

APR 27 1982

Docket Nos. 50-280
50-281

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Mr. R. H. Leasburg
 Vice President - Nuclear Operations
 Virginia Electric and Power Company
 Post Office Box 26666
 Richmond, Virginia 23261

*Posted
 Amdt 78
 to DPR-37*

Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment No. 77 to Facility Operating License No. DPR-32 and Amendment No. 78 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated December 28, 1979 and February 26, 1980.

These amendments revise the Technical Specifications related to the auxiliary feedwater systems. One change adds the requirement that at least one of the three auxiliary pumps be operable when the temperature and pressure of the opposite unit are greater than 350°F and 450 psig, respectively. One change adds an action statement which requires at least three auxiliary pumps be restored to operable status within 72 hours or be in hot shutdown within the next 12 hours. Other changes include a requirement for a source of water for the one auxiliary feedwater pump of the opposite unit, flow tests for all of the auxiliary feedwater pumps, and valve exercise tests for the pump discharge valves.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Joseph D. Neighbors, Project Manager
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. 77 to DPR-32
2. Amendment No. 78 to DPR-37
3. Safety Evaluation
4. Notice of Issuance

cc w/encls:

See next page

*See attached for concurrences

ICE	DL:ORB#1*	DL:ORB#1	DL:ORB#1*	DL:OR *	OELD*		
SURNAME	CParrish	DNeighbors	SAVarga	TMNovak			
DATE	4/ /82	4/ /82	4/ /82	4/ /82	4/ /82		

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated December 28, 1979 and February 26, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

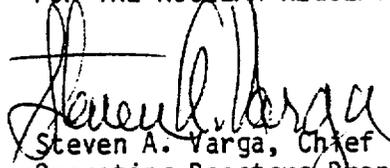
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 77, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 27, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Virginia Electric and Power Company (the licensee) dated December 29, 1979 and February 26, 1980 comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

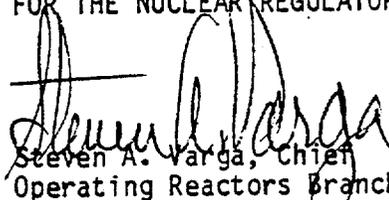
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 78, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 27, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

3.6-1
3.6-2
3.6-3
3.6-4
3.6-5

4.8-1
4.8-2

Insert Pages

3.6-1
3.6-2
3.6-3
3.6-4
3.6-5
3.6-6
4.8-1
4.8-2
4.8-3

3.6 TURBINE CYCLE

Applicability

Applies to the operating status of the Main Steam and Auxiliary Feed Systems.

Objective

To define the conditions required in the Main Steam System and Auxiliary Feed System for protection of the steam generator and to assure the capability to remove residual heat from the core during a loss of station power.

Specification

- A. A unit's Reactor Coolant System temperature or pressure shall not exceed 350°F or 450 psig, respectively, or the reactor shall not be critical unless the five main steam line code safety valves associated with each steam generator in unisolated reactor coolant loops are operable.
- B. To assure residual heat removal capabilities, the following conditions shall be met prior to the commencement of any unit operation that would establish reactor coolant system conditions of 350°F and 450 psig which would preclude operation of the Residual Heat Removal System.

1. Two motor driven auxiliary feedwater pumps shall be operable and one of three auxiliary feedwater pumps for the opposite unit shall be operable.
 2. A minimum of 96,000 gal of water shall be available in the tornado missile protected condensate storage tank to supply emergency water to the auxiliary feedwater pump suction. A minimum of 60,000 gal of water shall be available in the tornado protected condensate storage tank of the opposite unit to supply emergency water to the auxiliary feedwater pump suction of that unit.
 3. All main steam line code safety valves, associated with steam generators in unisolated reactor coolant loops, shall be operable.
- C. Prior to reactor power exceeding 10%, the steam driven auxiliary feedwater pump shall be operable.
- D. System piping, valves, and control board indication required for the operation of the components enumerated in Specification B. 1, 2, 3, and C shall be operable.
- E. The iodine - 131 activity in the secondary side of any steam generator, in an unisolated reactor coolant loop, shall not exceed 9 curies. Also the specific activity of the secondary coolant system shall be ≤ 0.10 $\mu\text{Ci}/\text{cc}$ DOSE EQUIVALENT I-131. If the specific activity of the secondary coolant system exceeds $0.10 \mu\text{Ci}/\text{cc}$ DOSE EQUIVALENT I-131, the reactor shall be shut down and cooled to 500°F or less within 6 hours after detection and in the Cold Shutdown Condition within the following 30 hours.

- F. With one auxiliary feedwater pump inoperable, restore at least three auxiliary feedwater pumps (two motor driven feedwater pumps and one steam driven feedwater pump) to operable status within 72 hours or be in hot shutdown within the following 12 hours.
- G. The requirements of Specification B2 above may be modified to allow utilization of protected condensate storage tank water with the auxiliary steam generator feed pumps provided the water level is maintained above 60,000 gallons, sufficient replenishment water is available in the 300,000 gallon condensate storage tank, and replenishment of the protected condensate storage tank is commenced within two hours after the cessation of protected condensate storage tank water consumption.

Basis

A reactor which has been shutdown from power requires removal of core residual heat. While reactor coolant temperature or pressure is greater than 350°F or 450 psig, respectively, residual heat removal requirements are normally satisfied by steam bypass to the condenser. If the condenser is unavailable, steam can be released to the atmosphere through the safety valves, power operated relief valves, or the 4 inch decay heat release line.

The capability to supply feedwater to the generators is normally provided by the operation of the Condensate and Feedwater Systems. In the event of complete loss of electrical power to the station, residual heat removal would continue to be assured by the availability of either the steam driven auxiliary feedwater pump or one of the motor driven auxiliary feedwater pumps and the

110,000 gallon condensate storage tank. In the event of a fire which would render the auxiliary feedwater pumps inoperable, residual heat removal would continue to be assured by the availability of either the steam driven auxiliary feedwater pump or one of the motor driven auxiliary feedwater pumps from the opposite unit.

A minimum of 92,000 gallons of water in the 110,000 gallon condensate tank is sufficient for 8 hours of residual heat removal following a reactor trip and loss of all off-site electrical power. If the protected condensate storage tank level is reduced to 60,000 gallons, the immediately available replenishment water in the 300,000 gallon condensate tank can be gravity-feed to the protected tank if required for residual heat removal. An alternate supply of feedwater to the auxiliary feedwater pump suction is also available from the Fire Protection System Main in the auxiliary feedwater pump cubicle.

The five main steam code safety valves associated with each steam generator have a total combined capacity of 3,725,575 pounds per hour at their individual set pressure; the total combined capacity of all fifteen main steam code safety valves is 11,176,725 pounds per hour. The ultimate power rating steam flow is 11,167,923 pounds per hour. The combined capacity of the safety valves required by Specification 3.6 always exceeds the total steam flow corresponding to the maximum steady-state power than can be obtained during one, two, or three reactor coolant loop operation.

The availability of the auxiliary feedwater pumps, the protected condensate storage tank, and the main steam line safety valves adequately assures that sufficient residual heat removal capability will be available when required.

The limit on steam generator secondary side iodine - 131 activity is based on limiting inhalation thyroid dose at the site boundary of 1.5 rem after a postulated accident that would result in the release of the entire contents of a unit's steam generators to the atmosphere. In this accident, with the halogen inventories in the steam generator being at equilibrium values, I-131 would contribute 75 percent of the resultant thyroid dose at the site boundary; the remaining 25 percent of the dose is from other isotopes of iodine. In the analysis, one-tenth of the contained iodine is assumed to reach the site boundary, making allowance for plate out and retention in water droplets.

The inhalation thyroid dose at the site boundary is given by:

$$\text{Dose (Rem)} = \frac{(C) (X/Q) (D_{\infty}/A_c) (B.R.)}{(.75) (P.F.)}$$

where: C = steam generator I-131 activity (curies)

$$X/Q = 8.14 \times 10^{-4} \text{ sec/m}^3$$

$$D_{\infty}/A_c = 1.48 \times 10^6 \text{ rem/Ci for I-131}$$

$$B.R. = \text{breathing rate, } 3.47 \times 10^{-4} \text{ m}^3/\text{sec.}$$

from TID 14844

P.F. = plating factor, 10

Assuming the postulated accident, the resultant thyroid dose is 1.5 rem.

The steam generator's specific iodine - 131 activity limit is calculated by dividing the total activity limit of 9 curies by the water volume of a steam generator. A full power, with a steam generator water volume of 47.6 M³, the specific iodine - 131 limit would be .18 $\mu\text{Ci/cc}$; at zero power, with a steam generator water volume of 101 M³, the specific iodine - 131 limit would be .089 $\mu\text{Ci/cc}$.

The limitations on secondary system specific activity ensure that the resultant off-site radiation dose will be limited to a small fraction of 10 CFR Part 100 limits in the event of a steam line rupture.

References

FSAR Section 4	Reactor Coolant System
FSAR Section 9.3	Residual Heat Removal System
FSAR Section 10.3.1	Main Steam System
FSAR Section 10.3.2	Auxiliary Steam System
FSAR Section 10.3.5	Auxiliary Feedwater Pumps
FSAR Section 10.3.8	Vent and Drain Systems
FSAR Section 14.3.2.5	Environmental Effects of a Steam Line Break.

4.8 AUXILIARY FEEDWATER SYSTEM

Applicability

Applies to periodic testing requirements of the Auxiliary Feedwater System.

Objective

To verify the operability of the auxiliary steam generator feedwater pumps and their ability to respond properly when required.

Specification

A. Tests and Frequency

1. Each motor driven auxiliary steam generator feedwater pump shall be flow tested for at least 15 minutes on a monthly basis to demonstrate its operability.
2. The turbine driven auxiliary steam generator feedwater pump shall be flow tested for at least 15 minutes on a monthly basis to demonstrate its operability.
3. The auxiliary steam generator feedwater pump discharge valves shall be exercised on a monthly basis.

- 4a. Within 72 hours prior to temperature and pressure exceeding 350°F and 450 psig respectively, the motor driven auxiliary feedwater pumps shall be flow tested from the 110,000 gallon above ground condensate storage tank to the steam generators.
- 4b. Within 72 hours after achieving reactor criticality, the steam turbine driven auxiliary feedwater pump shall be flow tested from the 110,000 gallon above ground condensate storage tank to the steam generators.
5. During periods of extended reactor shutdown, the testing requirements of Specification A. 1, 2, and 3 may be modified as follows:
 - a. Only one of the three auxiliary steam generator feedwater pumps shall be flow tested for at least 15 minutes on a monthly basis to demonstrate its operability provided the required components are tested prior to startup.
 - b. The auxiliary steam generator feedwater pump discharge valves of the pump tested in 5a shall be exercised on a monthly basis provided all the discharge valves are tested prior to startup.

B. Acceptance Criteria

These tests, except the system flow test, shall be considered satisfactory if control board indication and subsequent visual observation of the equipment demonstrate that all components have operated and sequenced properly.

The system flow test shall be considered satisfactory if the control board indication demonstrates that flow paths exist to each steam generator.

Basis

On a monthly basis the auxiliary steam generator feedwater pumps will be tested to demonstrate their operability by recirculation to the 110,000 Gallon Condensate Storage Tank.

The capacity of any one of the three feedwater pumps in conjunction with the water inventory of the steam generators is capable of maintaining the plant in a safe condition and sufficient to cool the unit down.

Proper functioning of the steam turbine admission valve and the ability of the feedwater pumps to start will demonstrate the integrity of the system. Verification of correct operation can be made both from instrumentation within the Main Control Room and direct visual observation of the pumps.

References

FSAR Section 10.3.1 Main Steam System

FSAR Section 10.3.2 Auxiliary Steam System



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 78 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

Introduction

By letters dated December 28, 1979, and February 26, 1980, the Virginia Electric and Power Company (the licensee) requested amendments to License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2. The changes would revise the Technical Specifications related to the Auxiliary Feedwater System.

The two applications were supplemented by the following letters:

November 19, 1979
January 10, 1980
February 1, 1980
March 21, 1980
May 19, 1980
September 30, 1980
October 6, 1980
October 17, 1980
December 12, 1980
December 23, 1980
December 26, 1980
April 20, 1981
September 14, 1981
November 9, 1981

Discussion and Evaluation

On November 17, 1980, the NRC issued a Safety Evaluation Report of the Auxiliary Feedwater (AFW) System Reliability for Surry 1 and 2. This evaluation provided the background for the review and closed several aspects of the review. However, several aspects of the review were left open for further information or evaluation. The open items were:

1. Recommendation GS-1: Technical Specifications were required to limit the time that one AFW system pump and its associated flow train and essential instrumentation can be inoperable. The Technical Specifications proposed by the licensee on December 28, 1979, provided that three pumps be restored to operable status within 72 hours or be in hot shutdown within the next 12 hours. These requirements are consistent with the Standard Technical Specifications and we find them acceptable.
2. Recommendation GS-6: Technical Specifications were required which would verify by a flow test the normal flow path for the primary AFW system water source to the steam generators. This change to the Technical Specifications has been proposed and based on our review, we find this change acceptable.
3. Additional Short Term Recommendation 3: The licensee should implement the following requirements as specified by Item 2.1.7.b on page A-32 of NUREG-0578:

"Safety-grade indication of auxiliary feedwater flow to each steam generator shall be provided in the control room."

This item was resolved by NRC letter dated August 7, 1981.

4. Recommendation GL-3: At least one AFW system pump and its associated flow path and essential instrumentation should automatically initiate AFW system flow and be capable of being operated independently of any alternating current power source for at least two hours. Conversion of direct current power to alternating current is acceptable.

By letter dated December 9, 1981, the licensee stated that at least one AFW system pump and flow path is automatically initiated upon a loss of all AC power. Remote on-off operation from the control room is provided to control flow. We consider the licensee's response acceptable.

5. Recommendation GL-5: The licensee should upgrade the AFW system automatic initiation signals and circuits to meet safety-grade requirements.

This item was resolved by NRC letter dated August 7, 1981.

6. NRC Recommendation - The AFWS flow control valves for both the motor and turbine pump trains are AC powered, normally open, fail-as-is motor operated valves which are located inside containment. Also, manual normally open valves are located inside containment. The AFWS design should be reevaluated, including the possibility of relocating the valves outside containment, assuming an accident inside containment which necessitates AFWS operation and which creates a containment environment (humidity, radiation) that precludes access to the valves. The reevaluation should consider the following:

- a. A possible common mode failure (environmentally induced) causing spurious closure or failure of the MOVs in a throttled position.
- b. An AFWS line break downstream of the MOVs and failure of the MOVs to operate.

By letter dated December 9, 1981, the licensee stated that the AFW system flow control valves for both the motor and turbine driven pumps will remain inside containment. The valves and operators currently installed do not have documentation to substantiate their environmental qualification. However, the licensee has committed to obtain the required documentation. Valve operators which cannot be qualified will be replaced by qualified valves.

We consider the licensee's response acceptable.

7. Basis for AFW System Flow Requirements

The licensee should provide a commitment date for responding to the information requested in Enclosure 2 of the NRC letter dated September 25, 1979.

By letter dated December 26, 1980, the licensee verified that the minimum flow rate requirement of 500 gpm by the auxiliary feedwater system could be maintained under the following transient conditions:

1. Loss of normal feedwater transient
 - a. With offsite power available
 - b. Concurrent with loss of offsite power
2. Rupture of a main steam line
3. Loss of all AC power
4. Loss of Coolant Accident (LOCA)
5. Cooldown

During our review, it was noted that the licensee's analysis did not evaluate the case for a feedwater line break - an event that is often considered limiting. The licensee was requested to verify that the required AFW flow could be provided for this event. Their response dated November 9, 1981 indicated that the flow requirement for a feedwater line break is much less than that required for the design basis loss of normal feedwater transient (i.e., 350 gpm vs 500 gpm).

Based on our review of the licensee's submittals, we conclude that sufficient AFW flow can be delivered to two steam generators by the AFW system to meet the minimum heat removal requirements following any design basis transient or accident assuming the worst case single active failure.

The NRC Safety Evaluation Report dated September 19, 1979, evaluated the fire protection program and concluded that the existing cross connect between the two Units' auxiliary feedwater systems was an acceptable means for providing auxiliary feedwater from one unit to the other if the AFW pumps in one unit were destroyed by fire. The Technical Specifications proposed by letter dated February 26, 1980, provided this requirement. We have reviewed the proposed Technical Specifications and find them acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 27, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-280 AND 50-281VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 77 to Facility Operating License No. DPR-32 and Amendment No. 78 to Facility Operating License No. DPR-37 issued to Virginia Electric and Power Company (the licensee), which revised Technical Specifications for operation of the Surry Power Station, Unit Nos. 1 and 2, respectively, (the facilities), located in Surry County, Virginia. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications related to the auxiliary feedwater systems. One change adds the requirement that at least one of the three auxiliary pumps be operable when the temperature and pressure of the opposite unit are greater than 350°F and 450 psig, respectively. The other change adds an action statement which requires at least three auxiliary pumps be restored to operable status within 72 hours or be in hot shutdown within the next 12 hours. Other changes include a requirement for a source of water for the one auxiliary feedwater pump of the opposite unit, flow tests for all of the auxiliary feedwater pumps, and valve exercise tests for the pump discharge valves.

The application for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

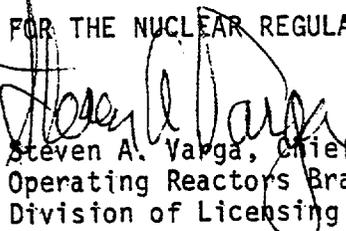
- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated December 28, 1979 and February 26, 1980, (2) Amendment Nos. 77 and 78 to License Nos. DPR-32 and DPR-37, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Swem Library, College of William and Mary, Williamsburg, Virginia 23185. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of April, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

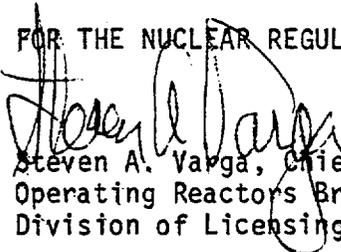
- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §1.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated December 28, 1979 and February 26, 1980, (2) Amendment Nos. 77 and 78 to License Nos. DPR-32 and DPR-37, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C. and at the Swem Library, College of William and Mary, Williamsburg, Virginia 23185. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 27th day of April, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing