure building filtration system was designed for a flow rate of 6000 cubic feet per minute with the additional capability to increase this flow rate by 50 percent to balance the system or to satisfy other system functions. This capability is described in Section 6.7.2.1 of the Millstone Unit 2 FSAR. In order to meet the required flow rate for the hydrogen purge system of 50 cubic feet per minute (identified in Section 6.6.3.1 of the FSAR), which is furnished by the enclosure building filtration system during initial testing, it was necessary to increase the speed of the enclosure building filtration system fans. The increased fan speed has resulted in an observed flow rate in the enclosure building filtration system of about 9000 cubic feet per minute.

In order to provide proper face velocities through the charcoal filters in the enclosure building filtration system, the Licensee has added 50 percent more elements to each filter bank. The Licensee states that the FSAR will be revised at a later date to reflect this revision.

The original safety analysis (described in Section 14.18 of the FSAR and the staff's independent evaluation reported in Section 15-2 of the Millstone Unit 2 Safety Evaluation Report) assumed a minimum flow rate of 6000 cubic feet per minute in the enclosure building filtration system. However, all components in the system were selected considering the 9000 cubic feet per minute flow rate as a specification requirement. The adsorber section of the system is not degraded because the face velocities remain the same due to the increased charcoal surface area. In addition, the enclosure building filtration region drawdown rate (which determines the amount of direct leakage of fission products) will be increased

due to the greater volumetric flow rates. As a result, the increased

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flow rate of 9000 cubic feet per minute would result in a reduction in the amount of direct outleakage of fission products from the containment compared to those calculated. We find that the proposed change to Specifications 4.6.5.1 and 4.9.15 does not create a significant hazard consideration and would not endanger the health and safety of the public. Therefore, we conclude that the requested change to Specifications 4.6.5.1 and 4.9.15 should be authorized.

In addition, the Licensee identified in the September 12, 1975 letter a number of proofreading errors in the Technical Specifications and requested that these be corrected. We find that the proposed changes do not create a significant hazard consideration. Therefore, we conclude that the requested changes to correct proofreading errors should be authorized. These changes are identified in the enclosed change No. 3 to the Millstone Unit 2 Technical Specifications.

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SURNAME ➤	PDO Reilly	J.S. COHEN	ODParr			
DATE ➤	9/18/75	9/19/75	9/19/75			

CHANGE NO. 3 TO TECHNICAL SPECIFICATIONS  
APPENDIX A TO LICENSE NO. DPR-65

1. In Specification 3.7.1.1 change the Action Statement to read as follows:  
"With one main steam line code safety valve inoperable, restore the inoperable valve to OPERABLE status within 4 hours or be in COLD SHUTDOWN within the next 36 hours.

In Specification 4.7.1.1, delete Table 3.7-1.

In Basis 3/4.7.1.1 delete the remainder of the basis following the sentence which ends with the phrase "at 100% RATED THERMAL POWER.

2. In Specification 4.6.5 items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$ " to "9000 cfm  $\pm$  10%".

In Specification 4.9.15, items b.1, b.2, b.4, and c.1, change "6000 cfm  $\pm$  10%" to "9000 cfm  $\pm$  10%".

3. Change the specifications identified in the following table as indicated to correct proofreading errors.

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