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MAY 5 1983

Docket No. 50-336

Mr. W. G. Council, Senior Vice President
 Nuclear Engineering & Operations
 Northeast Nuclear Energy Company
 P. O. Box 270
 Hartford, Connecticut 06101

Dear Mr. Council:

SUBJECT: TEMPORARY EQUIPMENT HATCH DOOR FOR MILLSTONE
 NUCLEAR POWER STATION, UNIT 2

The Commission has issued the enclosed Amendment No. 85 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit 2, in response to your application dated April 25, 1983.

This amendment adds a footnote to Technical Specification 3.9.4.a allowing the use of a temporary equipment hatch door to maintain containment integrity during fuel movement or core alterations while the unit is shutdown for Cycle 6 refueling. We understand that the temporary door will have sealable penetrations, not available in the permanent door, for hoses and electrical cables from outside equipment which will be needed to support the chemical decontamination of steam generator channel heads. To minimize the Unit 2 outage time, both defueling of the reactor and decontamination of the steam generator channel heads are planned to occur simultaneously shortly after the unit is shutdown on May 28. Also, the asterisk (*) in Technical Specification 3.9.4.c.1 and the associated footnote have been removed since they were applicable only to the 1980 refueling outage.

As stated in the enclosed Safety Evaluation, we conclude that your proposal will provide adequate containment integrity during the activities described. Therefore, we find the proposed modifications and technical specification changes to be acceptable.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by

Paul H. Leech, Project Manager
 Operating Reactors Branch #3
 Division of Licensing

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Enclosures and cc:

See Page 2

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OELD	M. J. Henschel	5/3/83

*Consent license
amendment 1st. Reg. sub. a*

DL 5/5/83

- Enclosures:
1. Amendment No. 85 to DPR-65
 2. Safety Evaluation
 3. Notice of Issuance
- cc w/enclosures:



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

DISTRIBUTION:
Docket File
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PMKreutzer

Docket No. 50-336

Docketing and Service Section
Office of the Secretary of the Commission

SUBJECT: NORTHEAST NUCLEAR ENERGY COMPANY, ET AL., Millstone Nuclear
Power Station, Unit No. 2

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).
- Other: Amendment No. 85
Referenced documents have been provided PDR.

Division of Licensing
Office of Nuclear Reactor Regulation

Enclosure:
As Stated

OFFICE →	ORB#3:DL					
SURNAME →	PMKreutzer/pn					
DATE →	5/6/83					

Northeast Nuclear Energy Company

cc:

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Mr. Lawrence Bettencourt, First Selectman
Town of Waterford
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Waterford, Connecticut 06385

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Region I Office
ATTN: Regional Radiation
Representative
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY
THE CONNECTICUT LIGHT AND POWER COMPANY
THE HARTFORD ELECTRIC LIGHT COMPANY
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY

DOCKET NO. 50-336

MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northeast Nuclear Energy Company, et al. (the licensee) dated April 25, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective on the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Charles M. Powell
Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 5, 1983

ATTACHMENT TO LICENSE AMENDMENT NO.85

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Remove and replace the following page of the Appendix A Technical Specifications with the enclosed page. The revised page is identified by amendment number and contains vertical lines indicating the area of change. The corresponding overleaf page is provided to maintain document completeness.

Remove

3/4 9-4

Insert

3/4 9-4

REFUELING OPERATIONS

DECAY TIME

LIMITING CONDITION FOR OPERATION

3.9.3 The reactor shall be subcritical for a minimum of 72 hours prior to movement of irradiated fuel in the reactor pressure vessel.

APPLICABILITY: MODE 6.

ACTION:

With the reactor subcritical for less than 72 hours, suspend all operations involving movement of irradiated fuel in the reactor pressure vessel.

SURVEILLANCE REQUIREMENTS

4.9.3 The reactor shall be determined to have been subcritical for at least 72 hours by verification of the date and time of subcriticality prior to movement of irradiated fuel in the reactor pressure vessel.

REFUELING OPERATIONS

CONTAINMENT PENETRATIONS

LIMITING CONDITION FOR OPERATION

3.9.4 The containment penetrations shall be in the following status:

- a. The equipment door closed and held in place by a minimum of four bolts,*
- b. A minimum of one door in each airlock is closed, and
- c. Each penetration providing direct access from the containment atmosphere to the outside atmosphere shall be either:
 1. Closed by an isolation valve, blind flange, manual valve, or special device, or
 2. Be capable of being closed by an OPERABLE automatic containment purge valve.

APPLICABILITY: During CORE ALTERATIONS or movement of irradiated fuel within the containment.

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or movement of irradiated fuel in the containment.

SURVEILLANCE REQUIREMENTS

4.9.4 Each of the above required containment penetrations shall be determined to be either in its isolated condition or capable of being closed by an OPERABLE automatic containment purge valve within 72 hours prior to the start of and at least once per 31 days during CORE ALTERATIONS or movement of irradiated fuel in the containment by:

- a. Verifying the penetrations are in their isolated condition, or
- b. Testing the containment purge valves per the applicable portions of Specification 4.6.3.1.2.

*For the Cycle 6 refueling only, during fuel movement or core alterations, a temporary equipment door may be substituted for the equipment hatch door to facilitate steam generator maintenance activities.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. DPR-65
NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.
MILLSTONE NUCLEAR POWER STATION, UNIT 2
DOCKET NO. 50-336

Introduction

Millstone Nuclear Power Station, Unit 2, is presently preparing for a June-September 1983 refueling outage. During this outage, the licensee will perform both the 10 year inservice inspection and steam generator sleeving. Prior to the steam generator sleeving, steam generator channel head decontamination needs to take place in order to reduce the exposure to maintenance personnel.

The licensee has scheduled the steam generator channel head decontamination concurrently with refueling activities. The technical specifications require that containment integrity be established when fuel is being moved. Therefore, containment integrity needs to be maintained during the steam generator channel head decontamination.

Steam generator channel head decontamination procedures require a number of hoses and electrical cables to enter containment from the auxiliary building. In order to find a path for these hoses and cables, the licensee has proposed building a temporary containment building equipment hatch door. The new door would provide a path for the hoses and cables to support the decontamination process and it would maintain containment integrity against a postulated fuel handling accident.

Evaluation

The Millstone Unit No. 2 containment building includes an equipment hatch, nineteen (19) feet in diameter, to permit transfer of large components into and out of the containment. It is fitted with a double gasketed flange around the dished door to minimize leakage in the unlikely event of a loss-of-coolant-accident (LOCA).

Technical Specification 3.9.4 specifies the required status of certain containment penetrations during core alterations or movement of irradiated fuel within the containment. These requirements ensure that a release of radioactive material within the containment will be restricted from leakage to the environment. The radioactive material released from a postulated fuel element rupture would be retained within the building.

The temporary equipment hatch door proposed to be utilized during the Cycle 6 refueling outage will consist of a circular 1/4 inch thick steel plate with stiffeners. The door will be mounted to the 3/4 inch thick, 8 inch wide, embedded plate which circles the exterior end of the equipment hatch containment penetration. It will be secured in place by thirteen (13) studs which will be welded to the embedded plate.

An O-ring gasket is located between the temporary door and the embedded plate to provide an "air tight" seal between the containment and the enclosure building.

Ten (10) six inch diameter penetrations through the door are provided for various hoses and electrical cables needed to support the steam generator channel head chemical decontamination project. The temporary door is designed to maintain primary containment penetration integrity required by Technical Specification 3.9.4 during core alterations or movement of irradiated fuel within the containment. The penetrations will be isolated by blind flanges when not in use. Penetrations in which cables or hoses pass through will be sealed with silicone RTV fire-resistant foam to provide the required sealing.

The features incorporated into the design of this temporary equipment hatch door ensure that containment integrity will be maintained while core alterations or fuel movements are conducted within the containment considering the negligible pressure gradient which will exist across the equipment hatch penetration.

The ten (10) penetrations will consist of one foot long segments of six inch diameter pipe penetrating the door and welded along their perimeters. The silicon foam will surround the hoses and cables and completely fill the one foot pipe segment. The silicon foam is quick drying and will harden to a rubber-like consistency. Testing of this foam shows that it will stay in place and maintain a vapor barrier. The hoses and cables will be supported by scaffolding on both sides of the temporary equipment hatch door. Therefore, the dead weight loads and vibrations of the hoses should not interfere with the integrity of the foam seal.

If one of the foam seal barriers fails during refueling operations, the technical specifications require that all refueling activities be immediately suspended. Should a fuel handling accident occur and all steam generator channel head decontamination activities are suspended, the licensee has stated that the hoses will not be a potential leak path. The licensee also stated that the decontamination system can be isolated within itself.

During refueling operations, the containment need be protected only for a fuel handling accident. Since we do not postulate high or medium energy pipe ruptures inside containment during refueling, the foam barriers need be only vapor barriers and do not need to withstand large differential pressures.

Based on our review and discussions with the licensee, we conclude that the licensee's proposal will provide adequate containment integrity during the proposed refueling outage. Therefore, we find the proposed modifications and temporary technical specification change acceptable.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: May 5, 1983

Principal Contributors:
Douglas Pickett, ORAB:DL
Dennis Kubicki

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-336NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 85 to Facility Operating License No. DPR-65, issued to The Connecticut Light and Power Company, The Hartford Electric Light Company, Western Massachusetts Electric Company, and Northeast Nuclear Energy Company (the licensees), which temporarily revised the Technical Specifications (TS) for operation of the Millstone Nuclear Power Station Unit No. 2 (the facility) located in the Town of Waterford, Connecticut. The amendment was effective upon the date of its issuance.

The amendment added a footnote to 3.9.4.a to allow the use of a temporary equipment hatch door to maintain containment integrity during fuel movement or core alterations while the Unit is shutdown for Cycle 6 refueling.

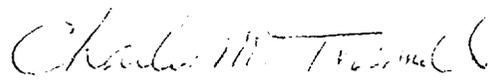
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

For further details with respect to this action, see (1) the application for amendment dated April 25, 1983, (2) Amendment No. 85 to License No. DPR-65 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Waterford Public Library, Rope Ferry Road, Waterford, Connecticut. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 5th day of May, 1983.

FOR THE NUCLEAR REGULATORY COMMISSION


Charles M. Trammell, Acting Chief
Operating Reactors Branch #3
Division of Licensing