

MAY 06 1981

Docket No. 50-336

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Mr. W. G. Council, Vice President  
 Nuclear Engineering & Operations  
 Northeast Nuclear Energy Company  
 P. O. Box 270  
 Hartford, Connecticut 06101

Dear Mr. Council:

The Commission has issued the enclosed Amendment No. 68 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2. This amendment consists of changes to the Technical Specifications (TS) in response to your application dated May 5, 1981.

The changes to the TS incorporate minor changes to certain TS issued by Amendment 66, dated April 7, 1981.

The proposed TS changes are to:

1. Add to Action Statement 3 on TS page 3/4 3-48, "This action is not required if the PORV block valve is closed with power removed in accordance with Specification 3.4.3a or 3.4.3b.";
2. Change the Channel Check requirement for, "5. PORV Block Valve Position Indicator" in Table 4.3-7 from M (monthly) to N.A. (not applicable);
3. Add an Item c. to Specification 3.4.3 to read, "c. The provisions of Specification 3.04 are not applicable.", and
4. Indicate the cycling of a PORV block valve is not required if such block valve is closed to meet Specification 3.4.3a or b.

We find the proposed TS changes are administrative in nature and, therefore, acceptable for the following reasons.

1. It is not necessary to take data once per shift to detect any individual PORV leakage if the block valve is confirmed closed as authorized by the TS 3.4.3 ACTION Statements a or b.
2. The monthly CHANNEL CHECK has not meaning in regards to a limit switch type valve position indicator; however, it does for the acoustic monitor channels used to detect PORV and Safety Valve leakage.

CP  
1

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OFFICE							
SURNAME							
DATE							

3. This provision, allowing a change in Mode when a PORV is out-of-service is contained in our current Standard TS, issued September 16, 1980, for Westinghouse facilities. It is an oversight that the provision was not included in our July 2, 1980 request for TS changes nor in the licensee's application for TS changes dated September 16, 1980.
4. It is not reasonable to cycle a PORV block valve for Surveillance Requirement 4.4.3.1 if the block valve is closed and power removed to block PORV leakage or ensure closure of a block valve is inoperable.

Certain modifications to your proposed changes were necessary to meet our requirements. These modifications have been discussed with and agreed to by your staff.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment. The amendment applies to administrative details. Therefore we have determined that it does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

Copies of the related Notice of Issuance are also enclosed.

Original signed by  
Robert A. Clark

Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

1. Amendment No. 68 to DPR-65
2. Notice of Issuance

cc w/enclosures:  
See next page

OFFICE	ORB#3:DL	ORB#3:DL	ORB#3:DL	AD-OR-DL	OR-D		
SURNAME	PK Kreutzer	EC Conner	RA Clark	TM Novak	L.R. GRAY		
DATE	5/5/81	5/5/81	5/5/81	5/5/81	5/5/81		

*Amend. & NE Notice only*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555

DISTRIBUTION:  
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PMKreutzer

Docket No. 50-336

Docketing and Service Section  
Office of the Secretary of the Commission

SUBJECT: NORTHEAST NUCLEAR ENERGY COMPANY, ET AL., Millstone Nuclear Power Station

Two signed originals of the Federal Register Notice identified below are enclosed for your transmittal to the Office of the Federal Register for publication. Additional conformed copies (12 ) of the Notice are enclosed for your use.

- Notice of Receipt of Application for Construction Permit(s) and Operating License(s).
- Notice of Receipt of Partial Application for Construction Permit(s) and Facility License(s): Time for Submission of Views on Antitrust Matters.
- Notice of Availability of Applicant's Environmental Report.
- Notice of Proposed Issuance of Amendment to Facility Operating License.
- Notice of Receipt of Application for Facility License(s); Notice of Availability of Applicant's Environmental Report; and Notice of Consideration of Issuance of Facility License(s) and Notice of Opportunity for Hearing.
- Notice of Availability of NRC Draft/Final Environmental Statement.
- Notice of Limited Work Authorization.
- Notice of Availability of Safety Evaluation Report.
- Notice of Issuance of Construction Permit(s).
- Notice of Issuance of Facility Operating License(s) or Amendment(s).

Other: Amendment No. 68.  
Referenced documents have been provided PDR.

Division of Licensing, ORB#3  
Office of Nuclear Reactor Regulation

Enclosure:  
As Stated

OFFICE →	ORB#3:DL					
SURNAME →	PMKreutzer/pr					
DATE →	5/6/81					



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

May 6, 1981

Docket No. 50-336

Mr. W. G. Council, Vice President  
Nuclear Engineering & Operations  
Northeast Nuclear Energy Company  
P. O. Box 270  
Hartford, Connecticut 06101

Dear Mr. Council:

The Commission has issued the enclosed Amendment No. 68 to Facility Operating License No. DPR-65 for Millstone Nuclear Power Station, Unit No. 2. This amendment consists of changes to the Technical Specifications (TS) in response to your application dated May 5, 1981.

The changes to the TS incorporate minor changes to certain TS issued by Amendment 66, dated April 7, 1981.

The proposed TS changes are to:

1. Add to Action Statement 3 on TS page 3/4 3-48, "This action is not required if the PORV block valve is closed with power removed in accordance with Specification 3.4.3a or 3.4.3b.";
2. Change the Channel Check requirement for, "5. PORV Block Valve Position Indicator" in Table 4.3-7 from M (monthly) to N.A. (not applicable);
3. Add an Item c. to Specification 3.4.3 to read, "c. The provisions of Specification 3.04 are not applicable.", and
4. Indicate the cycling of a PORV block valve is not required if such block valve is closed to meet Specification 3.4.3a or b.

We find the proposed TS changes are administrative in nature and, therefore, acceptable for the following reasons.

1. It is not necessary to take data once per shift to detect any individual PORV leakage if the block valve is confirmed closed as authorized by the TS 3.4.3 ACTION Statements a or b.
2. The monthly CHANNEL CHECK has no meaning in regards to a limit switch type valve position indicator; however, it does for the acoustic monitor channels used to detect PORV and Safety Valve leakage.

3. This provision, allowing a change in Mode when a PORV is out-of-service, is contained in our current Standard TS, issued September 16, 1980, for Westinghouse facilities. It is an oversight that the provision was not included in our July 2, 1980 request for TS changes nor in the licensee's application for TS changes dated September 16, 1980.
4. It is not reasonable to cycle a PORV block valve for Surveillance Requirement 4.4.3.1 if the block valve is closed and power removed to block PORV leakage or ensure closure if a block valve is inoperable.

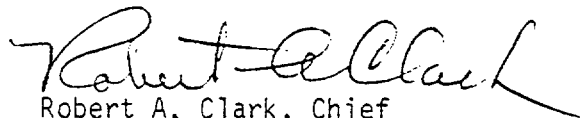
Certain modifications to your proposed changes were necessary to meet our requirements. These modifications have been discussed with and agreed to by your staff.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment. The amendment applies to administrative details. Therefore we have determined that it does not authorize a change in effluent types or total amounts nor an increase in power level, and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

Copies of the related Notice of Issuance are also enclosed.

Sincerely,



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Enclosures:

1. Amendment No. 68 to DPR-65
2. Notice of Issuance

cc w/enclosures:  
See next page

Northeast Nuclear Energy Company

cc:

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Day, Berry & Howard  
Counselors at Law  
One Constitution Plaza  
Hartford, Connecticut 06103

Anthony Z. Roisman  
Natural Resources Defense Council  
917 15th Street, N.W.  
Washington, D. C. 20005

Mr. Lawrence Bettencourt, First Selectman  
Town of Waterford  
Hall of Records - 200 Boston Post Road  
Waterford, Connecticut 06385

Northeast Nuclear Energy Company  
ATTN: Superintendent  
Millstone Plant  
Post Office Box 128  
Waterford, Connecticut 06385

Waterford Public Library  
Rope Ferry Road, Route 156  
Waterford, Connecticut 06385

Director, Criteria and Standards Division  
Office of Radiation Programs (ANR-460)  
U.S. Environmental Protection Agency  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Region I Office  
ATTN: EIS COORDINATOR  
John F. Kennedy Federal Building  
Boston, Massachusetts 02203

Northeast Utilities Service Company  
ATTN: Mr. James R. Himmelwright  
Nuclear Engineering and Operations  
P. O. Box 270  
Hartford, Connecticut 06101

Mr. John Shedlosky  
Resident Inspector/Millstone  
c/o U.S.N.R.C.  
P. O. Drawer KK  
Niantic, CT 06357

Mr. Charles Brinkman  
Manager - Washington Nuclear  
Operations  
C-E Power Systems  
Combustion Engineering, Inc.  
4853 Cordell Aven., Suite A-1  
Bethesda, MD 20014

cc w/enclosure(s) and incoming  
dtd: 5/5/81

Connecticut Energy Agency  
ATTN: Assistant Director, Research  
and Policy Development  
Department of Planning and Energy  
Policy  
20 Grand Street  
Hartford, CT 06106



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHEAST NUCLEAR ENERGY COMPANY  
THE CONNECTICUT LIGHT AND POWER COMPANY  
THE HARTFORD ELECTRIC LIGHT COMPANY  
THE WESTERN MASSACHUSETTS ELECTRIC COMPANY  
DOCKET NO. 50-336  
MILLSTONE NUCLEAR POWER STATION, UNIT NO. 2  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 68  
License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northeast Nuclear Energy Company (the licensee) dated May 5, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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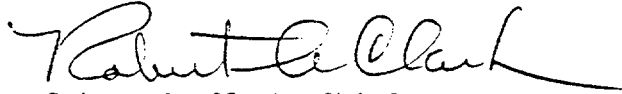
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-65 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 68, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance:



ATTACHMENT TO LICENSE AMENDMENT NO. 68

FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 3-48

3/4 3-49

3/4 4-3

TABLE 3.3-11ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>ACTION</u>
1. Pressurizer Water Level	2	1	1
2. Auxiliary Feedwater Flow Rate	1/S. G.	1/S. G.	1
3. RCS Subcooling Margin Monitor	1	1	2
4. PORV Position Indicator Acoustic Flow Monitor	1/valve	1/valve	3
5. PORV Block Valve Position Indicator	1/valve	1/valve	3
6. Safety Valve Position Indicator Acoustic Flow Monitor	1/valve	1/valve	3

TABLE 3.3-11 (Continued)

ACTION STATEMENTS

- ACTION 1 - With the number of OPERABLE channels less than required by Table 3.3-11, either restore the inoperable channel(s) to OPERABLE status within 30 days or be in HOT STANDBY within the next 12 hours.
- ACTION 2 - With the subcooling margin monitor INOPERABLE, determine the subcooling margin once per 12 hours.
- ACTION 3 - With any individual valve position indicator inoperable, obtain quench tank temperature, level and pressure information, and monitor discharge pipe temperature once per shift to determine valve position. This action is not required if the PORV block valve is closed with power removed in accordance with Specification 3.4.3.a or 3.4.3.b.

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Pressurizer Water Level	M	R
2. Auxiliary Feedwater Flow Rate	M	R
3. Reactor Coolant System Subcooling Margin Monitor	M	R
4. PORV Position Indicator (Acoustic Monitor)	M	R
5. PORV Block Valve Position Indicator	N.A.	R
6. Safety Valve Position Indicator (Acoustic Monitor)	M	R

REACTOR COOLANT SYSTEM

RELIEF VALVES

LIMITING CONDITION FOR OPERATION

---

3.4.3 Two power operated relief valves (PORVs) and their associated block valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With one or more PORV(s) inoperable, within 8 hours either restore the PORV(s) to OPERABLE status or close the associated block valve(s) and remove power from the block valve(s); otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one or more block valve(s) inoperable, within 8 hours either restore the block valve(s) to OPERABLE status or close the block valve(s) and remove power from the block valve(s); otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

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4.4.3.1 Each PORV shall be demonstrated OPERABLE:

- a. Once per 31 days by performance of a CHANNEL FUNCTIONAL TEST, excluding valve operation, and
- b. Once per 18 months by performance of a CHANNEL CALIBRATION.

4.4.3.2 Each block valve shall be demonstrated OPERABLE once per 92 days by operating the valve through one complete cycle of full travel. This demonstration is not required if a PORV block valve is closed and power removed to meet Specification 3.4.3 a or b.

## REACTOR COOLANT SYSTEM

### PRESSURIZER

#### LIMITING CONDITION FOR OPERATION

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3.4.4 The pressurizer shall be OPERABLE with a steam bubble and with at least 130 kw of pressurizer heater capacity capable of being supplied by emergency power. The pressurizer level shall be within  $\pm 5\%$  of its programmed value.

APPLICABILITY: MODES 1, 2 and 3.

#### ACTION:

- A. With the pressurizer inoperable due to an inoperable emergency power supply to the pressurizer heaters either restore the inoperable emergency power supply within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 12 hours.
- B. With the pressurizer otherwise inoperable, be in at least HOT STANDBY with the reactor trip breakers open within 6 hours and in HOT SHUTDOWN within the following 6 hours.

#### SURVEILLANCE REQUIREMENTS

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4.4.4 The pressurizer water level shall be determined to be within  $\pm 5\%$  of its programmed value at least once per 12 hours.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-336NORTHEAST NUCLEAR ENERGY COMPANY, ET AL.NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 68 to Facility Operating License No. DPR-65 issued to the Northeast Nuclear Energy Company, the Connecticut Light and Power Company, the Hartford Electric Light Company, and the Western Massachusetts Electric Company (the licensee), which revised Technical Specifications for operation of the Millstone Nuclear Power Station, Unit No. 2 (the facility) located in the Town of Waterford, Connecticut. The amendment is effective as of its date of issuance.

The amendment changes the Technical Specification to incorporate certain minor modifications to the changes previously issued by Amendment 66 (TMI Lesson Learned Technical Specifications) issued April 7, 1981. Specifically, the amendment eliminates unnecessary requirements and allows reactor startup when a power operated relief valve is inoperable.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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- 2 -

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated May 5, 1981, (2) Amendment No. 68 to License No. DPR-65, and (3) the Commission's letter dated May 6, 1981. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Waterford Public Library, Rope Ferry Road, Waterford, Connecticut. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 6th day of May, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief  
Operating Reactors Branch #3  
Division of Licensing