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Docket No. 50-336

Mr. W. G. Counsil, Senior Vice President
Nuclear Engineering and Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06101

Dear Mr. Counsil:

Confirming our telephoned authorization to you on March 15, 1983, the Commission has issued the enclosed Amendment No. 83 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit 2. This amendment consists of changes to the Technical Specifications in response to your application dated March 14, 1983. These changes have been discussed with and accepted by your staff.

This amendment, as you requested, defers the steam generator inservice inspection requirements of Technical Specification 4.4.5.1.3.c.1 to the May 1983 refueling outage. This deferral applies only to the unit outage which commenced on March 1, 1983. Without this amendment the specification would require inspection of a three percent sample of tubes from each steam generator prior to resumption of unit operation.

We have concluded, based on the eddy current and boroscopic examinations performed, rereview of the tapes of last year's eddy current examinations, and tube plugging, that operation of the Millstone 2 steam generators for approximately 9 weeks prior to performing a scheduled inservice inspection does not represent an undue risk to public health and safety. However, due to the uncertainties in the leak rate measurements, we have included in the amendment a temporary change in Specification 3.4.6.2.c lowering the primary-to-secondary leak rate limit from 0.50 gpm to 0.35 gpm per steam generator for the proposed period of operation. While your staff does not agree with the need for this temporary change in leak rate limit, they have accepted it.

A copy of the Safety Evaluation and of the Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert A. Clark

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

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P PDR

No legal objection
OELD
M. Rothschild
3/16/83

Enclosures:
1. Amendment No. 83 to DPR-65
2. Safety Evaluation
3. Federal Register Notice

cc: See next page

OFFICE				ORB#3:DL	ORB#3:DL	ORB#3:DL	AD:ORB:DL
SURNAME				PMKreutzer	PLeech/pn	RAClark	GClainas
DATE				3/16/83	3/16/83	3/16/83	3/16/83

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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Docket No. 50-336

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: NORTHEAST NUCLEAR ENERGY COMPANY, ET AL., Millstone Nuclear Power Station, Unit No. 2.

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

☐ Notice of Receipt of Application for Construction Permit(s) and Operating License(s).

☐ Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.

☐ Notice of Availability of Applicant's Environmental Report.

☐ Notice of Proposed Issuance of Amendment to Facility Operating License.

☐ Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.

☐ Notice of Availability of NRC Draft/Final Environmental Statement.

☐ Notice of Limited Work Authorization.

☐ Notice of Availability of Safety Evaluation Report.

☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendment No. 83
Referenced documents have been provided PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#3:DL					
SURNAME →	PMKreutzer/pn					
DATE →	3/ /83					

NRC FORM 102 7—79

Northeast Nuclear Energy Company

cc:

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ATTN: Under Secretary Energy
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Hartford, Connecticut 06115

U. S. Environmental Protection Agency
Region I Office
ATTN: Regional Radiation
Representative
John F. Kennedy Federal Building
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Generation Facilities Licensing
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY

THE CONNECTICUT LIGHT AND POWER COMPANY

THE HARTFORD ELECTRIC LIGHT COMPANY

THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated March 14, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective on March 15, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 16, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 83

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 4-7
3/4 4-9

4.4.5.1.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.

b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-6 at 40 month intervals fall into Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.5.1.3.a; the interval may then be extended to a maximum of once per 40 months.

c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-6 during the shutdown subsequent to any of the following conditions:

1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2.*

2. A seismic occurrence greater than the Operating Basis Earthquake.

3. A loss-of-coolant accident requiring actuation of the engineered safeguards.

4. A main steam line or feedwater line break.

*For the outage commencing on March 1, 1983 only, the inservice inspection requirements of Category C-1 of Table 4.4-6 are deferred to the 1983 refueling outage.

4.4.5.1.4 Acceptance Criteria

a. As used in this Specification

1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal tube wall thickness, if detectable, may be considered as imperfections.

2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.

3. Degraded Tube means a tube containing imperfections ≥20% of the nominal wall thickness caused by degradation.

4. % Degradation means the percentage of the tube wall thickness affected or removed by degradation.

5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective.

6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40% of the nominal tube wall thickness.

7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.5.1.3.c, above.

8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U - Bend to the top support of the cold leg.

b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.4-6.

REACTOR COOLANT SYSTEM

REACTOR COOLANT SYSTEM LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.6.2 Reactor Coolant System leakage shall be limited to:

a. No PRESSURE BOUNDARY LEAKAGE,

b. 1 GPM UNIDENTIFIED LEAKAGE,

c. 1 GPM total primary-to-secondary leakage through both steam generators and 0.5 GPM through any one steam generator*, and

d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

a. With any PRESSURE BOUNDARY LEAKAGE, be in COLD SHUTDOWN within 36 hours.

b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in COLD SHUTDOWN within the next 36 hours.

*For the period commencing on March 1, 1983 until the end of operation under fuel Cycle 5, the leakage through any one steam generator shall be limited to 0.35 GPM.

SURVEILLANCE REQUIREMENTS

4.4.6.2 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

a. Monitoring the containment atmosphere particulate radioactivity at least once per 12 hours.

b. Monitoring the containment sump inventory at least once per 12 hours,

c. Performance of a Reactor Coolant System water inventory balance at least once per 72 hours during steady state operation except when operating in the shutdown cooling mode.

REACTOR COOLANT SYSTEM

CHEMISTRY

LIMITING CONDITION FOR OPERATION

3.4.7 The Reactor Coolant System chemistry shall be maintained within the limits specified in Table 3.4-1.

APPLICABILITY: ALL MODES.

ACTION:

MODES 1, 2, 3 and 4

a. With any one or more chemistry parameter in excess of its Steady State Limit but within its Transient Limit, restore the parameter to within its Steady State Limit within 24 hours or be in COLD SHUTDOWN within the next 36 hours.

b. With any one or more chemistry parameter in excess of its Transient Limit, be in COLD SHUTDOWN within 36 hours.

MODES 5 and 6

With the concentration of either chloride or fluoride in the Reactor Coolant System in excess of its Steady State Limit for more than 24 hours or in excess of its Transient Limit, reduce the pressurizer pressure to $\leq$ 500 psia, if applicable, and perform an analysis to determine the effects of the out-of-limit condition on the structural integrity of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operations prior to increasing the pressurizer pressure above 500 psia or prior to proceeding to MODE 4.

SURVEILLANCE REQUIREMENTS

4.4.7 The Reactor Coolant System chemistry shall be determined to be within the limits by analysis of those parameters at the frequencies specified in Table 4.4-1.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 83 TO

FACILITY OPERATING LICENSE NO. DPR-65

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

Introduction

By letter dated March 14, 1983, Northeast Nuclear Energy Company (NNECO or the licensee) proposed a change to the Technical Specifications (TS) to allow for a one time postponement of the steam generator inspection requirements of Technical Specification 4.4.5.1.3.C.1 regarding unscheduled inspections due to tube leaks until the 1983 refueling outage scheduled for late May 1983.

Dicussion

Since April 1982 Millstone Unit 2 steam generators had been operating with slowly increasing primary to secondary leakage which was at 0.286 gpm at the end of February 1983. On March 1, 1983 NNECO initiated a shutdown of Millstone Unit 2 in accordance with Technical Specification Action Statement 3.4.6.2.b due to unidentified reactor coolant system (RCS) leakage exceeding the Technical Specification limit of 1.0 gpm. NNECO proceeded with investigations within the containment to identify and quantify leakage from two valves which, from past inspections, were known to be leaking. The results of these investigations, however, were not completely successful as the unidentified portion of the total RCS leakage could not be reduced to within the limits of Technical Specification 3.4.6.2.b. It was decided that an inspection of S. G. No. 1 would be conducted to better quantify the primary to secondary leakage from the steam generators.

Upon inspection of S. G. No. 1 with the use of a secondary-to-primary side hydrostatic test, leakage from an unplugged steam generator tube was identified. This leakage was quantified to be 0.21 gpm at a differential pressure of 200 psi. With this information, NNECO performed evaluations to determine the amount of primary-to-secondary leakage which would have existed at normal plant operating conditions. These evaluations were complicated due to the differences which exist between normal plant operating temperatures, pressures and stresses and those which existed at the time of the hydrostatic leak test. The evaluation is further complicated by the lack of a definitive characterization of the defect. Using conservative assumptions regarding the type of defect (i.e., to maximize primary-to-secondary leakage), it has been concluded, that at the time the plant was shut down, the primary-to-secondary leakage rate in S. G. No. 1 exceeded the limits of Technical Specification 3.4.6.2.c (0.5 gpm, in any one steam generator).

The leaking tube was located at Column 120, Row 94 adjacent to a stay rod on the hot leg side of S. G. No. 1. The eddy current inspection results for this tube obtained during last year's inspection have been rereviewed and it has been concluded that a defect in excess of the plugging limit existed but was overlooked. The rereview has quantified the depth of the defect to be approximately 83% through wall. The defect has been located at the top of the tube sheet, within the limits of eddy current accuracy, and coincides with various interferences from the combination of deposits, denting, and tube sheet entry.

NNECO has performed eddy-current examinations on the tubes adjacent to the defective tube as well as tubes located around another stay rod on the hot leg side of S. G. No. 1. No other defective tubes have been identified in these inspections. One potentially degraded tube has been identified (Column 121, Row 93) with an indication of 34%, although the indication could also be characterized as copper related interference.

NNECO has reviewed last year's eddy-current examination results for those tubes located around all stay rods in the hot leg of S. G. No. 1 and has not identified any degraded tubes. NNECO has also reviewed the eddy-current examination results for randomly selected tubes in the hot leg of S. G. No. 1, some of which exhibited a tube sheet dent signal, to ensure that additional defective tubes were not overlooked last year. The results of this review have not revealed any additional degraded tubes. Based on the eddy-current and visual inspections conducted on the defective tube, in combination with the destructive and nondestructive examination results obtained during the 1981/1982 refueling outage, the failure mechanism responsible for the leaking tube (120, 94) is most probably due to pitting corrosion. A boroscopic examination of the inside diameter of the leaking tube revealed that the defect appeared to be two pits joined by a crack.

NNECO has staked and plugged the defective tube and plugged the tube with the 34% indication. An inspection of S. G. No. 2 was not performed based on the fact that the reviews of last year's information for S. G. No. 1 has not revealed any additional oversights regarding degraded tubes. The primary-to-secondary leakage rate in S. G. No. 2 corrected to reflect the data obtained from S. G. No. 1, as discussed above, remains well within the Technical Specification limits. These factors provide substantial assurance that the integrity of S. G. No. 2 will not become unacceptably degraded during the remaining 9 weeks of operation.

NNECO currently intends to undertake several extensive projects related to the steam generators at Millstone Unit No. 2 during the upcoming refueling outage. These projects include decontamination of the primary side of the steam generators, extensive eddy-current and profilometer examinations in both steam generators and an extensive tube sleeving program. Based on the information obtained to date and presented herein, an eddy-current examination of the steam generators at this time is not considered to be necessary, particularly when a more extensive program of inspections is planned to commence in approximately 9 weeks with the Cycle 6 refueling outage. Personnel exposures to perform such an examination will be reduced during the outage by the steam generator channel head decontamination program. Current radiation fields in S. G. No. 1 are on the order of 17 rem/hour.

Concerning the capability to measure primary-to-secondary leakage rates in the future, NNECO is investigating the use of the condenser steam jet air ejector radiation monitors together with revised empirical relationships for the secondary side gross activity measurement technique. The discrepancy between the primary-to-secondary leakage measurements obtained during operation and that calculated from actual leak rate measurements obtained during the current outage can be attributed to the difficulty of obtaining a representative secondary side sample of steam generator bulk chemistry to be used for gross activity determinations. It is NNECO's intention to administratively include an appropriate scaling factor in the primary-to-secondary leak rate calculation to reflect the data obtained during the current outage and subsequent startup. These actions will result in more accurate absolute primary-to-secondary leakage rate determinations.

Evaluation

Northeast Nuclear Energy Company's program of hydrostatic leakage testing, eddy current and boroscopic examinations, rereview of eddy current inspection tapes from the last inspection, and tube plugging has provided sufficient verification of the source of the observed leakage and the corrective measures undertaken provide assurance

against excessive leakage during the proposed 9 additional weeks of operation prior to a full inservice inspection of the steam generators. However, for future assurance of steam generator tube integrity, the strength of tubes with linkage of individual pits via a crack network should be examined either analytically or experimentally.

Conclusion

We have concluded that operation of the Millstone 2 steam generators for approximately 9 weeks prior to performing a scheduled inservice inspection does not represent an undue risk to public health and safety. Due to uncertainties inherent in primary-to-secondary leak rate measurements, we recommend that the technical specification leak rate limit be decreased from 0.50 gpm to 0.35 gpm per steam generator. We also find it necessary to be kept informed of the minimum reasonable increase in primary-to-secondary leakage. To accomplish this, the licensee shall continue per T.S. Table 4.7-2 to report within 24 hours any increase of 0.05 gpm or more over the base-line leakage followed by a written report within 14 working days. The written report will contain the licensee's evaluation of the increased leakage and justification for continued operation. The staff may, after review of these increased leakage reports, take action as necessary, including ordering a reactor shutdown to protect the primary system integrity. In addition, the licensee should submit a complete inspection plan for the May 1983 refueling outage 30 days in advance of the outage.

Enviromental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 16, 1983

Principal Contributors:
Louis Frank
Herbert Conrad

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-336

NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 83 to Facility Operating License No. DPR-65, issued to Northeast Nuclear Energy Company, the Connecticut Light and Power Company, the Hartford Electric Light Company, and the Western Massachusetts Electric Company (the licensee), which revised Technical Specifications (TS) for operation of the Millstone Nuclear Power Station, Unit No. 2 (the facility) located in the Town of Waterford, Connecticut. The amendment was effective on March 15, 1983.

The amendment defers the steam generator inservice inspection requirements of TS 4.4.5.1.3.c.1 to the May 1983 refueling outage and applies only to the outage which commenced on March 1, 1983.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 14, 1983 (2) Amendment No. 83 to License No. DPR-65, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Waterford Public Library, Rope Ferry Road, Waterford, Connecticut. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 16th day of March, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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Docket No. 50-336

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: NORTHEAST NUCLEAR ENERGY COMPANY, ET AL., Millstone Nuclear Power Station, Unit No. 2.

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

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☐ Notice of Issuance of Construction Permit(s).

☐ Notice of Issuance of Facility Operating License(s) or Amendment(s).

☒ Other: Amendment No. 83
Referenced documents have been provided PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#3:DL					
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DATE →	3/17/83					

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