

DO NOT REMOVE

Docket Nos. 50-280
and 50-281

Posted
Ammt. 148 to DPR-37

Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: SUPERVISOR
SHIFT OPERATIONS (TAC NOS. 77944 AND 77945)

The Commission has issued the enclosed Amendment No. 151 to Facility Operating License No. DPR-32 and Amendment No. 148 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated October 11, 1990, as supplemented October 12, 1990.

These amendments add the position of Supervisor Shift Operations to the Surry TS. The individual in this position would be required to fulfill the functional and qualification requirements of the "Operations Manager" as required by ANS-3.1 (12/79 Draft).

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Bart C. Buckley, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 151 to DPR-32
- 2. Amendment No. 148 to DPR-37
- 3. Safety Evaluation

cc w/enclosures:
See next page

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| DATE | : 12/10/90 | : 12/10/90 | : 12/10/90 | : 12/11/90 | : 12/10/90 | : | : |

Mr. W. L. Stewart
Virginia Electric and Power Company

Surry Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 151
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 11, 1990, as supplemented October 12, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

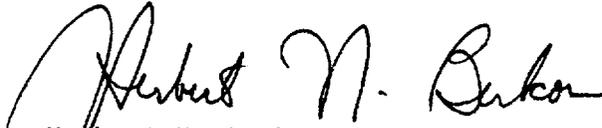
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 151, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 11, 1990, as supplemented October 12, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 148, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 31, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 151 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 148 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 6.1-1
TS 6.1-2

Insert Pages

TS 6.1-1
TS 6.1-2

B. The facility organization shall conform to the following requirements:

1. Each member of the facility staff shall meet or exceed the minimum qualifications of ANS-3.1 (12/79 Draft)* for comparable positions, and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except:
 - a. The Superintendent - Radiological Protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.
 - b. The Superintendent - Operations shall hold (or have previously held) a Senior Reactor Operator License for Surry Power Station or a similar design Pressurized Water Reactor plant.
 - c. The Supervisor Shift Operations shall hold an active Senior Reactor Operator License for Surry Power Station.
2. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.
3. The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5 of ANS-3.1 (12/79 Draft* and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the operating experience program.
4. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.
5. A health physics technician shall be on site when fuel is in the reactor.
6. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operational Phase"

6.0 ADMINISTRATIVE CONTROLS

6.1 Organization, Safety and Operation Review

Specification

- A. The Station Manager shall be responsible for the overall operation of the facility. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for the safe operation of the facility. During the absence of both, the Station Manager will delegate in writing the succession to this responsibility.

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.
2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
3. The Vice President - Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining and providing technical support to the plant to ensure nuclear safety.
4. The management position responsible for training of the operating staff and the position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.
5. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 151 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO.148 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated October 11, 1990, as supplemented October 12, 1990, the Virginia Electric and Power Company (the licensee) proposed a change to the Technical Specifications (TS) for the Surry Power Station, Units No. 1 and No. 2 (SPS-1&2). Specifically, the change would provide for and document exceptions to ANS-3.1 (12/79 Draft) in the SPS-1&2 station staff organization and the qualification requirements regarding the position of Operations Manager as described in the standard.

2.0 DISCUSSION

The proposed change to TS 6.1, "Organization, Safety and Operation Review," adds specific exceptions to the qualification requirements of ANS-3.1 (12/79 Draft) for the Superintendent Operations and the Supervisor Shift Operations.

ANS-3.1 (12/79 Draft), which is cited in TS 6.1 for establishing the qualification requirements of the plant's staff, requires that the individual fulfilling the function of the "Operations Manager" hold a current Senior Reactor Operator's (SRO) license. In the past, the licensee has determined that the Superintendent Operations is the equivalent position in the licensee's SPS-1&2 organization and therefore, that position has been filled by persons holding SRO licenses.

This requirement makes it difficult for the Superintendent Operations to perform certain management functions. Specifically, the ability to monitor the quality of operating shift qualification and requalification programs is substantially impaired. The Superintendent is not free to fully examine the training programs in progress, simply because he is also a trainee and is thus restricted from obtaining certain information. In addition, a substantial part of the Superintendent Operations' time is consumed in maintaining the SRO license in an active status, which requires 60 days of requalification time each year.

To relieve the Superintendent of these requirements so he can better fulfill his management responsibilities, the licensee would institute an intermediate position, directly subordinate to the Superintendent Operations, that has cognizance over all SPS-1&2 plant operating shifts. This position would be the Supervisor Shift Operations, who would be required to maintain a current and active SRO license and would fulfill the functional and qualification requirements of the "Operations Manager" as required by ANS-3.1 (12/79 Draft).

3.0 EVALUATION

The proposed change, as described above, is administrative in nature, and would change only the staff organizational position of the individual required to maintain an SRO license, thereby fulfilling the functions of the "Operations Manager" as described in ANS-3.1 (Draft 12/79). The required level of training and qualification would not be changed and the levels of responsibility and authority to the "Operations Manager" would remain as the individual immediately superior to the operating shift supervisors. Also, a subsequent version of ANS-3.1, 1987 explicitly provides for the organizational structure described in the licensee's above proposed change. Therefore, we find the proposed change to be acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 31, 1990

Principal Contributor:
B. Buckley