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Docket Nos. 50-280 and (50-281) DISTRIBUTION NRC PDR Docket File L PDR ORB #1 Rda. D Eisenhut OELD ACRS (10) C Miles L Harmon TBarnhardt (8) E Jordan J Taylor D Brinkman R Diggs W Jones D Neighbors (2)H Denton C Parrish Gray Files + 4

Mr. W. L. Stewart Vice President - Nuclear Operations Virginia Electric and Power Company Post Office Box 26666 Richmond, Virginia 23261

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No. 94 to Facility Operating License No. DPR-32 and Amendment No. 93 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated August 25, 1983.

These amendments revise the Technical Specifications to remove eight snubbers from auxiliary feedwater system and emergency diesel generator exhaust piping from Table 4.17-1. In additon, Specification 6.4.B.1.b is revised to add the Supervisor of Health Physics as an administrative controller of keys to locked barricades at radiation areas.

A copy of the Safety Evaluation is enclosed. The Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

/s/ Joseph D. Neighbors

Joseph D. Neighbors, Project Manager Operating Reactors Branch No. 1 Division of Licensing

Enclosures:

- Amendment No. 94 to DPR-32
- 2. Amendment No. 93 to DPR-37
- 3. Safety Evaluation
- 4. Notice of Issuance

cc w/enclosures: See next page

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Mr. W. L. Stewart Virginia Electric and Power Company

Surry Power Station Units 1 and 2

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Mr. Sherlock Holmes, Chairman Board of Supervisors of Surry County Surry County Courthouse Surry, Virginia 23683

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 94 License No. DPR-32

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated August 25, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:
 - B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 94 , are hereby incorporated in the license. The Licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief Operating Reactors Branch No. 1 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: JAN 3 1 1984



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 93 License No. DPR-37

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated August 25, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I:
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:
 - B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 93 , are hereby incorporated in the license. The Licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Operating Reactors Branch No. 1 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance:

JAN 3 1 1984

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages	<u>Insert Pag</u>			
4.17-17 4.17-29	4.17-17			
4.17-29	4.17-29 4.17-41			
6.4-2	6.4-2			

TABLE 4.17-1 UNIT NO. 1 SUPPRESSOR DATA

	SYSTEM	DESIGNATION	SIZE	LOCATION	ACCESSIBILITY	RADIATION CATEGORY	REMOVAL CATEGORY	
Aux.	Feed System	1-WAPD-HSS-140	1 1/2"	Containment Operating Leve	1 I	н	R	
Aux.	Feed System	1-WAPD-HSS-141A	1 1/2"	Containment Operating Leve	1 I	н	R	
Aux.	Feed System	1-WAPD-HSS-141B	1 1/2"	Containment Operating Leve	1 I	H	R	
Aux.	Feed System	1-WAPD-HSS-142	1 1/2"	Containment Operating Leve	1 I	Н	R	
Aux.	Feed System	1-WAPD-HSS-143A	1 1/2"	Containment Operating Leve	1 I	H	R	
Aux.	Feed System	1-WAPD-HSS-143B	1 1/2"	Containment Operating Leve	1 I	H	R	
Aux.	Feed System	1-WAPD-HSS-146	1 1/2"	Safeguards	A	N	R	
Aux.	Feed System	1-WAPD-HSS-148	1 1/2"	Safeguards	A	N	R	

TABLE 4.17-1
UNIT NO. 1 SUPPRESSOR DATA

	SYSTEM	DESIGNATION	SIZE	LOCATION	ACCESSIBILITY	RADIATION JATEGORY	REMOVAL CATEGORY
Make	e-up System	1-WCMU-HSS-100	1 1/2"	Safeguards	A	N	R
Cont	ainment Spray System	1-CS-HSS-01	1 1/2"	Safeguards Basement	A	N	R
Serv	vice Water System	1-SW-HSS-2	PSA-1/4	Machinery Room #3	A	N	D
S/G	Blowdown	1-WGCB-HSS-10	1 1/2"	Aux. Basement	A	N	R
S/G	Blowdown	1-WGCB-MSS-23	PSA-3	Aux. Basement	· A	N	D
S/G	Blowdown	1-WGCB-MSS-11	PSA-1/2	A Loop room-E1. 13/6"	I	H	ď
S/G	Blowdown	1-WGCB-MSS-12	PSA-1/2	A loop room-E1. 7'8"	I	H	D
S/G	Blowdown	1-WGCB-MSS-13	PSA-1/2	A loop room-E1. 8'3"	ī	$\mathbf{H}_{i,j}^{(i)}$	D
S/G	Blowdown	1-WGCB-MSS-14A	PSA-1/2	A loop room-E1. 6'	I	Н	D
S/G	Blowdown	1-WGCB-MSS-14B	PSA-1/2	A loop room-E1. 6'	I	H	D
S/G	Blowdown	1-WGCB-MSS-15A	PSA-1/2	A S/G Cubicle	I	Н	D
s/G	Blowdown	1-WGCB-MSS-15B	PSA-1/2	A S/G Cubicle	I	Н	D
As/G	Blowdown	1-WGCB-MSS-16	PSA-1/2	A S/G Cubicle	I	H	D
nd s/G	Blowdown Blowdown	1-WGCB-MSS-17A	PSA-1/2	B loop room-E1. 13	I	н	D
nts/G	Blowdown	1-WGCB-MSS-17B	PSA-1/2	B 100p room-E1. 13'	I	Н	D
်s/G	Blowdown	1-WGCB-MSS-18A	PSA-1/2	B loop room-E1. 7	I	Н	D
	Blowdown	1-WGCB-MSS-18B	PSA-1/2	B loop room-E1. 7	I	H	D

TABLE 4.17-2
UNIT NO. 2 SUPPRESSOR DATA

SYSTEM	DESIGNATION	SIZE	LOCATION	ACCESSIBILITY	RADIATION CATEGORY	REMOVAL CATEGORY	
Charging System	2-CH-HSS-303	1 1/2"	Aux. Building	A	N	R	
Charging System	2-CH-HSS-304	1 1/2"	Aux. Building	A .	N	R	
Charging System	2-CH-HSS-305	1 1/2"	Containment Basement	I	Н	R	
Charging System	2-CH-HSS-306A	1 1/2"	Containment Basement	I	H	R	
Charging System	2-CH-HSS-306B	1 1/2"	Containment Basement	I	н	R	
Charging System	2-CH-HSS-307	1 1/2"	Containment Basement	I	H	R	
Charging System	2-CH-MSS-1	PSA-3	Containment Basement	I	Н	D	
Charging System	2-CH-MSS-2	PSA-3	Containment Basement	I	H	D	
Service Water System	2-SW-MSS-1	PSA-3	Containment Basement	I	Н	D	

- 1. The intent of 10 CFR 20.203(c)(2)(iii) shall be implemented by satisfying the following conditions:
 - a. The entrance to each radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted.
 - b. The entrance to each radiation area in which the intensity of radiation is equal to or greater than 1000 mrem/hr shall be provided with locked barricades to prevent unauthorized entry into these areas. Keys to these locked barricades shall be maintained under the administrative control of the Shift Supervisor on duty and/or Supervisor Health Physics.
 - c. All such accessible high radiation areas shall be surveyed by Health Physics personnel on a routine schedule, as determined by the Supervisor-Health Physics, to assure a safe and practical program.
 - d. Any individual entering a high radiation area shall have completed the indoctrination course designed to explain the hazards and safety requirements involved, or shall be escorted at all times by a person who has completed the course.
 - e. Any individual or group of individuals permitted to enter a high radiation area per 1.d. above, shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. DPR-32

AND AMENDMENT NO. 93 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

Introduction

In a letter dated August 25, 1983, the Virginia Electric and Power Company (the licensee) submitted an application for amendments to the operating licenses for the Surry Power Station, Unit Nos. 1 and 2. This evaluation is related to snubbers (shock suppressors) and the Supervisor of Health Physics.

Discussion and Evaluation

The proposed change will remove eight snubbers from the Surry Technical Specifications. Two of the snubbers to be removed, I-EE-HSS-01, and I-EE-HSS-03, are located on the emergency diesel generator exhaust piping in Unit No. 1. Three of the snubbers, 2-EE-HSS-01, 2-EE-HSS-02, and 2-EE-HSS-03, are located on the emergency diesel generator exhaust piping in Unit No. 2. The other three snubbers to be removed, 1-WAPD-HSS-145, 1-WAPD-HSS-147, and 1-WAPD-HSS-149, are in the auxiliary feed system in Unit No. 1.

The three emergency diesel generators and associated equipment are located in Category I tornado protected structures for both Unit Nos. 1 and 2. The diesel exhaust piping is routed up through the service building roof and is surrounded by a tornado missile shield. A portion of the exhaust piping extended beyond the structure and with the missile shield open on one side, the piping was not adequately protected for all postulated tornado generated missiles. The portion of the exhaust piping exposed to a potential tornado generated missile and the associated snubbers were removed. Removal of the exposed exhaust piping eliminates the possibility of loss of a diesel generator due to a tornado generated missile.

The snubbers in the Auxiliary Feed System were removed after a seismic and thermal reanalysis was performed for the Main Feed Line. The analysis revealed that the three snubbers in the Main Steam Valve House could be replaced with struts. Since there is no significant thermal movement at the original snubber locations, this modification increases the reliability of the Main Feed Lines.

The modifications will increase the reliability for the Main Feed Lines and upgrade the tornado missile protection for the emergency diesel generators. Therefore, we find the above proposed changes to be acceptable.

The change related to the Supervisor of Health Physics adds this position to Technical Specification 6.4.B.1.b. as one who has administrative controls of keys to barriers at radiation areas. This is an administrative change and is the same as exists on some other facilities (e.g., North Anna). We find this change to be acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to $10 \ \text{CFR } \S 51.5(d)(4)$, that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: **JAN 3** 1 1984

Principal Contributors:

H. Shaw

D. Neighbors