



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DO NOT REMOVE

November 18, 1982

Docket Nos. 50-280
and ~~50-281~~

Posted
Amend. 82
to DPR-37

Mr. R. H. Leasburg
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Leasburg:

The Commission has issued the enclosed Amendment No. 81 to Facility Operating License No. DPR-32 and Amendment No. 82 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated October 6, 1982.

These amendments revise the Technical Specifications to modify reporting and notification requirements related to the instantaneous release rates of gaseous wastes.

Copies of the Environmental Impact Appraisal and the Notice of Issuance/Negative Declaration are also enclosed.

Sincerely,

A handwritten signature in cursive script that reads "Joseph D. Neighbors".

Joseph D. Neighbors, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 81 to DPR-32
2. Amendment No. 82 to DPR-37
3. Environmental Impact Appraisal
4. Notice of Issuance/Negative Declaration

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated October 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

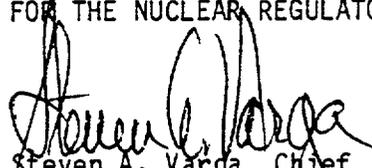
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 82, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3.11-3	3.11-3
3.11-5	3.11-5

A-1 Above are met.

10. The requirements of Specification 3.0.1 are not applicable.

B. Gaseous Wastes

1a. The controlled release rates of gaseous wastes, excluding halogen and airborne particulates originating from station operation shall be limited as follows:

$$\sum \frac{Q_i}{(\text{MPC})_i} \leq 2.0 \times 10^5 \frac{\text{m}^3}{\text{sec}}$$

where Q_i is the controlled release rate (curies per second) of any radioisotope i and $(\text{MPC})_i$, in units of microcuries per cubic centimeter is defined in column 1, Table II of Appendix B to 10 CFR 20.

- b. With the release rate of gaseous wastes exceeding the above limit, without delay restore the release rate to within the limit.
 - c. The provisions of specification 6.6.2.a are not applicable.
2. The release rates of activity shall not exceed 16 percent of those specified in paragraph B.1a above when averaged over any calendar quarter or 10 percent of those specified in paragraph B.1a above when averaged over any 12 consecutive months.
- 3a. The release rate limit of all radioiodines and radioactive materials in particulate form with half-lives greater than eight days released from the site to the environs as part of the gaseous wastes shall be such that

$$3 \times 10^5 Q \leq 1$$

where

Q = the measured release rate of the radioiodines and radioactive materials in particulate form with half-lives greater than eight days (Ci/sec).

- b. The average release rate per site of all radioiodines and radioactive materials in particulate form with half-lives greater than eight days during any calendar quarter shall be such that

$$13 [3 \times 10^5 Q] \leq 1$$

fuel in the containment.

11. The requirements of Specification 3.0.1 are not applicable.

Basis

The releases of radioactive materials will be kept as low as practicable as required by 10 CFR 50 and will not exceed the concentration limits specified in 10 CFR 20. At the same time, the licensee is permitted the flexibility of operation, compatible with considerations of health and safety, to assure that the public is provided a dependable source of power under unusual operating conditions which may temporarily result in releases in excess of four percent of the concentration limits specified in 10 CFR 20. However, all releases must be kept within the concentration limits specified in 10 CFR 20. It is expected that using this operational flexibility under unusual operating conditions, the licensee shall exert every effort to keep levels of radioactive materials released from the plant as low as practicable and that annual releases will not exceed a small fraction of the annual average concentration limits specified in 10 CFR 20.

The limiting conditions for operation contained in specification A-3 above, which relates to the total number of curies which may be released in liquid effluents in any year, is based on the expected performance of the Surry Power Station assuming both units are operating with 0.25 percent leaking fuel and each unit is experiencing a 20 gallon per day primary to secondary system leak rate.

The formulas and limits prescribed in specifications 3.11.B.1 and B.2 take atmospheric dilution into account and assures that at the point of maximum ground concentration at the site boundary, the requirements of 10 CFR 20 will not be exceeded. The limit is based on the highest annual average value of X/Q which will occur at the



UNITED STATES
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WASHINGTON, D. C. 20555

ENVIRONMENTAL IMPACT APPRAISAL

BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-32

AND AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

Introduction

By letter dated October 6, 1982, the Virginia Electric and Power Company (the licensee) requested amendments to the Surry Power Station, Unit Nos. 1 and 2 licenses which would revise the Technical Specifications. This change would modify the reporting and notification requirements related to the instantaneous release rates of gaseous wastes.

Discussion and Evaluation

The proposed revision to the Surry Technical Specifications, Section 3.11.B, consists of the addition of subparagraphs b and c as shown here:

- 1a. The controlled release rates of gaseous wastes, excluding halogen and airborne particulates originating from station operation shall be limited as follows:

$$\sum \frac{Q_i}{(\text{MPC})_i} \leq 2.0 \times 10^5 \frac{\text{m}^3}{\text{sec}}$$

where Q_i is the controlled release rate (curies per second) of any radioisotope i and $(\text{MPC})_i$, in units of microcuries per cubic centimeter is defined in column 1, Table II of Appendix B to 10 CFR 20.

- b. With the release rate of gaseous wastes exceeding the above limit, without delay restore the release rate to within the limit.
- c. The provisions of specification 6.6.2.a are not applicable.

Section 3.11.B.1b is designed to ensure that the annual MPC limits will not be exceeded, by requiring the licensee to take action without delay should the instantaneous release rate, corresponding to 10 CFR 20 annual limits, be exceeded at any time. Section 3.11.B.1c removes the reporting requirement for violations of Section 3.11.B.1b. Section 3.11.B.2 reads as follows:

- 2. The release rates of activity shall not exceed 16 percent of those specified in paragraph B.1a above when averaged over any calendar quarter or 10 percent of those specified in paragraph B.1a above when averaged over any 12 consecutive months.

If the limits specified in this paragraph are exceeded, the licensee is required to report the release event to the NRC according to the provisions of Section 6.6.2a of the Technical Specifications. This requirement helps to ensure that releases to unrestricted areas will be kept as low as reasonably achievable.

The existing Technical Specifications, which do not include subparagraphs b and c of Section 3.11.B.1, require a report to be generated whenever the limit of Section 3.11.B.1a is exceeded in any amount. The proposed revision to the Technical Specifications eliminates this particular reporting requirement, while leaving intact the action requirement of 3.11.B.1a of the action and reporting requirements imposed by paragraph 3.11.B.2. The proposed Technical Specification is consistent with the Standard Radiological Effluent Technical Specifications for Pressurized Water Reactors (Rev. 3, June 1982).

The proposed revision will result in no environmental impact because there is no change in the limit on the release rate of gaseous wastes (3.11.B.1a), and because the licensee is required, as before, to take action without delay to restore the release rate to within the limit (3.11.B.1b) if it is exceeded. Therefore, the proposed Technical Specification will not change the total quantities or types of radioactivity discharged to the environment from the Surry Power Station.

Basis for Negative Declaration

On the basis of the foregoing evaluation, it is concluded that there would be no significant environmental impact attributable to the proposed action. Having made this conclusion, the Commission has further concluded that no environmental impact statement for the proposed action need be prepared and that a negative declaration to this effect is appropriate.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 18, 1982

Principal Contributor:
Richard Struckmeyer

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-280 AND 50-281VIRGINIA ELECTRIC AND POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES
AND
NEGATIVE DECLARATION

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 81 to Facility Operating License No. DPR-32 and Amendment No. 82 to Facility Operating License No. DPR-37 issued to Virginia Electric and Power Company (the licensee), which revised Technical Specifications for operation of the Surry Power Station, Unit Nos. 1 and 2, respectively, (the facilities), located in Surry County, Virginia. The amendments are effective as of the date of issuance.

The amendments revise the Technical Specifications to modify reporting and notification requirements related to the instantaneous release rates of gaseous wastes.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since these amendments do not involve a significant hazards consideration.

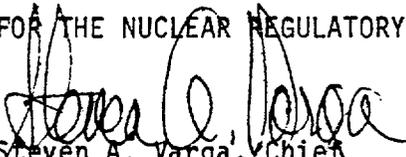
The Commission has prepared an environmental impact appraisal for the revised Technical Specifications and has concluded that an environmental impact statement for this particular action is not warranted because there will be no environmental impact attributable to action other than that which has already been predicted and described in the Commission's Final Environmental Statement for the facility dated May 1972.

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For further details with respect to this action, see (1) the application for amendments dated October 6, 1982, (2) Amendment Nos. 81 and 82 to License Nos. DPR-32 and DPR-37, and (3) the Commission's related Environmental Impact Appraisal. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Swem Library, College of William and Mary, Williamsburg, Virginia 23185. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 18th day of November, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing