

Docket Nos. 50-280
and 50-281

April 11, 1990

Posted

Amolt. 138 to DPR-37

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: ENRICHED
FISSIONABLE MATERIAL (TAC NOS. 76005 AND 76006)

The Commission has issued the enclosed Amendment No. 138 to Facility Operating License No. DPR-32 and Amendment No. 138 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated December 11, 1989.

These amendments delete Technical Specification 5.3.A.7, which stated that up to 10 grams of enriched fissionable material may be used either in the core or available on site, in the form of fabricated neutron flux detectors, for the purposes of monitoring core neutron flux.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Bart C. Buckley, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 138 to DPR-32
- 2. Amendment No. 138 to DPR-37
- 3. Safety Evaluation

cc w/enclosures:
See next page

OFC :	LA:PD22	PM:PD22	SPYB	:D:PD22	:OGC	:	:
NAME :	DM/Ter	BBuckley	RJones	HBerkow	CPW	:	:
DATE :	3/20/90	3/20/90	3/21/90	3/22/90	3/21/90	:	:

Mr. W. L. Stewart
Virginia Electric and Power Company

Surry Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138
License No. DPR-32

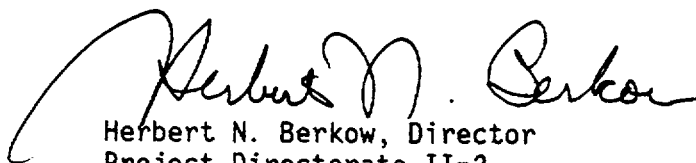
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated December 11, 1989 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 138, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 11, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 138
License No. DPR-37

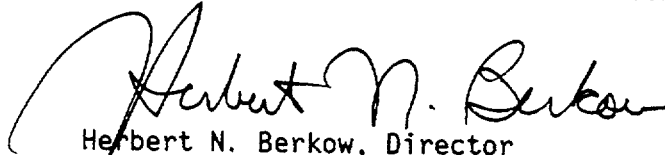
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated December 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 138, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 11, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 138 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 138 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Page

TS 5.3-3

Insert Page

TS 5.3-3

- b. The moderator temperature coefficient in the power operating range is less than or equal to:
 - 1) +3.0 pcm/°F at less than 50% of rated power, or
 - 2) +3.0 pcm/°F at 50% of rated power and linearly decreasing to 0 pcm/°F at rated power.
- c. Capable of being made subcritical in accordance with Specification 3.12.A.3.C.

B. Reactor Coolant System

- 1. The design of the Reactor Coolant System complies with the code requirements specified in Section 4 of the FSAR.
- 2. All piping, components, and supporting structures of the Reactor Coolant System are designed to Class 1 seismic requirements, and have been designed to withstand:
 - a. Primary operating stresses combined with the Operational seismic stresses resulting from a horizontal ground acceleration of 0.07g and a simultaneous vertical ground acceleration of 2/3 the horizontal, with the stresses maintained within code allowable working stresses.
 - b. Primary operating stresses when combined with the Design Basis Earthquake seismic stresses resulting from a horizontal ground acceleration of 0.15g and a simultaneous vertical ground



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 138 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 138 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

Pursuant to 10 CFR 50.90, Virginia Electric and Power Company (VEPCO) proposed to amend Facility Operating Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station, Units 1 and 2. By letter dated December 11, 1989, VEPCO proposed to delete Technical Specification (TS) 5.3.A.7 which states that up to 10 grams of enriched fissionable material may be used either in the core or available on site, in the form of fabricated neutron flux detectors, for the purposes of monitoring core flux.

2.0 DISCUSSION AND EVALUATION

License condition 2.C of the Surry Power Station Facility Operating Licenses Nos. DPR-32 and DPR-37, allows the receipt, possession, and use of special nuclear material as fission detectors, as well as other uses, in amounts as required. However, as cited in Section 1.0 above, TS 5.3.A.7 permits up to 10 grams of enriched fissionable material to be used either in the core or available on site, in the form of fabricated neutron flux detectors, to monitor core neutron flux. VEPCO has stated that greater than 10 grams of enriched fissionable material is presently in use at the Surry units in the form of core neutron flux detectors in accordance with the license condition.

VEPCO conducted an investigation to ascertain the source of TS 5.3.A.7 and noted that TS 5.3.A.7 was part of the TS submittal, but could not identify any correspondence relating to the 10 gram-figure. The current Standard TS do not have a corresponding figure or discussion of the quantity of fissionable material used on site. VEPCO concluded that the 10-gram figure was included in the original TS for information purposes only and was not intended to limit the quantity of special nuclear material used in neutron flux detectors. This conclusion is supported by provisions of license condition 2.C of the Facility Operating Licenses, as amended, which contains the phrase "... in amounts as required;" with no corresponding limiting condition for operation. The total quantity of special nuclear material used in neutron flux detectors will not alter conclusions resulting from any accident evaluated in the Updated Final Safety Analysis Report.

3.0 SUMMARY

The staff has reviewed VEPCO's submittal of December 11, 1989, and concludes that the proposed amendment to delete TS 5.3.A.7 is acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in a requirement with respect to the installation or use of the facilities components located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 11, 1990

Principal Contributor:

B. Buckley