

December 4, 1990

Docket No. 50-281

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Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY UNIT 2 - ISSUANCE OF AMENDMENT RE: TYPE C LEAK RATE TEST INTERVAL
(TAC NO. 77735)

The Commission has issued the enclosed Amendment No. 145 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated September 14, 1990.

The amendment documents in the Technical Specifications a one-time exemption from requirements of Appendix J to 10 CFR Part 50. This exemption was granted on September 26, 1990, and permits delaying the Type C local leak rate tests for 76 containment isolation valves associated with 42 penetrations until April 30, 1991. It is our understanding that VEPCO intends to test these valves during the Cycle 10 refueling outage, scheduled to start April 5, 1991, or during an earlier outage of sufficient duration.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

Bart C. Buckley, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 145 to DPR-37
- 2. Safety Evaluation

cc w/enclosures:
See next page

OFC :	LA:PD22	: PM:PD22	: PM:PD22	: D:PD22	: OGC	:
NAME :	Miller	: JWilliams	: jd:Buckley	: HBerlow	: E HOLLER	:
DATE :	11/9/90	: 11/13/90	: 11/13/90	: 11/13/90	: 11/16/90	:

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Mr. W. L. Stewart
Virginia Electric and Power Company

Surry Power Station

cc:

Michael W. Maupin, Esq.
Hunton and Williams
Post Office Box 1535
Richmond, Virginia 23212

Attorney General
Supreme Court Building
101 North 8th Street
Richmond, Virginia 23219

Mr. Michael R. Kansler, Manager
Surry Power Station
Post Office Box 315
Surry, Virginia 23883

Mr. E. Wayne Harrell
Vice President - Nuclear Operations
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Senior Resident Inspector
Surry Power Station
U.S. Nuclear Regulatory Commission
Post Office Box 166, Route 1
Surry, Virginia 23883

Mr. J. P. O'Hanlon
Vice President - Nuclear Services
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. Sherlock Holmes, Chairman
Board of Supervisors of Surry County
Surry County Courthouse
Surry, Virginia 23683

Mr. R. F. Saunders
Manager - Nuclear Licensing
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Mr. W. T. Lough
Virginia Corporation Commission
Division of Energy Regulation
Post Office Box 1197
Richmond, Virginia 23209

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street N.W., Suite 2900
Atlanta, Georgia 30323

C. M. G. BATTERY, M.D., M.P.H.
Department of Health
109 Governor Street
Richmond, Virginia 23219

DATED: December 4, 1990
AMENDMENT NO. 145 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

G. Lainas, 14/H/3

H. Berkow

D. Miller

B. Buckley

OGC-WF

D. Hagan, 3302 MNBB

E. Jordan, 3302 MNBB

B. Grimes, 9/A/2

G. Hill (4), P-137

Wanda Jones, P-130A

J. Calvo, 11/F/23

J. Williams

ACRS (10)

GPA/PA

OC/LFMB

M. Sinkule, R-II

Others as required

cc: Plant Service list



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 145
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 14, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

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(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 145, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 4, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 145 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Revise Appendix A as follows:

Remove Pages

TS 4.4-2

Insert Pages

TS 4.4-2

- b. The leakage rate test will be performed at a pressure of at least 39.2 psig (P_a).
 - c. The measured leakage rate L_{am} shall not exceed 75% of the design basis accident leakage rate (L_a) of 0.1 weight percent per 24 hours at pressure P_a .
2. Type B and C tests will be performed at a pressure of at least 39.2 psig (P_a) in accordance with the provisions of Appendix J, Section III.B and C.*† Also, within 72 hours after use of the personnel airlock, the seals will be tested at least at the peak calculated accident pressure to verify that they are properly sealed.

C. Acceptance Criteria

Type A, B, and C tests will be considered to be satisfactory if the acceptance criteria delineated in Appendix J, Sections III.A.5(b), III.B.3, and III.C.3 are met.

D. Retest Schedule

The retest schedules for Type A, B, and C tests will be in accordance with Section III.D of Appendix J.*†

E. Inspection and Reporting of Tests

Inspection and reporting of tests will be in accordance with Section V of Appendix J.

* Type C testing for Unit 1 can be deferred beyond the 2 year Appendix J requirement until the end of Operating Cycle 10 but the deferral shall expire no later than December 31, 1990 in accordance with the NRC exemption received on June 22, 1990.

† Type C testing for Unit 2 can be deferred beyond the 2 year Appendix J requirement until the end of Operating Cycle 10 but the deferral shall expire no later than April 30, 1991 in accordance with the NRC exemption dated September 26, 1990.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

December 1, 1990

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 145 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NO. 2

DOCKET NO. 50-281

1.0 INTRODUCTION

By letter dated September 14, 1990, Virginia Electric and Power Company (VEPCO) requested a one-time exemption from the requirements of 10 CFR Part 50, Appendix J, pertaining to the Type C leak rate test interval. Paragraph III.D.3 of Appendix J requires that a Type C test be performed during each shutdown for refueling, but in no case at intervals greater than 2 years. VEPCO requested extension of the 2-year interval to June 30, 1991, to permit Type C testing during the Unit 2 refueling outage scheduled to begin in April 1991. On September 26, 1990, the staff issued the exemption; however, the exemption will only be effective until April 30, 1991.

In a separate letter dated September 14, 1990, VEPCO also requested to add an administrative footnote to the Surry Power Station, Unit No. 2 Technical Specifications. This footnote is intended to document the modified test interval provided in the September 26, 1990 exemption.

2.0 EVALUATION

This amendment is administrative in nature, documenting the exemption issued on September 26, 1990 into the plant Technical Specifications. The original amendment proposed by VEPCO was based on the requested test interval extension to June 30, 1991. However, the exemption issued by the staff permits extension only to April 30, 1991. The final amendment is consistent with the exemption.

The proposed amendment does not permit facility operation outside the scope of the September 26, 1990 exemption. Therefore, the Safety Evaluation issued with the exemption is unaffected by, and, therefore applicable to this amendment.

3.0 SUMMARY

Based on the above, the staff has concluded the addition of the footnote to the Surry Power Station, Unit No. 2 Technical Specifications is acceptable. The footnote accepted by the staff documents the September 26, 1990 extension of the Type C test interval to April 30, 1991.

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4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

5.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 4, 1990

Principal Contributor:
Joe Williams