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Dockets Nos. 50-245
and 50-336

Northeast Nuclear Energy Company
 ATTN: Mr. D. C. Switzer
 President
 P. O. Box 270
 Hartford, Connecticut 06101

Gentlemen:

The Commission has issued the enclosed Amendment No. 33 to Provisional Operating License No. DPR-21 and Amendment No. 19 to Facility Operating License No. DPR-65 for the Millstone Nuclear Power Station, Units Nos. 1 and 2. The amendments consist of changes to the Technical Specifications in response to your applications dated August 18, 1976 and October 15, 1976.

These amendments will change the Administrative Controls Section (Section 6.0) of the Millstone Units Nos. 1 and 2 Technical Specifications. The changes consist of (1) changes to the offsite and onsite organization, (2) an increase from 7 to 14 days of the allowable time for the Plant Operating Review Committee (PORC) and the Plant Superintendent to approve documented changes to written procedures, (3) adding an additional responsibility for the PORC to review events requiring 24 hour notification to the Commission, and (4) incorporation of a radiation protection program in the Technical Specifications.

Copies of the Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
 Operating Reactors Branch #3
 Division of Operating Reactors

Enclosures:

1. Amendment No. 33 to DPR-21
2. Amendment No. 19 to DPR-65
3. Safety Evaluation
4. Federal Register Notice

Handwritten notes:
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SURNAME >	CParrish	DJaffe:acr	BGrimes	LSCHANDLER	GLear
DATE >	11/22/76	11/22/76	11/22/76	11/22/76	12/13/76

cc: William H. Cuddy, Esquire
Day, Berry & Howard
Counselors At Law
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The Hartford Electric Light Company
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Hartford, Connecticut 06101

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Roisman, Kessler and Cashdan
1025 15th Street, N. W.
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Washington, D. C. 20005

Robert Bishop
Department of Planning & Energy Policy
20 Grand Street
Hartford, Connecticut 06115

Mr. Albert L. Partridge, First Selectman
Town of Waterford
Hall of Records - 200 Boston Post Road
Waterford, Connecticut 06385

Northeast Nuclear Energy Company
ATTN: Mr. E. J. Ferland
Plant Superintendent
Millstone Plant
P. O. Box 127
Waterford, Connecticut 06385

Waterford Public Library
Rope Ferry Road, Route 156
Waterford, Connecticut 06385



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-245

MILLSTONE NUCLEAR POWER STATION UNIT NO. 1

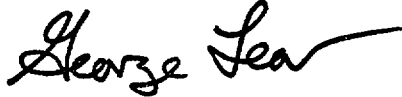
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 33
License No. DPR-21

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, Northeast Nuclear Energy Company (the licensees) dated August 18, 1976 and October 15, 1976, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

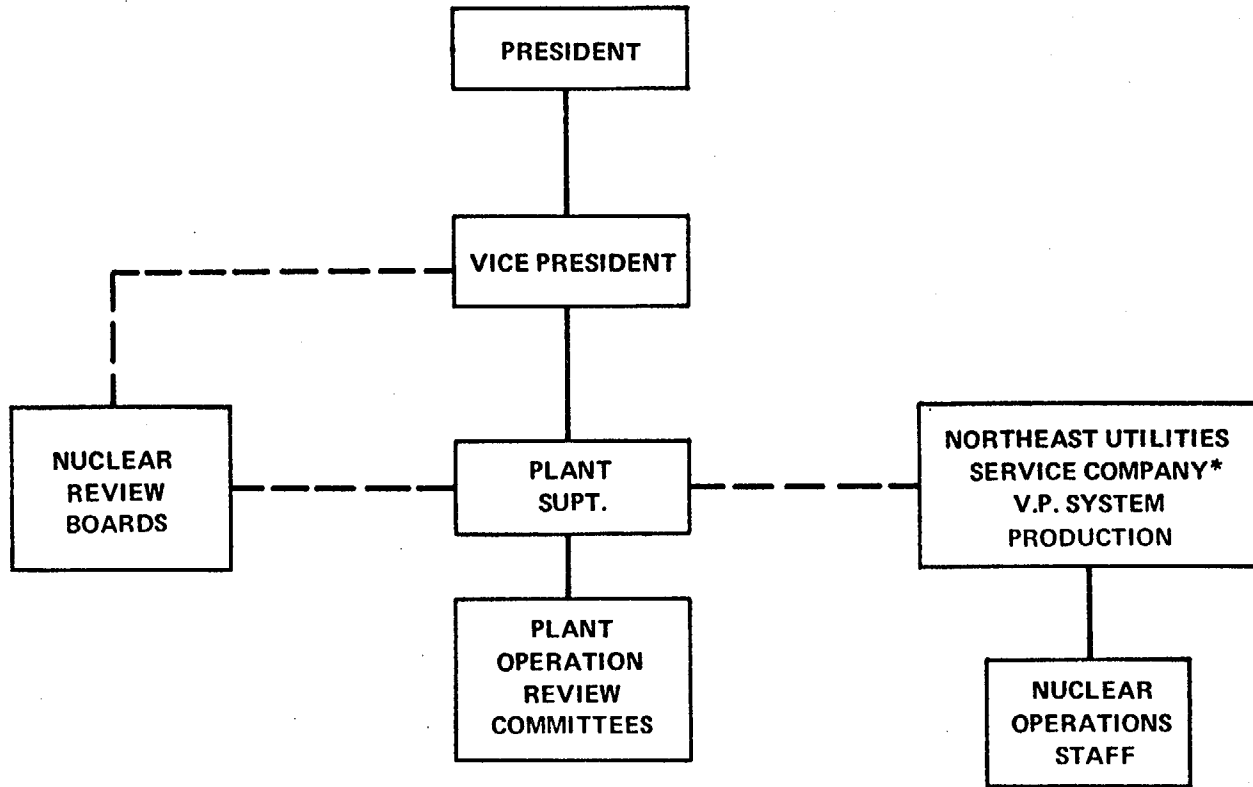
Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 13, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 33
TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-21
DOCKET NO. 50-245

Replace pages 155, 156, 158, 159, 160, 163, 165, 168, 170, 175, 176,
and 177 with the attached revised pages. Add page 178.

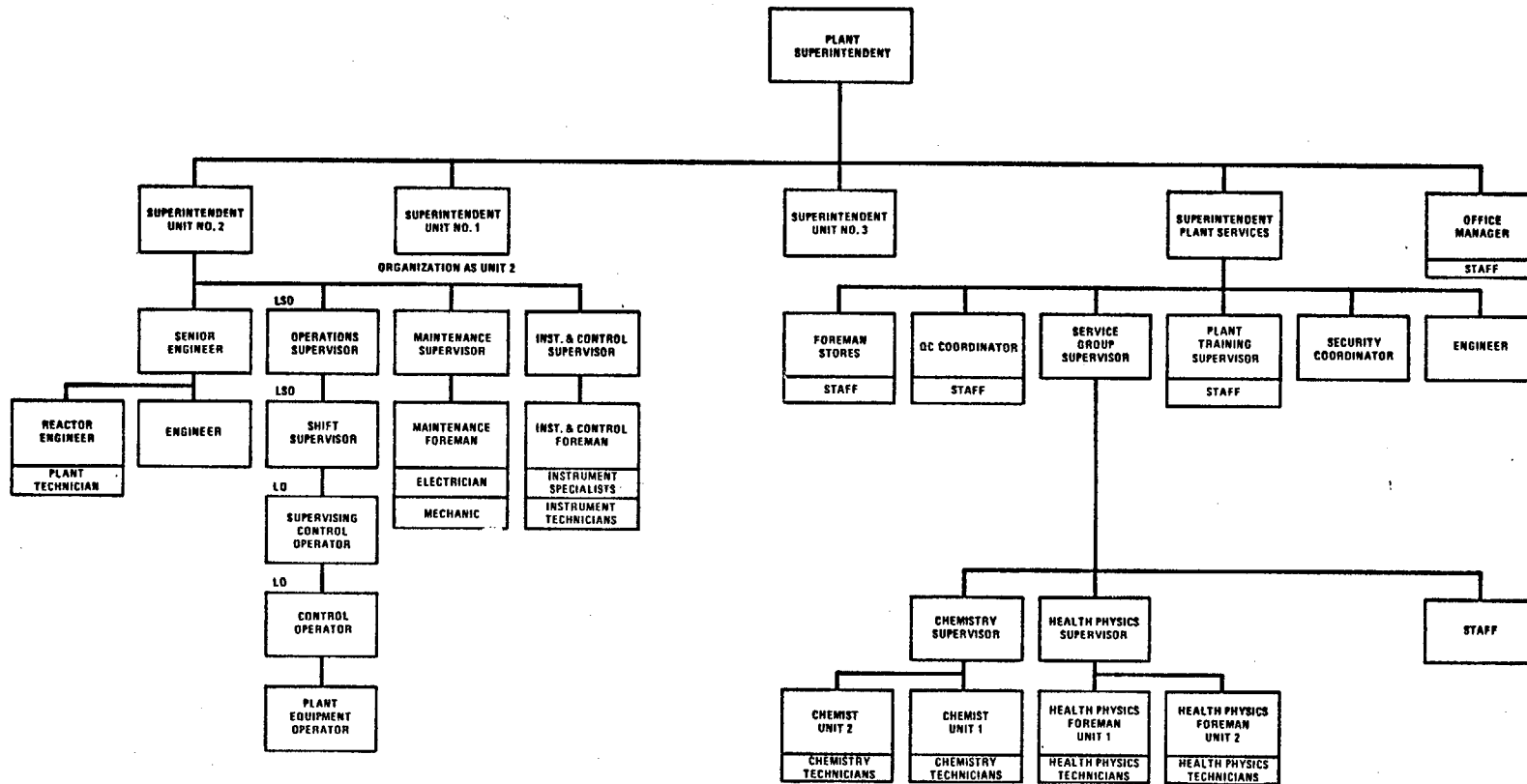
NORTHEAST NUCLEAR ENERGY COMPANY



*PROVIDES OPERATING AND ENGINEERING SUPPORT BY CONTRACTUAL ARRANGEMENT.

FIGURE 6.2-1

Offsite Organization for Facility Management and Technical Support



CODE: LO - LICENSED OPERATOR
 LSO - LICENSED SENIOR OPERATOR

FIGURE 6.2-2
 Facility Organization – Millstone Nuclear Power Station - Unit 1

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Plant Superintendent and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Unit Superintendent
Vice Chairman	
& Member:	Operations Supervisor-Unit 1
Member:	Maintenance Supervisor
Member:	Instrument and Control Supervisor
Member:	Reactor Engineer
Member:	Senior Engineer or Startup Supervisor*
Member:	Health Physicist or Plant Services
Member:	Superintendent or Service Group Supervisor
	or Health Physics Supervisor

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

*When position is staffed.

ADMINISTRATIVE CONTROLS

QUORUM

6.5.1.5 A quorum of the PORC shall consist of the Chairman or Vice Chairman or Plant Superintendent and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Unit Superintendent to affect nuclear safety. |
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Sections 1.0 - 5.0 of these Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Superintendent of Nuclear Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour notification to the Commission. |
- g. Review of facility operations to detect potential safety hazards. |
- h. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Review Board. |
- i. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question. |

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Superintendent written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Provide immediate written notification to the Superintendent of Nuclear Production and the Nuclear Review Board of disagreement between the PORC and the Plant Superintendent; however, the Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The PORC shall maintain written minutes of each meeting and copies shall be provided to the Superintendent of Nuclear Production and Chairman of the Nuclear Review Board.

6.5.2 SITE OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.2.1 The SORC shall function to advise the Plant Superintendent on all matters related to nuclear safety of the entire Millstone Station Site.

COMPOSITION

6.5.2.2 The SORC shall be composed of the:

Chairman:	Plant Superintendent
Member:	Unit 1 Superintendent
Member:	Unit 2 Superintendent
Member:	Unit 3 Superintendent
Member:	Designated Member of Unit 1 PORC
Member:	Designated Member of Unit 2 PORC
Member:	Designated Member of Unit 3 PORC

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.3.2 The NRB shall be composed of the following: (The titles listed are minimum level requirements. Other appropriate higher level position may be substituted.)

Chairman:	Project Engineer
Member:	Nuclear Production Department Engineer cognizant of plant operations
Member:	Engineer, Licensing and Safeguards Section, Generation Engineering Division
Member:	Engineer, Generation Engineering Division
Member:	Unit Superintendent, Northeast Nuclear Energy Company or Assistant Plant Superintendent Connecticut Yankee Atomic Power Company

ALTERNATES

6.5.3.3 Alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, not more than two alternates shall participate in NRB activities at any one time.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.3.5 The NRB shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.3.6 A quorum of NRB shall consist of the Chairman or his designated alternate and three members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. Any other area of facility operation considered appropriate by the NRB or the Vice President System Production.

AUTHORITY

6.5.3.9 The NRB shall report to and advise the Vice President System Production on those areas of responsibility specified in Sections 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President System Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.3.7 above, shall be prepared, approved and forwarded to the Vice President System Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.3.8 above, shall be forwarded to the Vice President System Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.4.9 The SNRB report to and advise the Vice President System Production on those areas of responsibility specified in Sections 6.5.4.7 and 6.5.4.8.

RECORDS

6.5.4.10 Records of SNRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SNRB meeting shall be prepared, approved and forwarded to the Vice President System Production within 14 days following each meeting.
- b. Reports of reviews encompassed by, Section 6.5.4.7 above, shall be prepared, approved and forwarded to the Vice President System Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.4.8 above, shall be forwarded to the Vice President System Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE Report requiring 24 hour notification to the Commission shall be reviewed by the PORC and submitted to the NRB and the Superintendent of Nuclear Production.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.

ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PORC and approved by the Plant Superintendent within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 Information to be reported to the Commission, in addition to the reports required by Title 10, Code of Federal Regulations, shall be in accordance with the Regulatory Position in Revision 4 of Regulatory Guide 1.16, "Reporting of Operating Information - Appendix "A" Technical Specifications."

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- b. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- c. Safety Class 1 Inservice Inspection Program Review, Specification 4.4.10.1.
- d. Core Barrel Movement, Specifications 3.4.11 and 4.4.11.
- e. ECCS Actuation, Specifications 3.5.2 and 3.5.3.

6.10 RECORD RETENTION

TABLE 6.12-1

PROTECTION FACTORS FOR RESPIRATORS

DESCRIPTION	MODES ¹	PROTECTION FACTORS ²	GUIDES TO SELECTION OF EQUIPMENT
		PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE ³	BUREAU OF MINES APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS *or schedule superseding for equipment of type listed
I. AIR-PURIFYING RESPIRATORS			
Facepiece, half-mask ^{4,7} Facepiece, full ⁷	NP NP	5 100	21B 30 CFR § 14.4(b)(4) 21B 30 CFR § 14.4(b)(5); 14F 30 CFR 13
II. ATMOSPHERE-SUPPLYING RESPIRATOR			
1. Airline respirator			
Facepiece, half-mask	CF	100	19B 30 CFR § 12.2(c)(2) Type C(i)
Facepiece, full	CF	1,000	19B 30 CFR § 12.2(c)(2) Type C(i)
Facepiece, full ⁷	D	100	19B 30 CFR § 12.2(c)(2) Type C(ii)
Facepiece, full	PD	1,000	19B 30 CFR § 12.2(c)(2) Type C(iii)
Hood	CF	5	6
Suit	CF	5	6
2. Self-contained breathing apparatus (SCBA)			
Facepiece, full ⁷	D	100	13E 30 CFR § 11.4(b)(2)(i)
Facepiece, full	PD	1,000	13E 30 CFR § 11.4(b)(2)(ii)
Facepiece, full	R	100	13E 30 CFR § 11.4(b)(1)
III. COMBINATION RESPIRATOR			
Any combination of air-purifying and atmosphere-supplying respirator		Protection factor for type and mode of operation as listed above	19B CFR § 12.2(e) or applicable schedules as listed above

1, 2, 3, 4, 5, 6, 7 [These notes are on the following pages.]

MILLSTONE-UNIT 1

175

Amendment No. 14, 33

TABLE 6.12-1 (Continued)

¹ See the following symbols:

CF: continuous flow
D: demand
NP: negative pressure (i.e., negative phase during inhalation)
PD: pressure demand (i.e., always positive pressure)
R: recirculating (closed circuit)

²(a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency [above 99.9% removal efficiency by U.S. Bureau of Mines type dioctyl phthalate (DOP) test] particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

³ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote ⁵, below, concerning supplied-air suits and hoods.

TABLE 6.12-1 (Continued)

- ⁴ Under chin type only. Not recommended for use where it might be possible for the ambient airborne concentration to reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR Part 20.
- ⁵ Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.
- ⁶ No approval schedules current available for this equipment. Equipment must be evaluated by testing or on basis of available test information.
- ⁷ Only for shaven faces.

NOTE 1: Protection factors for respirators, as may be approved by the U.S. Bureau of Mines according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the U.S. Bureau of Mines in accordance with its applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix B, Table I of this part are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits.

ADMINISTRATIVE CONTROLS

6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. A High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1.a above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor or Health Physics Supervisor on duty.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION UNIT NO. 2

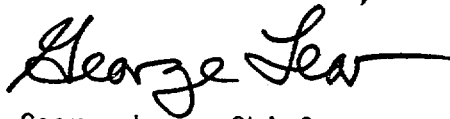
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 19
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by The Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, and Northeast Nuclear Energy Company (the licensees) dated August 18, 1976 and October 15, 1976 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink that reads "George Lear". The signature is written in a cursive style with a long horizontal stroke at the end.

George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 13, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 19

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

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ADMINISTRATIVE CONTROLS

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6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Superintendent shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

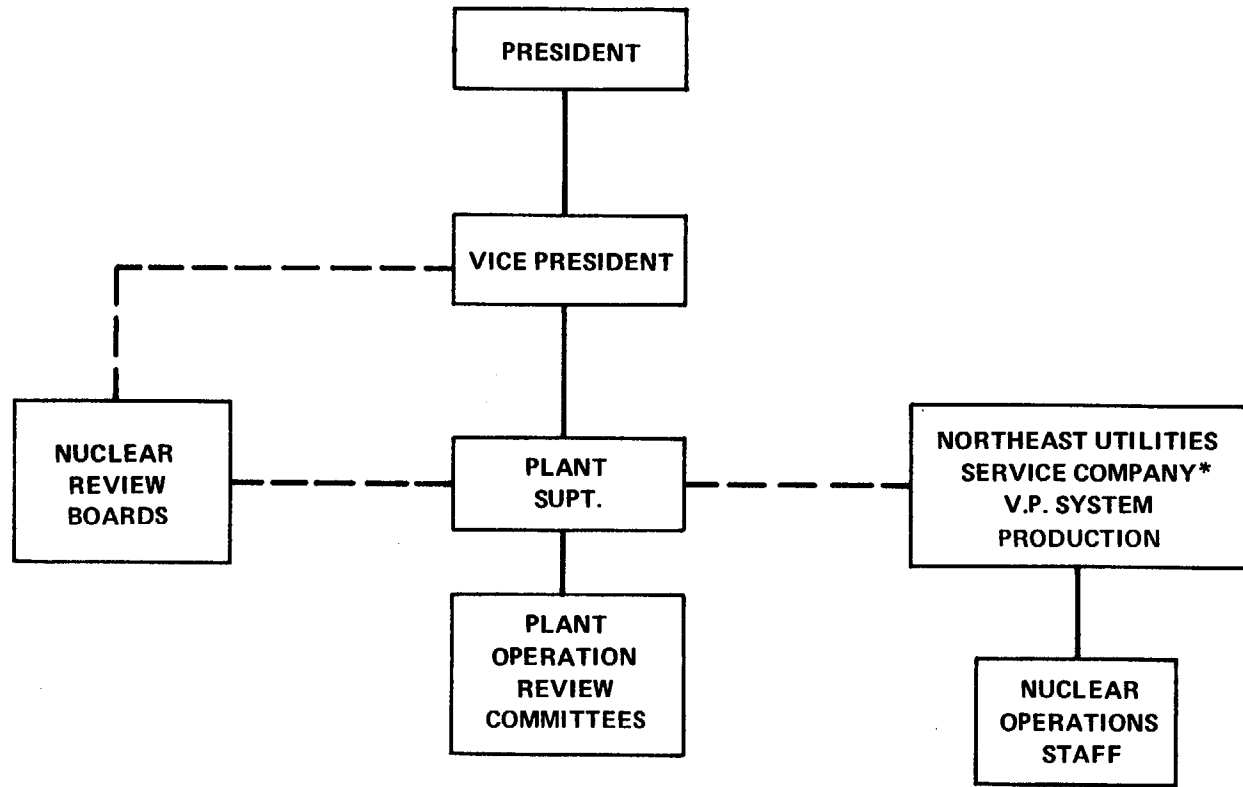
6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

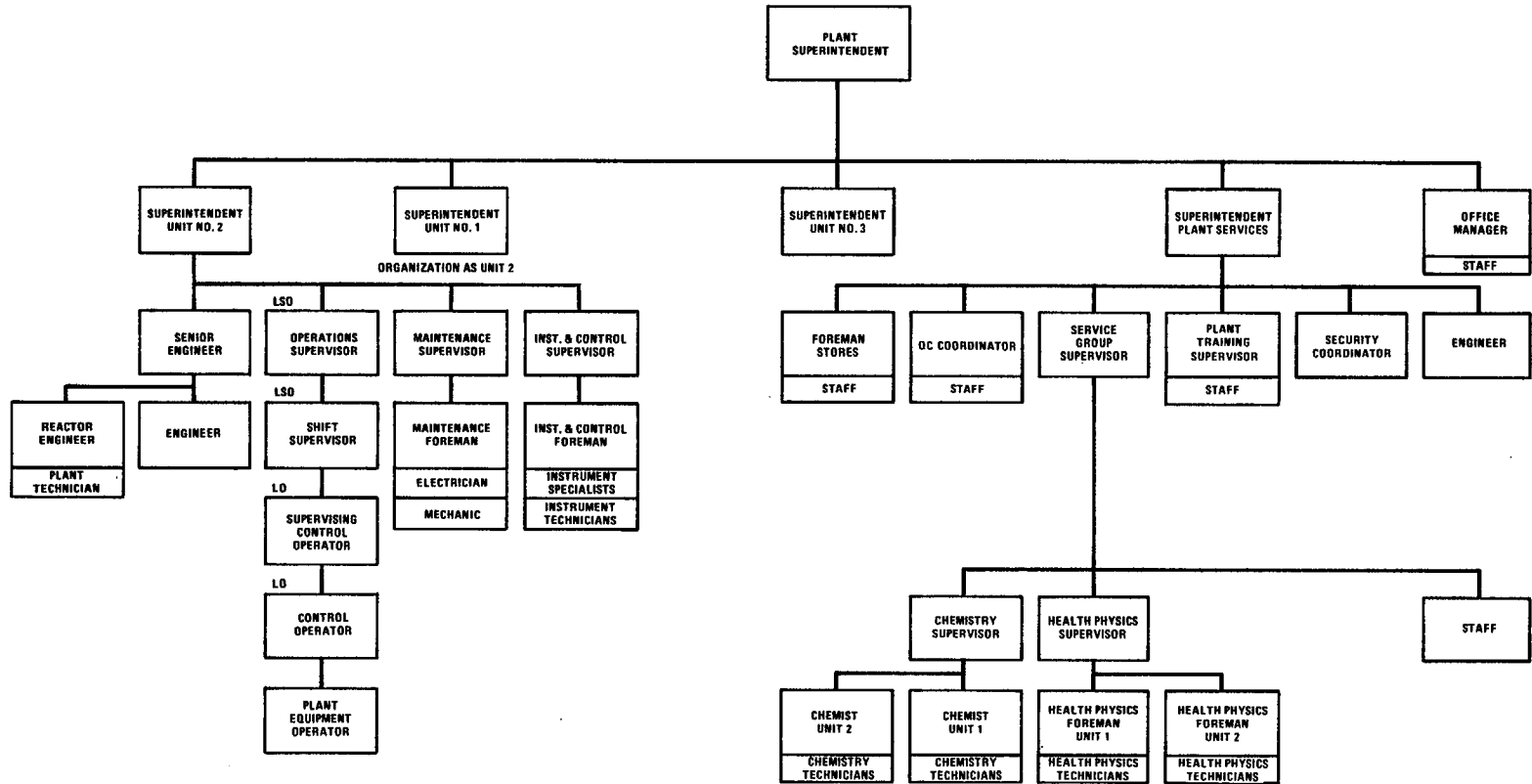
NORTHEAST NUCLEAR ENERGY COMPANY



*PROVIDES OPERATING AND ENGINEERING SUPPORT BY CONTRACTUAL ARRANGEMENT.

FIGURE 6.2-1

Offsite Organization for Facility Management and Technical Support



CODE: LO - LICENSED OPERATOR
 LSO - LICENSED SENIOR OPERATOR

FIGURE 6.2-2
 Facility Organization – Millstone Nuclear Power Station - Unit 2

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Plant Superintendent and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Superintendent on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Unit Superintendent
Vice Chairman & Member:	Operations Supervisor-Unit 2
Member:	Maintenance Supervisor
Member:	Instrument and Control Supervisor
Member:	Reactor Engineer
Member:	Senior Engineer or Startup Supervisor*
Member:	Health Physicist or Plant Services Superintendent or Service Group Supervisor or Health Physics Supervisor

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman.

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ADMINISTRATIVE CONTROLS

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- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Sections 1.0 - 5.0 of these Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Superintendent of Nuclear Production and to the Chairman of the Nuclear Review Board.
- f. Review of events requiring 24 hour notification to the Commission.
- g. Review of facility operations to detect potential safety hazards.
- h. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Review Board.
- i. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend to the Plant Superintendent written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Provide immediate written notification to the Superintendent of Nuclear Production and the Nuclear Review Board of disagreement between the PORC and the Plant Superintendent; however, the Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The PORC shall maintain written minutes of each meeting and copies shall be provided to the Superintendent of Nuclear Production and Chairman of the Nuclear Review Board.

6.5.2 SITE OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.2.1 The SORC shall function to advise the Plant Superintendent on all matters related to nuclear safety of the entire Millstone Station Site.

COMPOSITION

6.5.2.2 The SORC shall be composed of the:

Chairman:	Plant Superintendent
Member:	Unit 1 Superintendent
Member:	Unit 2 Superintendent
Member:	Unit 3 Superintendent
Member:	Designated Member of Unit 1 PORC
Member:	Designated Member of Unit 2 PORC
Member:	Designated Member of Unit 3 PORC

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.2.3 Alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in SORC activities at one time.

MEETING FREQUENCY

6.5.2.4 The SORC shall meet at least once per six months and as convened by the SORC Chairman.

QUORUM

6.5.2.5 A quorum of the SORC shall consist of the Chairman and four members including alternates.

RESPONSIBILITIES

6.5.2.6 The SORC shall be responsible for:

- a. Review of 1) all common site procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Plant Superintendent to affect site nuclear safety.
- b. Review of all proposed changes to Section 6.0 "Administrative Controls" of these Technical Specifications.
- c. Performance of special reviews and investigations and reports as requested by the Chairman of the Site Nuclear Review Board.
- d. Review of the Plant Security Plan and implementing procedures and shall submit changes to the Chairman of the Site Nuclear Review Board.
- e. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Site Nuclear Review Board.
- f. Render determinations in writing or meeting minutes with regard to whether or not each item considered under 6.5.2.6(a) through (e) above constitutes an unreviewed safety question.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.2.7 The SORC shall:

- a. Recommend to the Plant Superintendent written approval or disapproval in meeting minutes of items considered under 6.5.2.6(a) through (e) above.
- b. Provide immediate written notification or meeting minutes to the Superintendent of Nuclear Production and the Site Nuclear Review Board of disagreement between the SORC and the Plant Superintendent; however, the Plant Superintendent shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.2.8 The SORC shall maintain written minutes of each meeting and copies shall be provided to the Superintendent of Nuclear Production and Chairman of the Site Nuclear Review Board.

6.5.3 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.3.1 The NRB shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- b. nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering

ADMINISTRATIVE CONTROLS

COMPOSITION

6.5.3.2 The NRB shall be composed of the following: (The titles listed are minimum level requirements. Other appropriate higher level position may be substituted.)

Chairman: Project Engineer
Member: Nuclear Production Department Engineer cognizant
of plant operations
Member: Engineer, Licensing and Safeguards Section,
Generation Engineering Division
Member: Engineer, Generation Engineering Division
Member: Unit Superintendent, Northeast Nuclear
Energy Company or Assistant Plant Superintendent
Connecticut Yankee Atomic Power Company

ALTERNATES

6.5.3.3 Alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, not more than two alternates shall participate in NRB activities at any one time.

CONSULTANTS

6.5.3.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.

MEETING FREQUENCY

6.5.3.5 The NRB shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.3.6 A quorum of NRB shall consist of the Chairman or his designated alternate and three members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.3.7 The NRB shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Sections 1.0 - 5.0 of these Technical Specifications or licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. REPORTABLE OCCURRENCES requiring 24 hour notification to the Commission.
- h. Indications of a significant unanticipated deficiency, affecting nuclear safety, in some aspect of design or operation of safety related structures, systems or components.
- i. Reports and meetings minutes of the PORC.

AUDITS

6.5.3.8 Audits of facility activities shall be performed under the cognizance of the NRB. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. Any other area of facility operation considered appropriate by the NRB or the Vice President System Production.

AUTHORITY

6.5.3.9 The NRB shall report to and advise the Vice President System Production on those areas of responsibility specified in Sections 6.5.3.7 and 6.5.3.8.

RECORDS

6.5.3.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President System Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.3.7 above, shall be prepared, approved and forwarded to the Vice President System Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.3.8 above, shall be forwarded to the Vice President System Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

ADMINISTRATIVE CONTROLS

AUTHORITY

6.5.4.9 The SNRB report to and advise the Vice President System Production on those areas of responsibility specified in Sections 6.5.4.7 and 6.5.4.8. |

RECORDS

6.5.4.10 Records of SNRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SNRB meeting shall be prepared, approved and forwarded to the Vice President System Production within 14 days following each meeting. |
- b. Reports of reviews encompassed by, Section 6.5.4.7 above, shall be prepared, approved and forwarded to the Vice President System Production within 14 days following completion of the review. |
- c. Audit reports encompassed by Section 6.5.4.8 above, shall be forwarded to the Vice President System Production and to the management positions responsible for the areas audited within 30 days after completion of the audit. |

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE Report requiring 24 hour notification to the Commission shall be reviewed by the PORC and submitted to the NRB and the Superintendent of Nuclear Production. |

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- b. The Safety Limit violation shall be reported to the Commission, the Superintendent of Nuclear Production and to the NRB immediately.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the Superintendent of Nuclear Production within 10 days of the violations.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November 1972.
- b. Refueling operations.
- c. Surveillance activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PORC and approved by the Plant Superintendent prior to implementation and reviewed periodically as set forth in each document.

ADMINISTRATIVE CONTROLS

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed by the PORC and approved by the Plant Superintendent within 14 days of implementation.

6.9 REPORTING REQUIREMENTS

ROUTINE REPORTS AND REPORTABLE OCCURRENCES

6.9.1 Information to be reported to the Commission, in addition to the reports required by Title 10, Code of Federal Regulations, shall be in accordance with the Regulatory Position in Revision 4 of Regulatory Guide 1.16, "Reporting of Operating Information - Appendix "A" Technical Specifications."

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report. These reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification:

- a. Inoperable Seismic Monitoring Instrumentation, Specification 3.3.3.3.
- b. Inoperable Meteorological Monitoring Instrumentation, Specification 3.3.3.4.
- c. Safety Class 1 Inservice Inspection Program Review, Specification 4.4.10.1.
- d. Core Barrel Movement, Specifications 3.4.11 and 4.4.11.
- e. ECCS Actuation, Specifications 3.5.2 and 3.5.3.

6.10 RECORD RETENTION

ADMINISTRATIVE CONTROLS

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.
- i. Records of annual physical inventory of all sealed source material of record.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material released to the environs.
- f. Records of transient or operational cycles for those facility components designed for a limited number of transients or cycles.

ADMINISTRATIVE CONTROLS

2. Written Procedures to assure proper selection, supervision, and training of personnel using such protective equipment.
 3. Written procedures to assure the adequate fitting of respirators; and the testing of respiratory protective equipment for OPERABILITY immediately prior to use.
 4. Written procedures for maintenance to assure full effectiveness of respiratory protective equipment, including issuance, cleaning and decontamination, inspection, repair and storage.
 5. Written operational and administrative procedures for proper use of respiratory protective equipment including provisions for planned limitations on working times as necessitated by operational conditions.
 6. Bioassays and/or whole body counts of individuals (and other surveys, as appropriate) to evaluate individual exposures and to assess protection actually provided.
- e. The licensee shall use equipment approved by the U.S. Bureau of Mines under its appropriate Approval Schedules as set forth in Table 6.12-1. Equipment not approved under U.S. Bureau of Mines Approval Schedules shall be used only if the licensee has evaluated the equipment and can demonstrate by testing, or on the basis of reliable test information, that the material and performance characteristics of the equipment are at least equal to those afforded by U.S. Bureau of Mines approved equipment of the same type, as specified in Table 6.12-1.
- f. Unless otherwise authorized by the Commission, the licensee shall not assign protection factors in excess of those specified in Table 6.12-1 in selecting and using respiratory protective equipment.

REVOCATION

6.12.3 The specifications of Section 6.12 shall be revoked in their entirety upon adoption of the proposed change to 10 CFR 20, Section 20.103, which would make such provisions unnecessary.

TABLE 6.12-1

PROTECTION FACTORS FOR RESPIRATORS

MILLSTONE-UNIT 2

6-22

Amendment No. 19

DESCRIPTION	MODES ¹	PROTECTION FACTORS ²	GUIDES TO SELECTION OF EQUIPMENT
		PARTICULATES AND VAPORS AND GASES EXCEPT TRITIUM OXIDE ³	BUREAU OF MINES APPROVAL SCHEDULES* FOR EQUIPMENT CAPABLE OF PROVIDING AT LEAST EQUIVALENT PROTECTION FACTORS *or schedule superseding for equipment of type listed
I. AIR-PURIFYING RESPIRATORS			
Facepiece, half-mask ^{4,7} Facepiece, full ⁷	NP NP	5 100	21B 30 CFR § 14.4(b)(4) 21B 30 CFR § 14.4(b)(5); 14F 30 CFR 13
II. ATMOSPHERE-SUPPLYING RESPIRATOR			
1. Airline respirator			
Facepiece, half-mask	CF	100	19B 30 CFR § 12.2(c)(2) Type C(i)
Facepiece, full	CF	1,000	19B 30 CFR § 12.2(c)(2) Type C(i)
Facepiece, full ⁷	D	100	19B 30 CFR § 12.2(c)(2) Type C(ii)
Facepiece, full	PD	1,000	19B 30 CFR § 12.2(c)(2) Type C(iii)
Hood	CF	5	6
Suit	CF	5	6
2. Self-contained breathing apparatus (SCBA)			
Facepiece, full ⁷	D	100	13E 30 CFR § 11.4(b)(2)(i)
Facepiece, full	PD	1,000	13E 30 CFR § 11.4(b)(2)(ii)
Facepiece, full	R	100	13E 30 CFR § 11.4(b)(1)
III. COMBINATION RESPIRATOR			
Any combination of air-purifying and atmosphere-supplying respirator		Protection factor for type and mode of operation as listed above	19B CFR § 12.2(e) or applicable schedules as listed above

1, 2, 3, 4, 5, 6, 7 [These notes are on the following pages.]

TABLE 6.12-1 (Continued)

¹ See the following symbols:

CF: continuous flow
D: demand
NP: negative pressure (i.e., negative phase during inhalation)
PD: pressure demand (i.e., always positive pressure)
R: recirculating (closed circuit)

²(a) For purposes of this specification the protection factor is a measure of the degree of protection afforded by a respirator, defined as the ratio of the concentration of airborne radioactive material outside the respiratory protective equipment to that inside the equipment (usually inside the facepiece) under conditions of use. It is applied to the ambient airborne concentration to estimate the concentration inhaled by the wearer according to the following formula:

$$\text{Concentration Inhaled} = \frac{\text{Ambient Airborne Concentration}}{\text{Protection Factor}}$$

(b) The protection factors apply:

- (i) only for trained individuals wearing properly fitted respirators used and maintained under supervision in a well-planned respiratory protective program.
- (ii) for air-purifying respirators only when high efficiency [above 99.9% removal efficiency by U.S. Bureau of Mines type dioctyl phthalate (DOP) test] particulate filters and/or sorbents appropriate to the hazard are used in atmospheres not deficient in oxygen.
- (iii) for atmosphere-supplying respirators only when supplied with adequate respirable air.

³ Excluding radioactive contaminants that present an absorption or submersion hazard. For tritium oxide approximately half of the intake occurs by absorption through the skin so that an overall protection factor of not more than approximately 2 is appropriate when atmosphere-supplying respirators are used to protect against tritium oxide. Air-purifying respirators are not recommended for use against tritium oxide. See also footnote ⁵, below, concerning supplied-air suits and hoods.

TABLE 6.12-1 (Continued)

- ⁴ Under chin type only. Not recommended for use where it might be possible for the ambient airborne concentration to reach instantaneous values greater than 50 times the pertinent values in Appendix B, Table I, Column 1 of 10 CFR Part 20.
- ⁵ Appropriate protection factors must be determined taking account of the design of the suit or hood and its permeability to the contaminant under conditions of use. No protection factor greater than 1,000 shall be used except as authorized by the Commission.
- ⁶ No approval schedules current available for this equipment. Equipment must be evaluated by testing or on basis of available test information.
- ⁷ Only for shaven faces.

NOTE 1: Protection factors for respirators, as may be approved by the U.S. Bureau of Mines according to approval schedules for respirators to protect against airborne radionuclides, may be used to the extent that they do not exceed the protection factors listed in this Table. The protection factors in this Table may not be appropriate to circumstances where chemical or other respiratory hazards exist in addition to radioactive hazards. The selection and use of respirators for such circumstances should take into account approvals of the U.S. Bureau of Mines in accordance with its applicable schedules.

NOTE 2: Radioactive contaminants for which the concentration values in Appendix B, Table I of this part are based on internal dose due to inhalation may, in addition, present external exposure hazards at higher concentrations. Under such circumstances, limitations on occupancy may have to be governed by external dose limits.

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6.13 HIGH RADIATION AREA

6.13.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. A High Radiation Area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. A High Radiation Area in which the intensity of radiation is greater than 1000 mrem/hr shall be subject to the provisions of 6.13.1.a above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor or Health Physics Supervisor on duty.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 33 TO PROVISIONAL OPERATING LICENSE NO. DPR-21 AND

AMENDMENT NO. 19 TO FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY

MILLSTONE NUCLEAR POWER STATION UNITS NOS. 1 AND 2

Introduction

By applications for license amendment dated August 18, 1976 and October 15, 1976, Northeast Nuclear Energy Company (NNECO) requested changes to the Administrative Controls Section (Section 6.0) of the Millstone Units Nos. 1 and 2 Technical Specifications. The proposed changes consist of: (1) changes to the offsite and onsite organization, (2) an increase from 7 to 14 days of the allowable time for the Plant Operating Review Committee (PORC) and the Plant Superintendent to approve documented changes to written procedures, (3) adding an additional responsibility for the PORC to review events requiring 24 hour notification to the Commission, and (4) incorporation of a radiation protection program in the Technical Specifications.

In the course of conducting our review we found it necessary to modify the proposed Technical Specifications in order to meet our requirements. The modifications were made following discussions with and agreement of NNECO.

Discussion and Evaluation

Our discussion and evaluation of NNECO's proposed Technical Specification changes are contained in the following sections:

Changes to the offsite and onsite organizations

NNECO has proposed two changes to the offsite organizational structure contained in Millstone Units Nos. 1 and 2 Technical Specification 6.2.1. The first change involves the title "V. P. System Operations" which is being changed to "V. P. System Production." The second change involves a modification to the reporting level for the Nuclear Review Board (NRB) which reported directly to the President of NNECO. The proposed reporting level for the NRB is the Vice President of NNECO who in turn reports to the President of NNECO.

The proposed modifications to the offsite organization are acceptable since they in no way degrade the managerial, technical support, or review and audit functions of the offsite organization.

NNECO has proposed a number of changes to the onsite organization for Millstone Units Nos. 1 and 2 as stated in Technical Specification 6.2.2. The only significant change is that the Quality Control (QC) Coordinator, who formerly reported directly to the Plant Superintendent, now reports through the Superintendent of Plant Services to the Plant Superintendent.

Although the QC Coordinator reports to a lower level of management, sufficient separation still exists between the QC and maintenance functions to allow the QC Coordinator to be independent of those people he must audit; therefore, the proposed changes to the onsite organization are acceptable.

A number of changes are made in the Technical Specifications to make them consistent with the new organizational structures. These changes are acceptable.

Increase from 7 to 14 days of the allowable time for the Plant Operating Review Committee (PORC) and the Plant Superintendent to approve documented changes to written procedures

The increase in allowable approval time from 7 days, as required by Technical Specification 6.8.3.c., to 14 days is consistent with our current guidance. The 14 day interval has been incorporated in our Standard Technical Specification format and is acceptable for application in the Millstone Units Nos. 1 and 2 Technical Specifications.

Additional Responsibility for the PORC

Technical Specification 6.5.1.6 sets forth the responsibilities for the PORC. NNECO has proposed that in addition to those responsibilities already listed, that the PORC also perform "Review of events requiring 24 hour notification to the Commission". This responsibility had been performed by the PORC but was not listed in the Technical Specifications. This requested change provides an appropriate level of review for significant plant occurrences and is acceptable.

Radiation Protection Program

NNECO has proposed an additional section to the Millstone Units Nos. 1 and 2 Technical Specifications. The new section, designated as Section 6.13, describes the measures that NNECO will take to limit the exposure of personnel to radiation in areas where the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr. These measures include: (1) control of high radiation areas (i.e., limiting access) through the use of Radiation Work Permits, (2) equipping individuals within these areas with continuous

radiation monitoring devices, and (3) providing locked doors and key control in areas of radiation >1000 mr/hr.

Title 10 CFR Part 20, Section 20.203(c)(2) addresses specific methods to be used to protect personnel in high radiation areas. Section 20.203(c)(5) allows the Commission to approve an alternative program provided that the licensee demonstrates that "...the alternate methods of control will prevent unauthorized entry into a high radiation area, and that the requirements of subparagraph (3) of this paragraph is met;" subparagraph (3) provides that the radiation control program be established so as not to prevent a person from leaving an area of high radiation.

We have determined that NNECO's proposed radiation protection program meets the requirements of 10 CFR Part 20, Section 20.203(c)(5) in that NNECO has demonstrated in the August 18, 1976 application that the proposed program will prevent unauthorized entry into areas of high radiation while allowing proper exit from such areas. Thus, the proposed program is an acceptable alternative to the requirements of 20.203(c)(2).

Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4), that an environmental statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the changes do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the changes do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 13, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-245 AND 50-336

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO OPERATING
LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 33 to Provisional Operating License No. DPR-21 and Amendment No. 19 to Facility Operating License No. DPR-65 to Northeast Nuclear Energy Company, The Connecticut Light and Power Company, the Hartford Electric Light Company, and Western Massachusetts Electric Company, which revised Technical Specifications for operation of the Millstone Nuclear Power Station, Units Nos. 1 and 2, located in the Town of Waterford, Connecticut. The amendments are effective as of their date of issuance.

These amendments will change the Administrative Controls Section (Section 6.0) of the Millstone Units Nos. 1 and 2 Technical Specifications. The changes consist of: (1) changes to the offsite and onsite organization, (2) an increase from 7 to 14 days of the allowable time for the Plant Operating Review Committee (PORC) and the Plant Superintendent to approve documented changes to written procedures, (3) adding an additional responsibility for the PORC to review events requiring 24 hour notification to the Commission, and (4) incorporation of a radiation protection program in the Technical Specifications.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations

in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

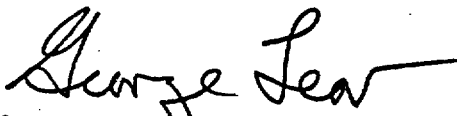
The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated August 18, 1976 and October 15, 1976, (2) Amendments Nos. 33 and 19 to Licenses Nos. DPR-21 and DPR-65, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Waterford Public Library, Rope Ferry Road, Route 156, Waterford, Connecticut.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 13 day of December 1976

FOR THE NUCLEAR REGULATORY COMMISSION


George Lear, Chief
Operating Reactors Branch #3
Division of Operating Reactors