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Docket No. 50-281

Mr. R. H. Leasburg Vice President - Nuclear Operations Virginia Electric and Power Company Post Office Box 26666 Richmond, Virginia 23261

Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment No. 72 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit No. 2. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated July 7, 1981.

The amendment provides a one time 60 day extension for the visual inspection surveillance requirement for inaccessible snubbers.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

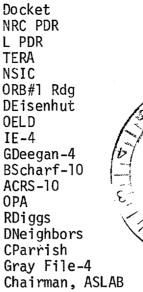
Original signed by: S. A. Varga

Steven A. Varga, Chief Operating Reactors Branch #1 Division of Licensing

Enclosures:

- 1. Amendment No. 72 to DPR-37
- 2. Safety Evaluation
- 3. Notice of Issuance
- cc: w/enclosures See next page

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UNITED STATES PUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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Operating Reactors Branch #1 Division of Licensing

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- 2. Safety Evaluation
- 3. Notice of Issuance

cc: w/enclosures See next page Mr. R. H. Leasburg Virginia Electric and Power Company

cc: Mr. Michael W. Maupin Hunton and Williams Post Office Box 1535 Richmond, Virginia 23213

> Mr. J. L. Wilson, Manager P. O. Box 315 Surry, Virginia 23883

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Mr. Sherlock Holmes, Chairman Board of Supervisors of Surry County Surry County Courthouse, Virginia 23683

Commonwealth of Virginia Council on the Environment 903 Ninth Street Office Building Richmond, Virginia 23219

Attorney General 1101 East Broad Street Richmond, Virginia 23219

Mr. James R. Wittine Commonwealth of Virginia State Corporation Commission Post Office Box 1197 Richmond, Virginia 23209

Director, Criteria and Standards Division Office of Radiation Programs (ANR-460) U. S. Environmental Protection Agency Washington, D. C. 20460

U. S. Environmental Protection Agency Region III Office ATTN: EIS COORDINATOR Curtis Building - 6th Floor 6th and Walnut Streets Philadelphia, Pennsylvania 19106 Mr. J. H. Ferguson Executive Vice President - Power Virginia Electric and Power Company Post Office Box 26666 Richmond, Virginia 23261



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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72 License No. DPR-37

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated July 7, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be infinical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:
 - (B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Americant No. 72, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

Steven A. Varga, Chief Operating Beactors Branch #1 Division of Licensing

FOR THE NUCLEAR REGULATORY COMMISSION

Attachment: Changes to the Technical Specifications

Date of Issuance: July 9, 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Revise Appendix A as follows:

<u>Remove Page</u>	Insert Page
4.17-1	4.17-1
4.17-2	4.17-2

4.17 SHOCK SUPPRESSORS (SNUBBERS)

Applicability

Applies to all hydraulic shock suppressors (soubbers) which are required to protect the reactor coolant system and safety related systems.

Objective

To specify the minimum frequency and type of surveillance to be applied to the hydraulic snubbers listed in Table 4.17-1 and 4.17-2.

Specification

A. All hydraulic shock suppressors whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected.

This inspection shall include but not necessarily be limited to, inspection of the hydraulic-fluid reservoir, fluid connections, and linkage connections to the piping and anchor to verify snubber operability in accordance with the following schedule:

Number of Snubbers	Next Required
Found Inoperable During	Inspection
Inspection or During	Interval
Inspection Interval	

0	
1	
2	

18 Months ± 25%
12 Months ± 25%
6 Months ± 25%

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3, 4	124 Days	+ 25%
5, 6, 7	62 Days	<u>+</u> 25% *
<u>></u> 8	31 Days	<u>+</u> 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized into two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

- B. All hydraulic snubbers whose seal material are other than ethylene propylene or other material that has been demonstrated to be compatible with the operating environment shall be visually inspected for operability every 31 days.
- C. The initial inspection shall be performed within 6 months from the date of issuance of these specifications. For the purpose of entering the schedule into specification 4.17-A, it shall be assumed that the facility had been on a 6 month inspection schedule.
- D. Once each refueling cycle, a representative sample of 10 hydraulic snubbers or approximately 10% of the hydraulic snubbers, whichever is less, shall be functionally tested for operability including verification of proper piston movement, lock-up and bleed.

*The current inspection interval for Unit 2 may be extended to September 10, 1981.

Amendment No. •72



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 72 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NO. 2

DOCKET NO. 50-281

Introduction

By letter dated July 7, 1981, Virginia Electric and Power Company (the licensee) requested an amendment to the Surry Power Station, Unit No. 2, which would change the Technical Specifications to allow a 60 day extension of the interval for inspecting 158 inaccessible hydraulic snubbers. The extension would be a one-time extension and would expire on September 10, 1981.

This extension will allow continued operation of Surry, Unit No. 2, until Surry, Unit No. 1, returns to power operations following an extended Steam Generator Replacement Outage and until North Anna, Unit No. 2, returns to power operations following the recent failure of a main transformer. This extension will maximize the use of the licensee's remaining nuclear units and minimize the cost of replacement power, and the ultimate cost to the consumer, during an anticipated peak load period.

Discussion and Evaluation

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The Technical Specifications require periodic inspections of snubbers on a schedule ranging from 31 days $\pm 25\%$ if more than eight snubbers are found to be inoperable to 18 months $\pm 25\%$ if no snubbers are found to be inoperable.

The present cycle of operations follows an extended Steam Generator Replacement and Refueling Outage. During this extended outage all hydraulic snubbers were removed, rebuilt or replaced as necessary and functionally tested satisfactorily prior to reinstallation. The first visual inspection conducted in April 1981 following the extended outage revealed seven inoperable snubbers. All of the snubbers were inoperable due to the leakage of fittings either at the valve block or the reservoir. In addition, two of the seven had loose baseplates which were replaced with redesigned baseplates. All inoperable snubbers were replaced and all leaking fittings were replaced or repaired. As a result of these inspection findings the present visual inspection interval is 62 days + 25%. Of the total of two hundred and fifty-two hydraulic snubbers, all seventyseven accessible snubbers and seventeen normally inaccessible snubbers located inside the containment will be visually inspected within the required interval. The seventeen hydraulic snubbers to be inspected inside the containment include six of the seven hydraulic snubbers found inoperable due to fitting leakage during the last visual inspection. The seventh snubber will not be inspected due to hazardous inspection conditions.

We have reviewed the licensee's request and find it to be reasonable in view of the stated need for power during this peak load period and the following reasons: (1) all snubbers were inspected and maintained during the steam generator repair outage and only seven snubbers out of 252 were found to be inoperable during the visual inspection in April 1981 and were inoperable because of leaking fittings. These leaking fittings could have been the result of improper reinstallation of snubbers during the steam cenerator outage, and would probably not have affected the ability of the snubber to perform, (2) all accessible snubbers will be inspected during the required interval, and (3) during a 60 day period the probability of a seismic event or LOCA is very low.

Based on the above reasons, we conclude that it is reasonable to extend the current inspection interval by 60 days which in this case would be until September 10, 1981.

Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Cate: July 9, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-281

VIRGINIA ELECTRIC AND POWER COMPANY

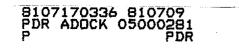
NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 72 to Facility Operating License No. DPR-37 issued to Virginia Electric and Power Company (the licensee), which revised Technical Specifications for operations for the Surry Power Station, Unit No. 2. (the facility) located in Surry County, Virginia. The amendment is effective as of the date of issuance.

The amendment provides a one time 60 day extension for the visual inspection surveillance requirement for inaccessible snubbers.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to



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10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated July 7, 1981, (2) Amendment No. 72 to License No. DPR-37, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C., and at the Swem Library, College of William and Mary, Williamsburg, Virginia 23185. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C., 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of July, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION Chief Varga. Operating Reactors Branch #1 Division of Licensing