

MAY 12 1979

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Docket No. 50-336

Mr. W. G. Council, Vice President
Nuclear Engineering & Operations
Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06101

Dear Mr. Council:

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2. This amendment consists of changes to the Technical Specifications in partial response to your applications dated December 16, 1977, December 15, 1978, February 12, 1979 and March 2, 1979 as supplemented on numerous other dates.

This amendment authorizes Cycle 3 operation at 2560 MWt with:

- o Modified (sleeved and reduced flow) guide tubes for the control element assemblies;
- o Credit taken for charging pump flow in the loss-of-coolant accident analysis;
- o A new reactor protection system trip from a reactor coolant pump speed sensing signal; and
- o Installation of a new neutron shield.

The amendment revises the Appendix A Technical Specifications by:

- o Incorporating changes resulting from the analyses of Cycle 3 reload fuel at 2700 MWt;
- o Adding a containment air recirculation system response time;

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(see reports
for enclosure)

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- o approving operations at reduced power level with inoperable main steam line safety valves; and
- o Removing the completed special steam generator surveillance requirements.

In addition, the safety evaluation supporting this amendment addresses our evaluation of:

- o New meteorological data;
- o Fuel handling accident inside containment;
- o Engineered safety features component leakage outside containment;
- o Control room habitability after postulated loss-of-coolant accident;
- o Environmental qualifications of safety-related equipment;
- o Containment electrical penetrations; and
- o Piping and support systems.

Our evaluation of Cycle 3 operation at power levels up to 2700 MWt has been completed as documented in the enclosed safety evaluation. However, you are to restrict operation, as required by License Condition 2.C.(1), to 2560 MWt until: (1) the Advisory Committee on Reactor Safeguards has reviewed and recommended approval of the power increase; and (2) the Commission has issued a subsequent amendment to that effect.

On March 28, 1979, Three Mile Island Unit No. 2 (TMI-2) experienced core damage which resulted from a series of events which were initiated by a Loss of Feedwater Event and apparently compounded by operational errors. We believe that several aspects of this accident have generic applicability to all light water power reactor facilities such as Millstone-2. To identify corrective actions to be taken by all licensees, I&E bulletins have been issued since the TMI-2 Accident. The particular bulletin that applies to the CE facilities is Bulletin No. 79-06B.

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NNECO provided their response to Bulletin No. 79-06B in a letter from W. Council to B. Grier dated April 24, 1979. Our evaluation of the response indicates that the actions taken by NNECO demonstrate understanding of the salient concerns arising from the TMI-2 Accident in reviewing their implications on Millstone-2 operations, and provide added assurance for the protection of the public health and safety during plant operation. A separate safety evaluation will be issued documenting our review of the NNECO response to I&E Bulletin No. 79-06B and identifying certain areas where additional information or action is needed.

In the process of our evaluation of your request, we find the following items need your attention as documented herein. For each item, your staff has agreed to supply the documentation indicated on the schedule shown.

1. Propose Technical Specification changes to provide continued assurance that the radiological consequences of engineered safety features component leakage outside containment are acceptable. Such a request for an amendment should be submitted within 90 days of the date of this amendment.
2. Provide at least 90 days before shut down for the Cycle 4 reload outage: (a) a proposal for a permanent type repair of the containment electrical penetrations; (b) an evaluation program (including the planned inspections) to determine the amount of guide tube wear experienced after two cycles of operation with sleeved fuel assemblies and one cycle with the reduced flow demonstration test; and (c) the plans for the Cycle 4 reload outage steam generator inspection to determine tube denting and support plate integrity.
3. Perform a multiple-frequency and multiaxis test in accordance with IEEE 344-1975 on the proximity probe and transmitter used in the reactor coolant pump speed sensing system prior to the startup from the Cycle 4 refueling outage.
4. Provide an assessment of the neutron dose rate reduction and the actual man-rem exposure savings experienced during Cycle 3 operation due to the installation of the neutron shield in the area of the reactor cavity.

Some portions of your proposed Technical Specifications have been modified to meet our requirements. These modifications have been discussed with and agreed to by your staff.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original signed by
Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 52 to DPR-65 (see reports)
- 2. Safety Evaluation
- 3. Notice

cc w/enclosure: See next page

* See previous yellow for concurrence

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 V. NOONAN
 5/11/79

[Signature]
 AD-E&I: DOR
 BGrimes
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Robert W. Reid, Chief
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*Oral agreement
 for DLE 5/11/79*
*Amendment of PR
 Notice only. Conference
 conducted on 5/11/79
 change to 4.2.C(2) of
 amendment*

JHB 5/8/79
JRB 5/08/79
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