

Docket Nos. 50-280
and 50-281

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Mr. W. L. Stewart
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

*Posted
Amndt. 160 to DPR-37*

Dear Mr. Stewart:

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: CONTROL ROD DROP
TEST FREQUENCY REQUIREMENT (TAC NOS. 75743 AND 75744)

The Commission has issued the enclosed Amendment No. 161 to Facility Operating License No. DPR-32 and Amendment No. 160 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated October 12, 1989.

These amendments replace the portion of the control rod drop time test frequency requirement associated with "the breach of the reactor coolant system integrity" with conditions similiar to the Westinghouse Standard Technical Specifications.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Bart C. Buckley, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 161 to DPR-32
- 2. Amendment No. 160 to DPR-37
- 3. Safety Evaluation

cc w/enclosures:
See next page

OFC	LA:PD22	PE22	PD22	SRXB	D:PD22	OGC	
NAME	<i>W. L. Stewart</i>	<i>DDorner</i>	<i>B. Buckley</i>	<i>R. Jones</i>	<i>H. Beck</i>	<i>P. J. Selby</i>	
DATE	<i>7/24/91</i>	<i>7/24/91</i>	<i>7/26/91</i>	<i>7/29/91</i>	<i>7/29/91</i>	<i>7/31/91</i>	

Mr. W. L. Stewart
Virginia Electric and Power Company

Surry Power Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161
License No. DPR-32

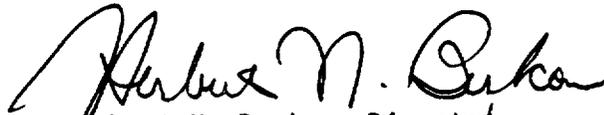
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 12, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 161, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 19, 1991



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 160
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 12, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 160, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 19, 1991

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 161 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 160 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Page

TS 4.1-9b

Insert Page

TS 4.1-9b

TABLE 4.1-2A

MINIMUM FREQUENCY FOR EQUIPMENT TESTS

<u>Description</u>	<u>Test</u>	<u>Frequency</u>	<u>FSAR Section Reference</u>
1. Control Rod Assemblies	Rod drop times of all full length rods at hot conditions	Prior to reactor criticality: a. For all rods following each removal of the reactor vessel head, b. For specially affected individual rods following any maintenance on or modification to the control rod drive system which could affect the drop time of those specific rods, and c. Each refueling shutdown.	7
2. Control Rod Assemblies	Partial movement of all rods	Every 2 weeks	7
3. Refueling Water Chemical Addition Tank	Functional	Each refueling shutdown	6
4. Pressurizer Safety Valves	Setpoint	Per TS 4.0.3	4
5. Main Steam Safety Valves	Setpoint	Per TS 4.0.3	10
6. Containment Isolation Trip	*Functional	Each refueling shutdown	5
7. Refueling System Interlocks	*Functional	Prior to refueling	9.12
8. Service Water System	*Functional	Each refueling shutdown	9.9
9. Fire Protection Pump and Power Supply	Functional	Monthly	9.10
10. Primary System Leakage	*Evaluate	Daily	4
11. Diesel Fuel Supply	*Fuel Inventory	5 days/week	8.5
12. Boric Acid Piping Heat Tracing Circuits	*Operational	Monthly	9.1
13. Main Steam Line Trip Valves	Functional (Full Closure)	Before each startup (TS 4.7)	10



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 160 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated October 12, 1989, the Virginia Electric and Power Company (the licensee) proposed a change to the Technical Specifications (TS) for the Surry Power Station, Units 1 and 2. Specifically, the proposed change would revise TS Table 4.1-2A by replacing the control rod drop time test frequency requirement associated with "the breach of the Reactor Coolant System integrity" with conditions similar to those in the Westinghouse Standard Technical Specifications (STS), which the NRC staff has approved.

2.0 DISCUSSION

Currently, control rod drop time tests are required "each refueling shutdown or after disassembly or maintenance requiring the breach of the Reactor Coolant System integrity." Literal interpretation of this requirement would result in unnecessary testing following any "breach of Reactor Coolant System integrity" which does not affect the drop time of any control rod, such as maintenance of a loop valve or opening of a power operated relief valve in the Reactor Coolant System. The proposed change would clarify the required test frequency by incorporating terminology from the Westinghouse STS. The revised control rod drop time test frequency requirement would include testing of all rods after each refueling shutdown and each removal of the reactor vessel head, and of "specially affected individual rods following any maintenance on or modification to the control rod drive system which could affect the drop time of those specific rods." This change retains all test frequency requirements intended in the original TS, while removing ambiguous wording which could result in unnecessary testing. Therefore, the staff finds the proposed change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (55 FR 10547). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Dorman

Date: August 19, 1991