

005 MS-010

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MEMORANDUM FOR: Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

FROM: Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

SUBJECT: REQUEST FOR PUBLICATION IN MONTHLY FR NOTICE -
NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT
HAZARDS CONSIDERATION DETERMINATION AND OPPORTUNITY FOR
A HEARING

Northeast Nuclear Energy Company, et al., Docket No. 50-336, Millstone Nuclear
Power Station Unit 2, New London County, Connecticut

Date of amendment request: April 13, 1983

Description of amendment request: The proposed changes in the Technical Specifications result primarily from Cycle 6 refueling considerations and the probability that additional steam generator tubes will need to be plugged because of corrosion effects. These changes include: (1) reduction of the required primary coolant flow rate from 362,600 gpm to 350,000 gpm to correspond with a steam generator tube plugging level of 15.3% (2500 tubes); (2) reduction of the total radial peaking factor (Fr) from 1.597 to 1.565 to offset the departure from nucleate boiling (DNB) analysis penalty which would result from reduction of the primary coolant flow rate to 350,000 gpm; (3) modification of the axial shape index (ASI) monitoring tent to ensure that rated power operation of the reactor core within the tent parameters will not exceed the 15.6 kw/ft linear heat rate following the Cycle 6 refueling; (4) revision of the total planar peaking factor (Fxy) curve to restore its monitoring limits to those at the original Beginning-of-Cycle 5 values; (5) restoration of the control element assembly (CEA) drop time from 3.1 seconds to 2.75

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seconds; (6) revision of the auxiliary feedwater (AFW) system action specification 3.7.1.2 to allow 72 hours instead of the present 48 hours to restore the required pumps to operable status, and to impose additional requirements if more than one auxiliary feedwater pump is inoperable. The associated AFW surveillance requirement 4.7.1.2.a.2(b) would also be modified to eliminate demonstration prior to entry into Mode 3 (hot standby) that the steam turbine-driven auxiliary feedwater pump can develop a discharge pressure of greater than or equal to 1080 psig on recirculation flow. The latter change recognizes that insufficient secondary steam pressure may be available in Mode 4 (hot shutdown) for the turbine driven pump to develop the specified pressure.

Basis for proposed no significant hazards consideration determination:

Considering the plant design changes resulting from the reload and the additional steam generator tube plugging, the licensee's reanalysis of the Loss-of-Reactor-Coolant-Flow event indicates to increase in consequences from the previously docketed analyses. The reanalysis of the hypothetical Control Rod Withdrawal (CRW) event results in a slightly lower Departure from Nucleate Boiling Ratio and the reanalysis of the Steam Line Rupture (SLR) event results in a slightly greater return to power (reactivity). Although the consequences of CRW and SLR events represent an increase over the current licensing basis and therefore require detailed staff review, they are within the acceptable limits defined in the Millstone Unit 2 Technical Specification bases and the Final Safety Analysis Report (FSAR).

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The licensee's reanalysis of the Steam Generator Tube Rupture (SGTR) event for Cycle 6 operation also results in a need for detailed staff review due to a small increase in dose consequences over those docketed in the February 12, 1979 submittal and the FSAR. However, these results are well within acceptable dose criteria and the current Technical Specification limits provide an adequate margin of safety.

The proposed reduction of CEA drop time from 3.1 to 2.75 seconds is a more stringent limitation than currently called for in the technical specifications.

The proposed revision of the AFW system specifications are in accordance with Standard Technical Specifications and are therefore acceptable to the staff.

Based on the foregoing conclusions and using the criteria contained in 10 CFR 50.92 the staff proposes to determine that this application does not involve a significant hazards consideration since the amendment would not involve a significant increase in the consequences of an accident previously evaluated for this facility.

Local Public Document Room location: Waterford Public Library, Rope Ferry Road, Waterford, Connecticut

Attorney for licensee: W. H. Cuddy, Esq., Day, Berry & Howard, One Constitution Avenue, Hartford Connecticut 06103

NRC Branch Chief: Robert A. Clark

Original signed by
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*See previous page for concurrence.

*No by objectors
Correction noted
above.*

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DATE		6/3/83	6/3/83	6/3/83	6/6/83	6/23/83

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 basis and therefore an ~~unreviewed safety question~~, they are well within the acceptable limits defined in the Millstone Unit 2 Technical Specification bases and the Final Safety Analysis Report (FSAR).

The licensee's reanalysis of the Steam Generator Tube Rupture (SGTR) event for Cycle 6 operation also results in ^{STEP} an ~~unreviewed safety question~~ due to a small increase in dose consequences over those docketed in the February 12, 1979 submittal and the FSAR. However, these results are well within acceptable dose criteria and the current Technical Specification limits provide an adequate margin of safety.

The proposed reduction of CEA drop time from 3.1 to 2.75 seconds is a more stringent limitation and it is therefore consistent with the Commission's example (ii) of 48 FR 14870.

The proposed modifications of the AFW system specifications are in accordance with Standard Technical Specifications and are therefore consistent with example (i) of the Commission's guidance (48 FR 14870).

Based on the foregoing conclusions, the Commission has made a proposed determination that this amendment request involves no significant hazards considerations.

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See attached note

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