Question # 1

Given the following:

- Unit 1 was operating at 100% power when offsite power was lost.
- Control room personnel have just transitioned to 0POP05-EO-ES02, "Natural Circulation Cooldown".

WHICH ONE (1) of the following actions should be taken to establish natural circulation cooldown if a reactor coolant pump cannot be restarted?

- A. Dump steam from all intact steam generators using SG PORVs.
- B. Place steam dump controls in MANUAL and dump steam to the condenser.
- C. Start one reactor cavity and supports vent supply and exhaust fan.
- D. Depressurize the RCS using auxiliary spray.

Tier #

Group #

10 CFR 55.43(b)

Answer:	A	Le	sson Plan L	OT 504.25, p. 8-9	
Question Sourc	æ:	Bank # Modified E New	ank #	X	
Question Cognitive Level:		Memory of Comprehe	Memory or Fundamental Knowledge Comprehension or Analysis		X_
Examination O	utline Cross-refere	ence:			
	Level K/A #		<u>RO</u> W/E 09ł	<1.01	<u>SRO</u>
	Importance	e Rating	3.0		3.4

1

1

1

1

Question # 2

As the Unit 1 operator, you have been instructed to establish Emergency Boration from the RWST per 0POP04-CV-0003, Emergency Boration. What is the MINIMUM flow rate required, as observed on FI-205A, for this task.

Α.	50 gpn	า				
В.	90 gpn	n				
C.	150 gp	m				
D.	190 gp	m				
Answe	r:	D	Lesson	Plan	(As available)	
Refere	nce:	0POP04-CV-0	0003, Em	ergency Bora	tion, P&ID 5R17	9F05007
Questi	on Sour	rce:	Bank # Modified New	l Bank #	X	
Questi	on Cogi	nitive Level:	N C	Memory or Fu	ndamental Know on or Analysis	vledgeX
Examir	nation C	Outline Cross-re	eference:			
		Level K/A #			<u>RO</u> 000024A1.13	<u>SRO</u>
		Import Tier #	ance Rat	ing	3.2 1	3.0 1

1

1

Group #

10 CFR 55.43(b)

Question # 3

Given the following:

- Unit 1 is at 70% power.
- "MAIN COND VACUUM LO" alarm is lit.
- "LP TURB EXH HOOD TEMP HI" alarm is lit.
- Condenser vacuum is 23 inches Hg and DECREASING slowly.

WHICH ONE (1) of the following actions should be taken?

- A. Commence an orderly shutdown of the Main Turbine.
- B. Trip the Reactor, then trip the Main Turbine.
- C. Commence RCS boration, and perform additional actions for a Fast Load Reduction.
- D. Increase turbine load 10% to allow increased exhaust hood cooling.

Answer: B Lesson Plan (As available)

Reference: 0POP04-CR-0001 "Loss of Condenser Vacuum: step 5,7

Question Source:		Bank # Modified Bank # New	X	
Question Cognitive Le	vel:	Memory or Fundar Comprehension or	mental Knowledge ⁻ Analysis	X_
Examination Outline C	cross-r	eference:		
· · · · · · · · · · · · · · · · · · ·	Level K/A # Import Tier # Group 10 CF	ance Rating # R 55.43(b)	<u>RO</u> 000051A2.02 3.9 1 1	<u>SRO</u> 4.1 1 1

Question # 4

The following conditions exist on Unit 1:

- Chemistry reports RCS Iodine-131 activity is 0.11 mCi/gram (Normal RCS Iodine-131 activity is 0.001 mCi/gram)
- In-service Mixed bed demineralizer I-131 decontamination factor is NOT adequate
- In-service Mixed bed demineralizer cesium decontamination factor is adequate
- Cesium values are normal

In addition to maximizing charging and letdown, which ONE of the following corrective actions should be taken in accordance with 0POP04-RC-0001, High Reactor Coolant System Activity?

- A. Increase sampling frequency to every 4 hours until activity returns to normal.
- B. Commence a rapid plant shutdown and reduce Tavg to < 500 °F.
- C. Place the Cation bed demineralizer in service.
- D. Place the alternate Mixed bed demineralizer in service.

Answer: D	Lesson Plan	(As available)	
Reference: 0POP04-R	C-0001, Section 4.1.2		
Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Le	vel: Memory o Comprehe	Memory or Fundamental Knowledge Comprehension or Analysis	
Examination Outline C	ross-reference:		
	Level K/A # mportance Rating Fier # Group #	<u>RO</u> 000076K3.06 3.2 1	<u>SRO</u> 3.8 1
	10 CFR 55.43(b)	I	I

Question # 5

The following conditions exist on Unit 1:

- Reactor power is 80%
- Rod Deviation alarm is lit

Group #

10 CFR 55.43(b)

- Rod Bottom alarm is lit
- Power Range Channel Deviation alarm is lit
- 2 Rod Bottom LEDs are lit on DRPI

Which ONE of the following describes the required operator response to these conditions?

- A. Check Axial Flux Difference and Quadrant Power Tilt Ration.
- B. Trip the reactor and perform 0POP05-EO-EO00, Reactor Trip or Safety Injection.
- C. Restore the rods per 0POP04-RS-0001, Stuck or Dropped Rod
- D. Restore the rods per 0POP04-RS-0001, Stuck or Dropped Rod, then verify operability by performing 0PSP03-RS-0001, Monthly Control Rod Operability.

Answer: B		Lesson Plan	(As available)	
Reference:	0POP04-F	RS-001		
Question Sou	rce:	Bank # Modified Bank # New	X	
Question Cog	nitive Level	: Memory or F Comprehens	undamental Knowledge ion or Analysis	X
Examination (Outline Cros	ss-reference:		
	Level K/A #		<u>RO</u> 000003A2.03	<u>SRO</u>
	Importanc	e Rating	3.6	3.8 1

2

1

Question # 6

WHICH ONE (1) of the following is a potential indication that the PRT has ruptured following a pressurizer PORV failing full OPEN?

- A. PRT temperature is decreasing.
- B. PORV relief line temperature is increasing.
- C. PRT level decreases to its normal value of 70%.
- D. Pressurizer level is decreasing.

Answer: A Lesson Plan (As available)

Reference: LP No. LOT201.04, "Pressurizer, Pressure Relief Tank RCDT"

Question Source: Bank # Modified Bank # New

 Question Cognitive Level:
 Memory or Fundamental Knowledge
 _____X____

 Comprehension or Analysis
 ______X_____

Examination Outline Cross-reference:

Level		<u>R0</u>		<u>SRO</u>
K/A #		000008A1.08		
Importance Rating		3.8		3.8
Tier #		1		1
Group #	2		2	
10 CFR 55.43(b)				

Х

CONFIRM REFERENCES

Question # 7

Given the following conditions:

- Reactor trip with SI occurred at 0100
- Operators were directed by LOCA procedure to enter OPOP05-EO-EC11, Loss of Emergency Coolant Recirculation" at 15:30

- At 1700:

- RCS subcooling based on core exit TCs: 82 degrees F
- Circulation conditions: natural circulation
- RVLIS indication: Plenum level 15%

WHICH ONE (1) of the following is the minimum SI Flow rate that must be maintained (at 17:00 hours) per OPOP05-EO-EC11, "Loss of Emergency Coolant Recirculation"? (Addendum 5 may be used)

A. Approximately 60 gpm.

- B. Approximately 150 gpm.
- C. Approximately 270 gpm.
- D. Approximately 775 gpm.

Answer:	В	Lesson Plan	(As available)

Reference: OPOP05-EO-EC11, "Loss of Emergency Coolant Recirculation"

Question Source:	Bank #	X
	Modified Bank # New	

Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

Х

Examination Outline Cross-reference:

Level	<u>R0</u>	<u>SRO</u>
K/A #	W/E11K1.02	
Importance Rating	3.6	4.1
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 8

WHICH ONE (1) of the following operator actions will result in Pressurized Thermal Shock (PTS) conditions following a Steam Generator Tube Rupture (SGTR) concurrent with a loss of offsite power (LOOP)?

- A. Overcooling the RCS when using the Turbine Bypass valves to cooldown the RCS to establish subcooling.
- B. Allowing the Safety Injection Accumulators to inject while cooling down and depressurizing.
- C. Delaying the termination of SI after the SI termination criteria are met.
- D. Allowing the ruptured SG pressure to INCREASE while cooling down the RCS to establish subcooling.

1

2

1

1

Answer:	С	Lesson Plan	LP No. LOT LOT502.07		
Resource: Question S	Source:	Bank # Modified Bank # New	X		
Question (Cognitive Leve	: Memory or Fu Comprehension	ndamental Knowledge or Analysis	X	_
Examinatio	on Outline Cros	ss-reference:			
	Level		<u>RO</u>		<u>SRO</u>
	Import	ance Rating	3.3		4.2

Tier #

Group #

10 CFR 55.43(b)

Question # 9

When establishing bleed and feed cooling per 0POP05-EO-FRH1, "Response to Loss of Secondary Heat Sink," both PORVs are opened.

WHICH ONE of the following is the reason for manually opening the PORVs rather than allowing then to open and close automatically.

- A. RCP damage may occur due to inadequate seal injection with RCS pressure at the PORV setpoint.
- B. Depressurizing a SG for condensate feed with RCS pressure at the PORV setpoint will exceed U-tube delta-P limits.
- C. A water solid RCS at NOP/NOT has a high potential to challenge the RCS pressure safety limit.
- D. Maintaining both PORVs fully open is necessary to provide adequate core cooling flow.

Answer: D Lesson Plan LOT 504.33.LP, Rev. 4, pg. 11

Resources: 0POP05-EOFRH1, Rev. 3, pg. 9

Question Source:	Bank # Modifie New	ŧ ed Bank #	X	
Question Cognitiv	e Level:	Memory or Fu Comprehensi	indamental Knowledge on or Analysis	X
Examination Outlin	ne Cross-refer	ence:		
	Level K/A # Importance Ra Tier # Group # 10 CFR 55.43	ating (b)	<u>RO</u> 000054K3.05 4.6 1 2	<u>SRO</u> 4.7 1 2

Question # 10

WHICH ONE (1) of the following describes the AUTOMATIC response of the Fuel Handling Building HVAC System upon receipt of an exhaust air HIGH RADIATION alarm on RIT-8035?

- A. All Main Exhaust and Main Exhaust Booster Fans START and the Exhaust System Dampers shift to align two filtering unit trains.
- B. All Main Exhaust and Main Exhaust Booster Fans STOP and the Exhaust System Dampers CLOSE.
- C. All Main Supply Fans STOP and the Exhaust System Dampers shift to align two filtering unit trains.
- D. ALL Main Supply Fans stop and the Exhaust System Dampers CLOSE.

Answer:	A	Lesson Plan	LP No. LOT202.38.L, Objective A LP No. LOT202.38.HO.01, pp. 3	4, p. 3 -9
Question	Source:	Bank # Modified Bank New	X 	
Question	Cognitive Level	: Memoi Compr	ry or Fundamental Knowledge rehension or Analysis	X
Examinati	on Outline Cros	ss-reference:		
	Level		RO	<u>SRO</u>

<u>R0</u>	SRC
000060K2.02	
2.7	3.1
1	1
2	2
	<u>RO</u> 000060K2.02 2.7 1 2

Question # 11

Which ONE of the following includes ONLY events for which area radiation monitors could be useful in identification?

- A. LOCA inside RCB, steam generator tube rupture, fuel handling accident in FHB, high radiation at drumming station.
- B. LOCA inside RCB, gas storage tank rupture or rupture of charcoal beds, fuel handling accident in FHB, RCS leakage at incore instrumentation seal table.
- C. LOCA inside RCB, main steam line break, RCS to CCWS leak, fuel handling accident in FHB.
- D. Steam generator tube rupture, main steam line break, high radiation at drumming station, high radiation in the primary sample room or post accident sample room.

Answer: B	Lesson Plan	LOT202.42.LP, Revision 3, Section	n 3.2
Question Source:	Bank # Modified Bank New	#X	
Question Cognitive Level	: Memor Compr	y or Fundamental Knowledge ehension or Analysis	X

Examination Outline Cross-reference:

Level	<u>R0</u>	<u>SRO</u>
K/A #	000061G.2.4.31	
Importance Rating	3.3	3.4
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 12

Which one of the following will prevent control rod motion in automatic or manual control?

- Failure of the following logic cabinet power supplies: one +100 VDC power supply, +16.5 VDC power supply, and one -16.5 VDC power supply.
- B. Failure of the following power cabinet power supplies: one +24 VDC power supply and one -24 VDC power supply.
- C. Failure of the power cabinet firing circuits for all multiplexing thyristors due to the main 120 VAC supply train lost.
- D. Failure of the logic cabinet slave cycler input due to the slave cycler receiving a "GO" pulse from the Master Cycler before completing the previous step.

Answer: D	Lesso	n Plan LP.N	IO.:LOT201.18.L p.	35-40	
Question Sourc	e: Bank : Modifi New	# ed Bank #	X		
Question Cogn	itive Level:	Memory or Fu Comprehensio	ndamental Knowled on or Analysis	lge _	_X
Examination O	utline Cross-refer	ence:			
	Level K/A # Importance R Tier # Group # 10 CFR 55.43	ating 3(b)	<u>RO-ONLY</u> 001K4.07 3.7 2 1	5	<u>SRO</u>

Question # 13

WHICH ONE (1) of the following Engineered Safety Feature loads will receive a MODE 1 signal first when a Safety Injection occurs?

A. Component Cooling Water pumps

Group #

10 CFR 55.43(b)

- B. Essential Cooling Water pumps
- C. Auxiliary Feedwater pumps
- D. Containment Spray pumps

Answer: I	D	Lesson Plan	LP N LP N	lo. LOT201.22.L, Object lo. LOT201.22.HO.01, p	ive 10, p.2 .7
Question So	ource:	Bank # Modified Bank # New	ŧ	x	
Question C	ognitive Level	: Memory Comprel	or Fu hensic	ndamental Knowledge on or Analysis	X
Examinatio	n Outline Cros	s-reference:			
	Level K/A #			<u>RO</u> 013K5.02	<u>SRO</u>
	Importa Tier #	ance Rating		2.9 2	3.3 2

1

1

Question # 14

The following conditions exist at Unit 1:

- Natural Circulation cooldown in accordance with STPEGS POP05-EO-ES02
- RCS temperature by core thermocouples is 510 deg. F.
- RCS pressure is 1900 psig
- ALL CRDM cooling fans have tripped and cannot be restarted

Which of the following conditions are the CRDM cooling fans analyzed to mitigate during a Natural Circulation cooldown in accordance with STPEGS POP05-EO-ES02?

- Α. Damage to the CRDM coils resulting from overheating.
- Β. Damage to the ex-core NIS resulting from overheating.
- C. Brittle Fracture to the reactor vessel head flange welds resulting from exceeding nil ductility temperature limits.
- D. Formation of a reactor vessel steam bubble which degrades RCS cooldown capability.

Answer: D Lesson Plan LOT504.25.LP

10 CFR 55.43(b)

Resources: STPEGS POP05-EO-ES02 step 5.1 WOG ERG "Natural Circulation Cooldown" p.26

Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Leve	el: Memory or F Comprehen	Fundamental Knowledge sion or Analysis	X
Examination Outline Cro	oss-reference:		
Level K/A #		<u>RO</u> 022K4.04	<u>SRO</u>
Impor	tance Rating	2.8	3.1
Tier #		2	2
Group	o #	1	1

Question # 15

WHICH ONE (1) of the following will cause an automatic trip of the Feedwater Booster pump?

- A. Lube oil temperature greater than 185 degrees F
- B. Lube oil pressure drops to 18 psig
- C. Deaerator storage tank level drops to 4%
- D. Loss of two Condensate pumps

Answer: C		Lesson Plan	LP No. LOT202.13.LP LP No. LOT202.13.LP	, Objective 5, , p. 17	р. 4
Question So	urce:	Bank # Modified Bank # New	X		
Question Co	gnitive Level:	Memory Compreh	or Fundamental Knowl ension or Analysis	edge	_X
Examination	Outline Cros	s-reference:			
	Level		RO-ONLY	<u>S</u>	RO

<u>RO-ONLY</u>
059000K4.16
3.1
2
1

Question # 16

Unit 2 is at 60% power when the RCP 2B trips resulting in a reactor trip. When plant conditions are STABLE, after the trip, SG 2B STEAM FLOW and PRESSURE will indicate as follows (with respect to the other three SGs):

Select ONE of the following:

	SG 2B STEAM FLOW	SG 2B STEAM PRESSURE
A.	HIGHER	LOWER
B.	LOWER	HIGHER
C.	HIGHER	SAME
D.	LOWER	SAME

Answer: D	Lessor	n Plan	(As av	ailable)		
Question Source:		Bank # Modified Bank New	#	X		
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		ntal Knowledge nalysis	X	_
Examination Outline Cross-reference:						
	Level K/A # Importa Tier # Group 10 CFF	ance Rating # R 55.43(b)		<u>RO</u> 003K3.02 3.5 2 1		<u>SRO</u> 3.8 2 1

Question # 17

The Liquid Waste Processing System discharge monitor (RT-8038) has been declared inoperable. Select the statement that describes the action that will permit continued liquid waste discharge.

- A. Liquid waste discharge must be secured until the discharge monitor is operable.
- B. Liquid waste discharge must be secured until the Plant Manager's permission to continue discharge is received.
- C. Two samples are analyzed, and two technically qualified staff members independently verify the release rate calculations and the discharge line valving.
- D. Samples must be taken each 15 minutes while discharge is in progress, to verify the effluent is within Tech Spec limits.

Answer:	С	Lesson Plan	(As available)	
Resource:	CY Technica CY-OP-LO-A	I Specification 3.3.3.7 DMIN-L89990 EO 4	7 p. 3/4 3-46 , 1c	
Question So	urce:	Bank # Modified Bank # New	X	
Question Cognitive Level:		Memory or Funda Comprehension of	Memory or Fundamental Knowledge Comprehension or Analysis	
Examination	Outline Cros	s-reference:		
	Lev K/A	el #	<u>RO</u> 068K4 01	<u>SRO</u>
	Imp Tier	ortance Rating	3.4 2	4.1 2

1

1

Group #

10 CFR 55.43(b)

Question # 18

Which ONE of the following Area Radiation Monitors would require that the unit be shut down if the monitor were out of service for greater than 7 days?

- A. RT-8050, RCB High Range Area Monitor
- B. RT-8090, SFP Area Monitor
- C. RT-8059, Waste Gas Area Monitor
- D. RT-8096, Emergency Operations Facility Area Monitor

Answer:	А	Lesson Plan	(As available)	
Resource:	LOT202.42 R	ev 3; Technical Speci	fication Table 3.3-10 Amendm	nent No. 77/66
Question Sou	rce:	Bank # Modified Bank # New	X	
Question Cognitive Level:		Memory or Fundame Comprehension or A	ntal KnowledgeX nalysis	_
Examination C	Dutline Cross-re	eference:		
	Level K/A #		<u>RO</u> 072A2.02	<u>SRO</u>
Importance Rating Tier # Group # 10 CFR 55.43(b)		ance Rating	2.8	2.9
		·	2	2
		# R 55.43(b)	1	1

Question # 19

Given the following conditions:

- Unit 1 is in Mode 4.
- RCS temperature is 300°F.
- RHR Train 1A is in service supplying low pressure letdown.
- PCV-0135, Letdown Backpressure Control Valve, is in Automatic.

Assuming no operator action, which ONE of the following will occur if an air leak develops on the supply line to PCV-0135?

- A. RHR pump discharge relief will lift.
- B. Running Charging pump will cavitate.

10 CFR 55.43(b)

- C. RCS pressure will increase.
- D. RCS pressure will decrease.

Answer:	D	Lesson Plan	(As available)	
Question S	Source:	Bank # Modified Bank # New	X	
Question C	Cognitive Level	: Memory or Fu Comprehensi	indamental Knowledge on or Analysis	X
Examinatio	on Outline Cros	ss-reference:		
	Level K/A # Import Tier # Group	ance Rating #	<u>RO-ONLY</u> 004A3.10 3.9 2 1	<u>SRO</u>

Question # 20

The following plant conditions exist:

- Unit 1 Reactor power is 80%.
- Rod Control is in MANUAL.
- All other controls in AUTO.
- Turbine Control is in IMP IN

An Emergency Boration is performed for TWO (2) minutes with no resulting reactor trip.

Considering steady-state to steady-state conditions, the parameter that will return to its original value is:

- A. RCS Tavg.
- B. PRZR Level.
- C. SG Pressure.
- D. Reactor Power.

Tier #

Group #

10 CFR 55.43(b)

Answer:	D		Lesson Plan		
Question Sou	rce:	Bank # Modifie New	ed Bank #	x	
Question Cog	nitive Le	evel:	Memory or F Comprehens	undamental Knowledge ion or Analysis	X
Examination C	Dutline (Cross-re	eference:		
	Level K/A # Importa	ance Ra	ating	<u>RO-ONLY</u> 004K5.20 3.6	<u>SRO</u>

2 1

Question # 21

Reactor power is being maintained at approximately 6% as the Main Turbine is being placed on-line. Power Range channel N-42 instrument power is lost. Which of the following actions must be taken in response to this failure?

- A. Take manual control of any Low Power Feed Regulating valves being used to feed steam generators.
- B. Perform a calorimetric to verify operability of unaffected Power Range instrumentation
- C. Contact Reactor Engineering to perform a flux map to verify QPTR within 12 hours.
- D. Restore the inoperable channel prior to exceeding 10% of Rated Thermal Power.

Answer: A	Lesson Plan	(As available)	
Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Le	vel: Memory or Compreher	Fundamental Knowledge nsion or Analysis	X
Examination Outline C	ross-reference:		
Lev K/A	el #	<u>RO</u> 015A2.01	<u>SRO</u>
Imp	ortance Rating	3.5	3.9

Tier #

Group #

10 CFR 55.43(b)

2

1

2

1

Question # 22

Given the following:

- Reactor power is 99%.
- Pressurizer level is 58%.
- Letdown flow 75 gpm (One Orifice).
- A charging line leak develops near the charging line containment penetration that diverts ALL charging flow from the line.
- Normal seal injection is maintained.

10 CFR 55.43(b)

• Assume NO operator action is taken.

WHICH ONE (1) of the following statements describes the FINAL pressurizer response?

- A. Decreasing pressurizer level to 17%, letdown isolates and pressurizer level increases leading to high level trip.
- B. Pressurizer pressure increases to the high pressure trip setpoint following loss of pressurizer spray and the auto start of pressurizer backup heaters due to level deviation.
- C. Stable lower pressurizer level following reduction of letdown flow, due to steam flashing in the regenerative heat exchanger.
- D. Stable higher pressurizer pressure due to heating of the water in the pressurizer, a result of losing the cooling effect of charging flow.

Answer:	А	Lesson Plan	LOT201.14	
Question	Source:	Bank # Modified Bank # New	X	
Question	Cognitive Level	: Memory or Fu Comprehensio	ndamental Knowledge on or Analysis	X
Examinati	on Outline Cros	s-reference:		
	Level K/A # Importance Ra Tier # Group #	ating	RO-ONLY 011A1.2 3.3 2 2	<u>SRO</u>

Question # 23

WHICH ONE (1) of the following bus losses will result in the loss of Containment Spray Pump 1C? A. Class IE 4.16 KV Bus E1C

- B. Non-Class IE 4.16 KV Bus 1D1
- C. Class IE 480 V Bus E1C1
- D. Non-Class IE 480 V Bus LC 1L2

Answer:	а	Lesson Plan	LP No. LOT201.11.L	
Question \$	Source:	Bank # Modified Bank # New	X	
Question (Cognitive Level	: Memory or Fu Comprehensio	ndamental Knowledge on or Analysis	X
Examination	on Outline Cros	ss-reference:		
	Level K/A # Importance Ra Tier # Group # 10 CFR 55.43	ating (b)	<u>RO</u> 0026K2.02 2.7 2 2	<u>SRO</u> 2.9 2 1

Question # 24

Given the following:

- Plant is at 100% power steady state operation.
- Preparations for performing a supplementary containment purge are in progress.
- Noble gas concentration inside the RCB is 5.2E-04 uCi/cc.

WHICH ONE (1) of the following actions should be taken to prevent the actuation of an ESF Containment Ventilation Isolation during the supplementary purge?

- A. Increase alarm setpoint of RT-8012 (Containment atmosphere radiation monitor) & 8013 (Stack radiation monitor).
- B. Ensure that RT-8011 (Containment atmosphere radiation monitor) output is blocked.
- C. Increase alarm setpoint on RT-8011 (Containment atmosphere radiation monitor).
- D. Remove power from CVI relays in the ESF actuation panel.

Answer: A		0POP02-HC-00	003 - "Supplementary C	ontainment Purge"
Question Source:	Bank # Modifie New	d Bank #	X	
Question Cognitive	Level:	Memory or Fun Comprehensio	damental Knowledge n or Analysis	x
Examination Outline	e Cross-refere	ence:		
Level K/A # Importar Tier # Group # 10 CFR	nce Rating 55.43(b)		<u>RO-ONLY</u> 029A2.03 2.7 2 2	<u>SRO</u>

Question # 25

WHICH ONE (1) of the following can supply power to the Positive Displacement Charging pump in the event of a loss of offsite power?

- A. Standby Diesel Generator #11.
- B. Standby Diesel Generator #12.
- C. Lighting Diesel Generator.

Group #

10 CFR 55.43(b)

D. TSC Diesel Generator.

Answer: D	Lesson Plan	LOT201.06	
References: Dwg 480V	MCC 1G8		
Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Leve	l: Memory or Fu Comprehensi	indamental Knowledge on or Analysis	X
Examination Outline Cro	ss-reference:		
Level K/A #		<u>RO</u> 062A1.01	<u>SRO</u>
Importance R	ating	3.4	3.8
Tier #		2	2

2

2

Question # 26

An operate failure was just received on a radiation monitor channel that is monitored by the RM-11.

WHICH ONE of the following colors would indicate the channel operate failure on the RM-11?

- A. dark blue
- B. light blue (cyan)
- C. white
- D. magenta
- Answer: A

Lesson Plan LOT202.41.LP

Question Source: Bank # _X____ Modified Bank # New

Question Cognitive Level:	Memory or Fundamental Knowledge	X
-	Comprehension or Analysis	

Examination Outline Cross-reference:

Level	RO	<u>SRO</u>
K/A #	073A2.02	
Importance Rating	2.7	3.2
Tier#	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 27

Given the following:

- Unit 1 is in Mode 6.
- Pressure transmitter PT-405 has failed HIGH.

WHICH ONE (1) of the following describes the response of the Residual Heat Removal (RHR) Inlet Isolation valves to this failure?

- A. BOTH valves in ONE (1) train CLOSE.
- B. ONE (1) valve in EACH of the THREE (3) trains CLOSES.
- C. BOTH valves in TWO (2) trains CLOSE.
- D. ONE (1) valve in each of TWO (2) trains CLOSES.

Answer:	D		Lesson Plan	LP NO. LC	DT201.09	
Reference	s: DWG:	9-Z-421	82			
Question S	Source:	Bank # Modifie New	d Bank #	x		
Question Cognitive Level:		:	Memory or Fu Comprehensio	ndamental on or Analys	Knowledge sis	X
Examinatio	Examination Outline Cross-reference:					
	Level K/A #			<u>RO</u> 005K2.03		<u>SRO</u>
	Importance Ra	ating		2.7		3.8
	Lier#			2		2
	10 CFR 55.43	(b)		3		3

Question # 28

Given the following:

- Unit 1 is in MODE 4.
- RCS temperature is 325 degrees.
- RCS pressure is 340 psig.
- RHR is in service.
- An unisolable leak in the Instrument Air (IA) system has occurred.
- IA system pressure is 60 psig and decreasing.

WHICH ONE (1) of the following describes how the RHR system will respond?

- A. RHR heat exchanger bypass valves FCV-851/852/853 will fail OPEN and cause RCS temperature to DECREASE.
- B. RHR heat exchanger bypass valves FCV-851/852/853 will fail CLOSED and cause RCS temperature to INCREASE.
- C. RHR heat exchanger flow control valves HCV-864/865/866 will fail OPEN and cause RCS temperature to DECREASE.
- D. RHR heat exchanger flow control valves HCV-864/865/866 will fail CLOSED and cause RCS temperature to INCREASE.

Answer: C

Lesson Plan LOT201.09, p. 11, LO #5. LOT504.02, LO #8. LOT202.26.TP.17

Question Source:	Bank #	X
	Modified Bank #	
	New	

Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

_X___

SRO

Examination Outline Cross-reference:

Level	RO-ONLY
K/A #	078K3.02
Importance Rating	3.4
Tier #	2
Group #	3
10 CFR 55.43(b)	

Question # 29

Unit 1 is operating at 100% power with all systems aligned normally. A tube leak of approximately 25 gpm develops in the operating ECW/CCW Heat Exchanger.

Which ONE of the following describes the FIRST indication the operator would have of this leak?

- A. 2M03/F5 CCW SURGE TK LVL HI alarm
- B. 2M03/F6 CCW SURGE TK LVL LO alarm
- C. CCW Chemistry results
- D. Increased cycling of the CCW Surge Tank Makeup Valve

Answer:	D	Lesso	n Plan	(As available)		
References:						
Question Sou	urce:	Bank a Modifi New	# ed Bank #	X		
Question Cognitive Level:		Memory or Fu Comprehension	ndamental Knowledge on or Analysis	X	-	
Examination Outline Cross-reference:						
	Le	vel		RO		SRO

Level	<u>R0</u>	<u>SRO</u>
K/A #	008K1.01	
Importance Rating	3.1	3.1
Tier #	2	2
Group # 10 CFR 55.43(b)	3	3

Question # 30

Given the following plant conditions:

- Safety Injection Actuated
- PZR pressure 1800 psig, slowly decreasing
- RCS temperature 560°F, slowly decreasing
- SG NR levels 1%, slowly increasing
- PRT pressure 3 psig, stable
- SG pressure 1100 psig, stable
- PZR level 28%, increasing
- RCB temperature 140°F, slowly increasing
- RCB pressure 3.5 psig
- RCB humidity Increasing

Which ONE of the following could be a cause of the above conditions?

- A. Steam line break upstream of MSIV
- B. Pressurizer PORV failed open
- C. RCS leak from a cold leg
- D. Pressurizer steam space break

Answer:	D	Lesson Plan	(As available)	
Question Sou	urce:	Bank # Modified Bank # New	X	
Question Cognitive Level:		Memory or Fund Comprehension	amental Knowledge or Analysis	X
Examination	Outline Cross	s-reference:		
	Lev K/A	el #	<u>RO</u> G2 1 7	<u>SRO</u>
	Imp	ortance Rating	37	44
	Tier	#	3	3
	Gro	 up #	Ū	Ū
	10 (CFR 55.43(b)		

Question # 31

0POP01-ZA-0018, Emergency Operating Procedures User's Guide, allows certain actions to be taken outside the guidance of the EOPs.

Which ONE of the following actions can be taken?

Group #

10 CFR 55.43(b)

- A. Isolating AFW to a ruptured SG and closing the Main Steam Isolation Valve from the ruptured SG during the performance of 0POP05-EO-EO00, Reactor Trip or Safety Injection.
- B. Cross-connecting AFW to supply all four SGs, at 300 gpm per SG, with one AFW pump during the performance of 0POP05-EO-FRS1, Response to Nuclear Power Generation ATWS.
- C. Resetting Phase A Isolation and restoring Instrument Air to Containment during the performance of 0POP05-EO-EO00, Reactor Trip or Safety Injection.
- D. Securing HHSI Pumps to prevent the pressurizer from going solid during the performance of 0POP05-EO-EO00, Reactor Trip or Safety Injection.

Answer:	С	Lesson Plan	(As available)	
Resources:	0POP01-ZA-	0018, Rev 12, sectior	า 4.25	
Question Sou	urce:	Bank # Modified Bank # New	x	
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		X
Examination	Outline Cross-	reference:		
Level K/A # Importance Rating Tier #		tance Rating	<u>RO</u> G2.1.1 3.7 3	<u>SRO</u>

Question # 32

Given the following:

- A Reactor Startup is being performed following a mid-cycle outage.
- Reactor Power has been stabilized at 1E-8 amps
- RCS temperature is at the no-load value
- Critical data has been taken.
- Prior to any additional control rod movement, a single SG Safety Valve on SG 1D fails opens and remains open.
- RCS Tavg decreases 9°F and reactor power starts to increase.

Which ONE of the following states the correct action, required to satisfy Technical Specification LCO(s)?

- A. Reduce power range high flux high trip setpoint to 87% rated thermal power.
- B. Restore SG safety valve to operable status prior to entering Mode 1.
- C. Restore RCS Tavg within 15 minutes or be in Mode 3 within the next 15 minutes.
- D. Immediately initiate boration to restore Shutdown Margin.

Answer: C		Lesson Plan	(As available)		
Reference:	T.S. L	COs 3.1.1.1; 3.1.1.4;	3.7.1.1		
Question Source:		Bank # Modified Bank # New	X		
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		X	_
Examination Outline	Cross-r	eference:			
	Level K/A # Import Tier # Group 10 CF	ance Rating # R 55.43(b)	RO G2.11 3.0 3		<u>SRO</u> 3.8 3

Question # 33

Given the following conditions on Unit 2:

- Fuel movement operations are in progress in the Spent Fuel Pool (SFP)
- The Fuel Handling Machine is located over a SFP storage location
- A fuel assembly is latched on the long handling tool
- The fuel assembly is visually clear of the storage cell
- The hoist is NOT in its full up position but hoist motion has stopped

Under these conditions, which ONE of the following indicates the limitations on the operation of the Fuel Handling Machine?

- A. The bridge and the trolley are interlocked to prevent movement in any direction.
- B. Bridge is restricted to inching speed mode but the trolley may be moved normally.
- C. The bridge may be moved normally but the trolley is restricted to inching speed mode.
- D. The bridge speed and the trolley speed are restricted to inching speed mode.

Answer: B		Lesson Plan	(As available)	
Resources:	0POP08-FH-(0002, Rev 10, step 5.5		
Question Sour	ce:	Bank # Modified Bank # New	x	
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		X
Examination O	outline Cross-r	eference:		
	Level K/A # Import	ance Rating	<u>RO-ONLY</u> G2.1.28 3.2	<u>SRO</u>

3

Tier #

Group #

10 CFR 55.43(b)

Question # 34

Which ONE of the following clearance situations would be allowed by 0PGP03-ZO-EC01, Equipment Clearance Orders?

- A. Using a Danger Tag as an administrative lock in place of a mechanical locking device to meet the requirements of 0POP01-ZA-0001, Plant Operations Administrative guidelines.
- B. Moving Electrical Grounding Devices to other parts of de-energized equipment using Test Tags only.
- C. Sequencing an Equipment Clearance Order so that a pump is mechanically isolated prior to disabling the motive force.
- D. Hanging a Danger Tag on a switchgear cubicle door with a position of "RACKED OUT", then placing the breaker in the TEST position.

Answer: B		Lesson Plan	(As available)	
Resource: 0PGP03	-ZO-EC	01		
Question Source:		Bank # Modified Bank # New	X	
Question Cognitive Level:		Memory or Fundamer Comprehension or Ar	ntal KnowledgeX nalysis	<u> </u>
Examination Outline (Cross-re	ference:		
	Level		<u>RO</u> C2 2 13	<u>SRO</u>
Importance Rating Tier # Group #		ince Rating #	3.6 3	3.8 3

10 CFR 55.43(b)

Question # 35

Unit 2 is operating at 50% power when the CVCS Failed Fuel Rad Monitor, ALARMS. Chemistry has reported the following confirmed sample results:

Dose Equivalent I-131	= .0005 mci/gram
Gross Activity	= 40 mci/gram
E-bar	= 2.65 Mev

Which ONE of the following statements applies for the given conditions?

A. Plant may remain at this power level indefinitely.

B. Perform isotopic analysis for lodine once per 4 hours.

- C. Be in HOT STANDBY with Tave < 500°F within 6 hours.
- D. Within 1 hour initiate action to place the unit in HOT STANDBY within the next 6 hours.

Answer:	С	Lesson Plan	(As available)	
Resource:				
Question So	urce:	Bank # Modified Bank # New	X	
Question Cognitive Level:		Memory or Fundame Comprehension or A	ntal KnowledgeX_	_
Examination	Outline Cross-r	eference:		
	Level K/A # Impor Tier # Group 10 CF	tance Rating # R 55.43(b)	<u>RO</u> G2.2.22 3.4 3	<u>SRO</u> 4.1 3

Question # 36

A job has been assigned which required a pre-job ALARA review. The job is projected to result in a total exposure of 1 person-rem.

Which ONE of the following total exposures would be the minimum requirement that the job be placed on ALARA hold per 0PGP03-ZR-0052, ALARA Program?

- A. Total exposure reaches 0.75 person-rem
- B. Total exposure reaches 1.0 person-rem
- C. Total exposure reaches 1.25 person-rem
- D. Total exposure reaches 1.5 person-rem

Answer:	С	Lesson Plan	(As available)		
Resource:	0PGP03-ZR-0052, Rev 3, section 5.7				
Question Sc	purce:	Bank # Modified Bank # New	X		
Question Cognitive Level:		Memory or Fundamer Comprehension or Ar	ntal KnowledgeX nalysis		
Examination	Outline Cross-re	eference:			
	Level K/A #		<u>RO-ONLY</u> G2.3.2	<u>SRO</u>	

2.5

3

Importance Rating

10 CFR 55.43(b)

Tier #

Group #
Question # 37

A mechanic has the following exposure history:

· Current Year to date:

150 mrem TEDE from offsite occupational exposure at another nuclear facility 750 mrem TEDE from onsite occupational exposure

In the week following the above reported information, the mechanic records the following exposure on a job in Unit 2:

Gamma - 55 mrem Neutron - 20 mrem

Which one of the following correctly lists the dose margin (TEDE) to the STP Administrative Action Level for this individual, if NO Personnel Dose Extension Authorization has been granted?

- A. 1025 mrem
- B. 1045 mrem
- C. 1175 mrem
- D. 1195 mrem

`	Loogon Dlan
,	LESSUII FIAII

(As available)

Resource: PGP0	3-ZR-0050
----------------	-----------

Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Level:	Memory or Fundam Comprehension or <i>i</i>	ental Knowledge Analysis	<u></u> X

Level	<u>RO</u>	<u>SRO</u>
K/A #	G2.3.4	
Importance Rating	2.5	3.1
Tier #	3	3
Group #		
10 CFR 55.43(b)		

Question # 38

The following conditions exist:

- RHR was in operation
- The plant is in Mode 5
- Reactor Coolant System is in Mid-Loop

The Control Room Crew has entered 0POP04-RH-0001, Loss of Residual Heat Removal. RHR is lost and CANNOT be restored.

Which ONE of the following methods of cooling will the operators use to remove the decay heat from the reactor?

- A. Start a HHSI pump and initiate Cold Leg Injection.
- B. Start a LHSI pump with flow through the RHR heat exchanger, and initiate Hot Leg Injection.
- C. Feed a SG using an AFW pump, and bleed steam through the respective SG PORV.
- D. Manually initiate Safety Injection, use RCS feed an bleed to remove decay heat.

Answer:	D	Lesson Plan	(As available)		
Resource:	0POP04-RH-0	H-0001			
Question Sou	rce:	Bank # Modified Bank # New	X		
Question Cognitive Level:		Memory or Fundame Comprehension or Ar	ntal KnowledgeX nalysis	-	
Examination C	Dutline Cross-re	eference:			
	Level		<u>RO</u> G2 4 24	<u>SRO</u>	
	Import Tier #	ance Rating	3.3 3	3.7 3	

Group # 10 CFR 55.43(b)

Question # 39

Given the following conditions on Unit 1:

- Reactor power is being maintained at 30%
- · SGFPs 11 and 12 are in service
- · All controls are in automatic

SGFP 12 trips due to a loss of lube oil and 0POP04-FW-0002, Steam Generator Feedpump Trip is entered.

Which ONE of the following describes the immediate operator actions that should be taken?

- A. 1) Check the SGFP Master Speed Controller operating correctly.
 - 1) Ensure Feed Flow is greater than or equal to Steam Flow.
- B. 1) Start a standby FW Booster Pump.2) Reduce turbine load.
- C. 1) Check the SGFP Master Speed Controller operating correctly.2) Reduce turbine load.
- D. 1) Start a standby FW Booster Pump.
 2) Place all SG Feedwater Regulating Valves in MANUAL.

Answer:	А	Lesson Plan	(As available)
---------	---	-------------	----------------

- Resource: 0POP04-FW-0002 Rev 2, Steps 1-3
- Question Source: Bank # ____X___ Modified Bank # _____ New
- Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

_X___

Level	RO-ONLY	<u>SRO</u>
K/A #	G2.4.49	
Importance Rating	4.0	
Tier #	3	
Group #		
10 CFR 55.43(b)		

Question # 40

Given the following conditions:

- •Unit 1 is operating at 75%
- •Condensate pump is out of service for maintenance
- •Power is being increased at 10% per hour

Condensate pump 11 trips from UKNOWN causes.

Which ONE of the following is the appropriate action per the COND PMP TRIP annunciator response?

- Α. Stop the power increase and stabilize the Deaerator level at the present power level.
- Β. Attempt to restart the Condensate Pump 11, then continue the power increase once it is running.
- C. Commence a power decrease to less than 50% at 5% per minute.
- D. Trip the turbine and stabilize the Deaerator level at no load conditions

Answer:	С	Lesson Plan	(As available)	
Reference:	0POP09-AN-09M	1, window A-1		
Question Sou	rce:	Bank # Modified Bank # New	X	
Question Cog	nitive Level:Memo	ry or Fundamental Kno Comprehension or Ar	owledge nalysisX	
Examination (Dutline Cross-refer	ence:		
	Level K/A #		<u>RO</u> 056G2 1 27	<u>SRO</u>
	Import	ance Rating	2.8	2.9
	Tier #	5	2	2
	Group	#	1	1

10 CFR 55.43(b)

Question # 41

The reactor operator is making a blended addition to the Volume Control Tank and sets up the controls as follows:

- TOT M/U BATCH integrator set at 400 gallons
- BA BATCH integrator set at 70 gallons
- RC M/U CONT selector switch set to MAN
- All blender values in AUTO
- RC M/U CONT is taken to START and the blending operation begins

Due to a malfunction, the TOT M/U BATCH integrator stops counting the gallons added part way through the operation.

Assuming the malfunction goes unnoticed by the operator, which ONE of the following describes the effect of the malfunction?

- A. Boric acid will continue to inject until stopped by the operator, causing a boration of the RCS.
- B. Reactor Makeup Water will continue to inject untill stoped by the operator, causing a dilution of the RCS.
- C. Boric acid and Reactor Makeup Water will both continue to inject at their present flowrates untill stopped by the operator causing the Volume Control Tank to fill with properly blended flow.
- D. Boric Acid and Reactor Makeup Water will both stop injecting immediately and operator action will be required to continue blending.

Answer: B Lesson Plan: LOT201.07 Rev 7 Student Handout

Question Source:	Bank # Modified Bank # New	X
Question Cognitive Level:	Memory or Fundame Comprehension or Ar	ntal Knowledge nalysis

Examination Outline Cross-reference:

Level	RO	<u>SRO</u>
K/A #	004K6.13	
Importance Rating	3.1	3.3
Tier #	2	2
Group #	1	1
10 CFR 55.43(b)		

Х

Question # 42

Per 0PGP03-ZF-0018, Fire Protection Operability Requirements, which ONE of the following identifies the minimum operability requirements for the Fire Protection Water Supply System?

Sto	rage Tank Vo Gal/tank	lume Numbo	er of Pumps	Pump C GPM	apacity
A.	200,000		2	2500	
В.	250,000		1	2000	
C.	300,000		2	2500	
D.	350,000		1	2000	
Answer:	С	Lesson Plan:	LOT201.29		
Reference:	0PGP03-ZF	-0018 Rev 9			
Question S	ource:	Bank # Modifie New	X ed Bank #		
Question C	Cognitive Leve	l: Memo	ry or Fundame Comprehens	ental Knowledge ion or Analysis	x
Examination Outline Cross-reference:					
	Level K/A # Impo Tier # Grou 10 Cl	t tance Rating # p # FR 55.43(b)	RO 086G2.4.25 2.9 2 2		<u>SRO</u> 3.4 2 2

Question #43

Unit 1 is operating at 100% power when Instrument Air pressure begins to decrease. The BOP operator reports that Instrument Air pressure is 95 psig and slowly lowering.

Which ONE of the folling actions should have already occurred?

- A. 1-IA-PV-8561, Instrument Air Dryer Bypass Valve is OPEN
- B. 1-CV-FV-0011, Letdown Orifice Header Isolation Valve is CLOSED
- C. 1-IA-PV-8559, Service and Instrument Air Crossover Valve is OPEN
- D. 1-IA-PV-8568, Instrument Air to Yard Isolation Valve is CLOSED

Answer: C Lesson Plan: LOT202.26

Reference: 0POP04-IA-0001, Rev 3; 0POP02-IA-0001, Rev 11, Section 5.0; 0POP09-AN-08M3-F-3, Rev 9.

Question Source:	Bank # Modified Bank # New	.#X	
Question Cognitive Level	I: Memory or Fu Comprehensi	undamental Knowledge ion or Analysis	X

Level	RO-ONLY	<u>SRO</u>
K/A #	079K4.01	
Importance Rating	2.9	
Tier #	2	
Group #	2	
10 CFR 55.43(b)		

Question # 44

The following conditions exist on Unit1:

- At 0400 hours, a fire in the Central Alarm Station was reported to the control room
- Fire brigade has been dispatched
- Control Room staff is performing 0POP04-ZO-0008, Fire/Explosion
- At 0445 hours, the Fire Brigade Leader reports that the fire is not yet extinguised

Under these conditions, which ONE of the following actions should be taken?

- A. Place EAB HVAC in the Smoke Purge mode of operation.
- B. Close at least one of the Control Room Envelope HVAC Train C Inlet Isolation Dampers, INL DMPR FV-9664 or FV-9665.
- C. Isolate EABHVAC Train C using the train actuation handswitch in the stairwell.
- D. Actuate the deluge valves for the Control Room Envelope Train C filters
- Answer: C Lesson Plan: LOT505.01

Reference: 0POP04-ZO-0008 Rev 3, step 4.28

 Question Source:
 Bank # ____X___

 Modified Bank # _____

 New

 Question Cognitive Level:
 Memory or Fundamental Knowledge _____

 Comprehension or Analysis
 ______X___

Level	RO	<u>SRO</u>
K/A #	000067G2.4.25	
Importance Rating	2.9	3.4
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question #45

Tropical storm Barry has made landfall at Galveston and spawned a series of strong tornados in the area. One of these tornados has caused structural damage to the switchyard equipment, which has caused a Loss of Offsite Power (LOOP).

Which ONE of the statements below correctly describes the emergency diesel for Train A actuation/operation under these conditions?

- A. Auto DG start signal, breaker auto closes after 10 seconds, all trips blocked (except for emergency trips) by energizing 20SD, ESF Load Sequencer in Mode II, recovery to perferred source of power is done manually.
- B. Auto DG start signal, breaker auto closes after 6 seconds, all trips blocked (except for emergency trips) by de-energizing 20SD, ESF Load Sequencer in Mode II, recovery to preferred source of power is done manually.
- C. Auto DG start signal, breaker auto closes after 6 seconds, all trips blocked (except for emergency trips) by de-energizing 20SD, ESF Load Sequencer in Mode III, recovery to perferred source of power is done manually.
- D. Auto DG start signal, breaker auto closes after 10 seconds, all trips blocked (except for emergency trips) by energizing 20SD, ESF Load Sequencer in Mode III, recovery to perferred source of power is done manually.

1

1

Answer:	В	Lesson Plan:	LOT201.41, L	_OT201.39	
Reference:	TS 3/4 3.8	.1.1; UFSAR 8	.3		
Question So	urce:	Bank # Modifi New	ed Bank #		
Question Co	gnitive Leve	el: Memo	ory or Fundame Comprehensi	ental Knowledge on or Analysis	X
Examination	Outline Cro	ss-reference:			
	Leve K/A #	 #	<u>RO</u> 000055A1.06	i	<u>SRO</u>
	Impo	rtance Rating	4.1		4.5
	Tier	#	1		1

Group #

10 CFR 55.43(b)

Question #46

An event has occurred on Unit 2 and operators are conducting 0POP05-EO-EO10, "Loss of Reactor or Secondary Coolant", when the following parameters are observed:

- All S/G pressures---800 psig and stable
- All S/G levels---being controlled at 10%
- PZR level---off-scale low
- Containment pressure---16 psig
- RWST level---17%
- RCS pressure---180 and stable

Based on these conditions, which ONE of the following procedures would the operators enter next to mitigate the event in progress?

- A. 0POP05-EO-ES12, "Post LOCA Cooldown and Depressurization
- B. 0POP05-EO-ES11, "SI Termination"
- C. 0POP05-EO-ES13, "Transfer to Cold Leg Recirculation"
- D. 0POP05-EO-EC11, "Loss of Emergency Coolant Recirculation"
- Answer: C Lesson Plan:
- Reference: 0POP05-EO-ES13, 0POP05-EO-EO10

Question Source:	Bank # Modii New	ied Bank #	X	
Question Cognitive L	evel: Mem	ory or Fundam Comprehen	iental Knowledge sion or Analysis	X_
Examination Outline	Cross-reference:			
L K	evel /A #	<u>R0</u> G2.4.1		<u>SRO</u>
lr T G	nportance Rating ier # iroup #	4.3 3		4.6 3

10 CFR 55.43(b)

Question #47

Which ONE of the following control room indications would be the most useful immediately following event to discriminate between a large steamline break in containment and a large LOCA inside containment?

- Α. Containment sump levels
- В. Pressurizer level
- C. Containment radiation levels
- Power range NIS D.

Answer: С Lesson Plan:

Reference:

Question Source:

Modified Bank # _X__

Question Cognitive Level:	Memory or Fundamental Knowledge	X
-	Comprehension or Analysis	

Bank #

New

Level	RO	<u>SRO</u>
K/A #	000011A2.13	
Importance Rating	3.7	3.7
Tier #	1	1
Group #	2	1
10 CFR 55.43(b)		

Question #48

Given the following:

- Unit 1 is at 100% power.
- The unit has experienced a loss of 120V AC Vital Instrument Power to DP-1204.
- Rod control is in AUTO.

Which ONE of the following describes the effect on the plant?

- A. An Urgent Failure will cause a loss of automatic and manual rod control.
- B. VCT automatic makeup signal is generated.
- C. Power Range N44 fails low resulting in CONTINUOUS rod insertion.
- D. Pressurizer Heater Groups A, B, C, D, and E energize.

Answer: B	Lessor	n Plan: LOT20	1.38		
Reference:	0POP04-VA-0	001			
Question Source:		Bank # Modified Bank New	.#	_x	
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		<u></u> X	
Examination Outli	ne Cross-refer	ence:			
	Level K/A #		<u>RO</u> 000057A	2.13	<u>SRO</u>
	Importance Ra	ating	3.0		3.4
	Tier #	0	1		1
	Group # 10 CFR 55.43	(b)	1		1

Question #49

The following conditions exist on Unit 1:

- Reactor power at 93%
- End of Life
- Power increasing via Boron Dilution

At 1100 hours, an indication of a single control rod in Shutdown Bank D suddenly falls to a position of 170 steps. Attemps to move the rod have failed. At 1130 the control rod was declared inoperable, but trippable.

Which of the following staments is CORRECT concerning this situation?

- A. The Unit must be taken to HOT STANDBY within the next 6 hours
- B. Power must be reduced to 75% RTP or lower within the next 30 minutes due to the ensuing Xenon transient, which will cause the deviation from the Axial Flux Distribution target band.
- C. Power operation at 80% allowed as long as the Shutdown Margin is verified within one hour and High Neutron Setpoint Trip is set at 85% within 4 hours.
- D. Power must be reduced to 75% RTP or lower within the next one (1) hour and have the rod restored to operable status within 72 hours.

Answer: B Lesson Plan:

Reference: 0POP04-RS-0001, "Control Rod Malfunction"; TS 3.1.3.1.b.3

Question Source:

Bank #	
Modified Bank #	
New	X

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

X

Level	RO	<u>SRO</u>
K/A #	000005K1.03	
Importance Rating	3.2	3.6
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question # 50

0POP02-RC-0004, Operation of Reactor Coolant Pumps, permits two successive starts of a RCP (provided the motor coasts to a complete stop).

Which ONE of the statements below describes the bases for RCP starting duties?

Limiting the number of RCP starts in a short period of time prevents damage to the:

- A. RCP motor stator windings.
- B. RCP breaker protection relays.
- C. RCP breaker junction terminals.
- D. RCP motor armature insulation.

10 CFR 55.43(b)

Answer:	А	Lesson Plan:					
Reference:	RC-04, RCP,	basis					
Question Sou	rce:	Bank # Modified Ban New	k #	753			
Question Cog	nitive Level:	Memo Comp	ry or Fu	indamental on or Analy	Knowledge rsis	9	_x
Examination (Outline Cross-re	eference:					
	Level K/A # Importance Ra Tier # Group #	ating	<u>RO</u> 015/17 3.2 1 1	′G2.1.28		<u>SRO</u> 015/17 3.3 1 1	'G2.1.28

Question # 51

Given the following conditions:

- Unit 1 is in Mode 5
- CCW Trains A and C are in service
- CCW Train B is selected to STANDBY
- ALL ECW Trains are in service

CCW Pump A trips causing common header pressure to decrease to 70 psig. What is the effect on CCW Pump B?

CCW Pump B will:

- A. start ONLY if the mode selector switch is placed in run.
- B. NOTstart because ECW Trains A and C are still in service.
- C. start because the common header pressure is less than the low-pressure setpoint.
- D. NOT start because the common header pressure did not decrease to the low-pressure setpoint.

Answer:	С	Lesson Plan:				
Reference:	LOT 201.12.L	.12.LP Rev 7 section 4.3.3.C				
Question Sou	rce:	Bank # Modified Bank # New	_X-124			
Question Cognitive Level:		Memory or Fundamental Knowledge Comprehension or Analysis		X		
Examination (Outline Cross-r	eference:				
	Level K/A # Impor	ance Rating	<u>RO</u> 026AK3.04 3.5	<u>SRO</u> 026AK3.04 3.7		

1

1

1

1

Importance Rating Tier # Group # 10 CFR 55.43(b)

Question # 52

The plant is in Mode 3 at normal operating temperature and pressure, Train A COMS has inadvertently been left ARMED for Cold Overpressure Protection.

The selected pressurizer pressure channel, PT-455, subsequently fails high.

With no operator actions, which ONE of the following states the expected plant response?

- A. PORV 655A initially opens, then closes when actual PZR pressure decreases to <2185 psig.
- B. PORV 655A stays closed initially, but will function as required for COMS.
- C. PORV 655A initially opens, and stays open when actual PZR pressure decreases to <2185 psig.
- D. PORV 655A stays closed initially, and PORV BLOCK VALVE (MOV-001A) closes when actual PZR pressure decreases to <2185 psig.

Answer:	А	Lesson Plan:		
Reference:				
Question Sou	rce:	Bank # Modified Bank # New	X-901	
Question Cog	nitive Level:	Memory or Fo Comprehensi	undamental Knowledge ion or Analysis	e
Examination (Dutline Cross-re	eference:		
	Level		RO	<u>SRO</u>

Level	<u>R0</u>	SRO
K/A #	APE027AA2.11	APE027AA2.11
Importance Rating	4.0	4.1
Tier #	1	1
Group #	1	2
10 CFR 55.43(b)		

Question # 53

- An event has occurred on Unit 2 and the Integrity Critical Safety Function is Orange. Assuming all plant systems functioned as designed, what event is the most likely cause for the Orange Path condition and what parameter(s) are used to assess this condition?
- A. LOCA on the cold leg and an increase in RCS hot leg temperature.
- B. Uncontrolled depressurization of all steam generators and an increase in containment pressure.
- C. LOCA on the cold leg and an increase in containment pressure.
- D. Uncontrolled depressurization of all steam generators and a decrease in RCS cold leg temperatures.

Answer: D Lesson Plan:

Reference: Procedure OPOP05-EO-F004, "Integrity Critical Safety Function Status Tree"

Question Source:

Bank # _____ Modified Bank # 31_____ New

Question Cognitive Level:Memory or Fundamental Knowledge
Comprehension or Analysis

_X___

Level	RO	<u>SRO</u>
K/A #	APE040G2.4.21	APE040G2.4.21
Importance Rating	3.7	4.3
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question # 54

Procedure 0POP05-EO-FRP1, "Response to Imminent Pressurized Thermal Shock Condition," has a step to terminate safety injection flow. However, if the safety injection termination criteria are not satisfied, a reactor coolant pump should be started. What is the basis for this step?

- A. Establishing forced flow which allows safety injection to be terminated.
- B. Allows RCS depressurization via the pressurizer spray nozzle.
- C. Equalizes steam generator pressures to allow cooldown of all 4 loops.
- D. Mixes the incoming SI water and the RCS water to raise the temperature of the water entering the downcomer.

Answer: D Lesson Plan:

Reference: WOG Emergency Response Guidelines Background Document, "Response to Imminent Pressurized Thermal Shock Condition.

Question Source:	Bank # Modified Bank # New	X
Question Cognitive Level:	Memory or F Comprehens	Fundamental Knowledge sion or Analysis

__X___

Level	RO	SRO
K/A #	W/E08EK2.2	W/E08EK2.2
Importance Rating	3.6	4.0
Tier #	1	1
Group # 10 CFR 55.43(b)	1	1

Question # 55

Given the following:

- Unit 2 has suffered a Large Break LOCA.
- Operators have performed all actions of 0POP05-EO-EO00, Reactor Trip or Safety Injection.
- While transitioning from EO00, a Red path was identified for Core Cooling and 0POP05-EO-FRC1, Response to Inadequate Core Cooling was entered.

Based on 0POP05-EO-FRC1, what is the correct sequence of major actions to address the inadequate core cooling?

- Establish safety injection flow to the RCS; Start RCPs and open all RCS vent paths to Α. containment; and, rapidly depressurize the SGs to depressurize the RCS.
- Β. Establish safety injection flow to the RCS; rapidly depressurize the SGs to depressurize the RCS; and, Start RCPs and open all RCS vent paths to containment.
- C. Start RCPs and open all RCS vent paths to containment; Establish safety injection flow to the RCS; and, rapidly depressurize the SGs to depressurize the RCS.
- D. Rapidly depressurize the SGs to depressurize the RCS; Establish safety injection flow to the RCS; and, start RCPs and open all RCS vent paths to containment.

Answer: В Lesson Plan:

Bank #

New

Procedure 0POP05-EO-FRC1, "Response to Inadequate Core Cooling" and Reference: WOG "Response to Inadequate Core Cooling"

Question Source:

Modified Bank #

Question Cognitive Level:

Memory or Fundamental Knowledge Х Comprehension or Analysis

Level	RO	SRO
K/A #	EPE074EK2.04	EPE074EK2.04
Importance Rating	3.9	4.1
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question # 56

Operators are performing a startup on Unit 1. The reactor is critical and currently at 1.0E-8 amps. The reactor startup is being performed in accordance with Procedure 0POP03-ZG-0004, "Reactor Startup."

Currently, the operators have completed all procedure steps up to and including stabilizing reactor power at 1.0E-8 amps. As the crew prepares for shift turnover, a rod control circuit fails and initiates a continuous rod withdrawal event.

With no operator action, which of the below reactor trips will terminate the power increase?

- A. Intermediate Range high flux trip at 20% power.
- B. Source Range high flux trip at 1.0E5 cps.
- C. Power Range Flux Low setpoint trip at 25% power.
- D. R & S Logic Train urgent failure alarms

Answer: C Lesson Plan:

Reference:LOT201.20HO.02, "SSPS Study Guide"

Question Source:

Bank # _____ Modified Bank # ___X___ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

X____

Level	RO	SRO
K/A #	APE001AA1.05	APE001AA1.05
Importance Rating	4.3	4.2
Tier #	1	1
Group #	2	1
10 CFR 55.43(b)		

Question # 57

Given the following:

- The reactor has tripped from 100% power due to a loss of offsite power.
- All systems have responded normally.
- The Unit Supervisor has just asked the Primary RO if "RCS temperature is STABLE or TRENDING to 567°F."

Which ONE of the following indications should the Operator use when reporting RCS temperature status in accordance with 0POP05-EO-ES01, Reactor Trip Response?

- A. T_{AVG} since DT is zero (0).
- B. T_{HOT} to verify subcooling.
- C. T_{COLD} to trend reactor vessel cooling.
- D. Core exit thermocouple to ensure core cooling.

Answer:	С	Lesson Plan:
---------	---	--------------

Reference: Procedure 0POP05-EO-ES01, "Reactor Trip Response," Step 1

Question Source:

Bank # ____805_ Modified Bank # _____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

_X__

Level	RO	<u>SRO</u>
K/A #	EPE007EA1.03	EPE007EA1.03
Importance Rating	4.2	4.1
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 58

During normal operation, RT-8027, Condenser Air Removal Rad Monitor alarms. A steam generator tube leak is suspected. Given the following information:

- Charging Flow: 120 gpm
- Letdown Flow: 75 gpm
- Pressurizer Level: 58% STEADY
- Tavg: 588.9 °F and STABLE

Tier #

Group #

10 CFR 55.43(b)

- Seal Injection Flow: 8 gpm per pump
- Seal Leakoff Flow: 3 gpm per pump

Which ONE of the following is the approximate steam generator leakrate?

A.	65 gpm								
В.	55 gpm								
C.	75 gpm								
D.	43 gpm								
Answe	r: A	Lessor	n Plan:						
Refere	ence:								
Questi	on Source:		Bank # Modifie New	d Bank	# _ -	991			
Questi	on Cognitive	Level:		Memory Compre	or Fun	damental n or Analy	Knowledge vsis	°X	
Examir	nation Outline	e Cross-refere	ence:						
	L P I	₋evel ⟨/A # mportance Ra	ating	<u> </u> 	RO EPE037 3.5	′G2.4.46		<u>SRO</u> EPE037G2. 3.6	4.46

1

2

1

2

Question # 59

Procedure 0POP05-EO-EO30, "Steam Generator Tube Rupture," is being performed in response to a tube rupture on SG B in Unit 2. The cooldown has just been completed but the target temperature value selected by the Unit Supervisor was HIGHER than that stipulated in the procedure.

This error could result in which ONE of the following conditions?

- A. Loss of RCS subcooling before RCS and ruptured SG pressures are equalized.
- B. Increasing pressure in the ruptured SG with resultant lifting of a SG B Safety Valve.
- C. Decreasing time for termination of the primary to secondary leakage.
- D. Filling the pressurizer solid during the subsequent depressurization.

Answer: A Lesson Plan:

Reference: Procedure 0POP05-EO-EO30, "Steam Generator Tube Rupture," and WOG ERG E-3, "Steam Generator Tube Rupture," Step 14

Question Source:	Bank #	226
	Modified Bank #	
	New	

Question Cognitive Level:	Memory or Fundamental Knowledge	
-	Comprehension or Analysis	X

Level	RO	SRO
K/A #	EPE038EA1.36	EPE038EA1.36
Importance Rating	4.3	4.5
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 60

Unit 1 is at 100% power when a leak develops in the pressurizer reference leg. Which one of the following is an indication that this failure has occurred?

- Pressurizer pressure begins to rise and spray valves open to modulate pressure; Backup heaters come on; and PRZR LEVEL DEV HI alarm actuates.
- B. Pressurizer pressure begins to rise and spray valves open to modulate pressure; Charging flow control valve closes; and PRZR LEVEL DEV HI alarm actuates.
- C. Pressurizer pressure begins to decrease; Backup and control group heater denergize; and, PRZR LEVEL DEV LO alarm actuates.
- D. Pressurizer pressure begins to decrease; Backup and control group heater denergize; and, Charging flow control valve closes.

Answer: B Lesson Plan:

Reference: LOT201.14.01 and LOT201.14

Question Source:

Bank # ____ Modified Bank # ____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

_X__

Level	RO	SRO
K/A #	APE028AK1.01	APE028AK1.01
Importance Rating	2.8	3.1
Tier #	1	1
Group #	3	3
10 CFR 55.43(b)		

Question # 61

Due to a breaker malfunction, MOV-CV0032B fails shut and isolates the corresponding Reactor Coolant Pump Seal Injection. Which statement below describes the conditions and actions that will occur due to this event.

- A. "RCP 1B(2B) THERM BAR CCW FLOW/TEMP/TRBL" Annunciation when the CCW flow rate increases to greater than 60 gpm the thermal barrier heat exchanger, and MOV-0374 closes to isolate the CCW from thermal barrier due to the high flow.
- B. "RCP 1B(2B) THERM BAR CCW FLOW/TEMP/TRBL" Annunciation when the CCW thermal barrier temperature increase above 187°F, and MOV-0374 closes to isolate the CCW from thermal barrier due to the high temperature
- C. "RCP 1B(2B) SEAL WTR INJ FLOW LO" Annunciator, while the CCW thermal barrier temperature increases.
- D. "RCP 1B(2B) SEAL WTR INJ FLOW LO" Annunciator, while the CCW thermal barrier flow rate decreases.

Answer: C Lesson Plan: LOT201.05.ho

Reference: 0POP09-AN-05M2, 0POP09-AN-04M7

Question Source:

Bank # _____ Modified Bank # _____ New X

Question Cognitive Level:

Memory or Fundamental Knowledge

Level	RO	SRO
K/A #	000022K3.06	
Importance Rating	3.2	3.3
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 62

Unit2 has the following conditions:

- Reactor is in Mode 5
- Mid-Loop Operations in effect
- Annunciator "RC MID LOOP LVL LO-LO" has been recieved
- RCS NR Hot Leg Level is at -3 inches
- RHR pump 1A, and 2A are exibiting erratic current and discharge flow indication

Which ONE of the following statements indicate the correct actions which should be taken?

- A. Increase RCS level using CCP or RWST gravity fill through the idle train LHSI pump and cold leg injection valve via 0POP09-AN-01M2 Annunciator Response Instruction.
- B. Stop all RHR pumps, Isolate letdown and known drain paths, and Initiate containment closure and evacuation of all non-essential personnel via 0POP04-RH-0001, "Loss of Residual Heat Removal" due to core possibly becoming uncovered.
- C. Reduce RHR flow to regain operation within the the acceptable band according to 0POP04-RH-0001---RCS Level versus RHR Pump Flow diagram in order not to damage the RHR pumps and introdue air into the RCS.
- D. Stop all RHR pumps, Isolate letdown and known drain paths, and Initiate RCS refil using CCP and RWST via 0POP04-RH-0001 "Loss of Residual Heat Removal."
- Answer: B Lesson Plan: LOT201.09HO.01, rev 8
- Reference: 0POP04-RH-0001 "Loss of Residual Heat Removal" 0POP09-AN-01M2 Annunciator Response Instruction

Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Level:	Memory or Fundan	nental Knowledge	

Х

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	000025K3.03	
Importance Rating	3.9	4.1
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Comprehension or Analysis

Question # 63

Which ONE of the following statements is correct concerning the Class 1E 125 VDC elctrical power?

- A. During Mode 5 only, on the loss of all DC power, the AC sources supply the battery chargers, which supplies power to the DC bus loads and provides power to recharge the batteries.
- B. During all modes of operation, the loss of a single battery charger for any single Train will cause that corresponding battery to supply the DC Bus loads for a minimum of 2 hours.
- C. During Mode1 only, on the loss of all AC, the battery chargers fail and the DC Bus is supplied by the battery for a minimum of 2 hours.
- D. During all modes of operation, on a loss of all AC, battery chargers lose power, the EDGs start and the 1E load centers sequence on, then the chargers resume powering the DC busses.

Answer:	D	Lesson Plan:	LOT201.37.HO.1, rev 6
---------	---	--------------	-----------------------

Reference: 0POP02-EE-0001, "ESF DC Distribution System"

Question Source:

Bank # _____ Modified Bank # _____ New X

Question Cognitive Level:

Memory or Fundamental Knowledge _____ Comprehension or Analysis

_X___

Level	RO	SRO
K/A #	000058K1.01	
Importance Rating	2.8	3.1
Tier #	1	1
Group #	2	2
10 CFR 55.43(b)		

Question # 64

The following conditions exist on Unit 2:

- SI initiation just occurred
- Undervoltage on all ESF 4.16kV buses
- EDG 1: Voltage= 4250 V, Speed= 650 rpm, Lube oil pressure= 35
- EDG 2: Voltage= 3740 V, Speed= 620 rpm, Lube oil pressure= 32

Which ONE of the following statements is correctly states what occurs?

- A. ESF Load Sequencer in Mode II; HHSI pump 1A, LHSI pump 1A, Containment spray pump 1A loaded to the Train A Bus.
- B. ESF Load Sequencer in Mode III; CCW pump 1A, Aux Feed Water pump 11, Containment spray pump 1A are NOT loaded to the Train A Bus
- C. ESF Load Sequencer in Mode II; HHSI pump 1B, LHSI pump 1B, Containment spray pump 1B loaded to the Train B Bus.
- D. ESF Sequencer in Mode III; CCW pump 1B, Aux Feed Water pump 12, Containment spray pump 1B are NOT loaded to the Train A Bus

Answer: D	Lesson Plan:	LOT201.41HO.1, Rev.5
-----------	--------------	----------------------

Reference: 0PSP03-DG-0001, "Standby Diesel 11(21) Operability test

 Question Source:
 Bank #

Modified Bank #

New

Level	RO	SRO
K/A #	064K3.01	
Importance Rating	3.8	4.1
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question #65

During an electrical maintenance activity on Unit 2, the "125 VDC SYSTEM E2A11 TRBL" annunciator is actuated. After some investigation, it has been determined that the system will NOT be operable in the near future.

Which of the following statements is correct concerning the current situation?

- A. Remove loads from the 125VDC Bus E2A11, cross-connect the bus to the 125 VDC 1A bus , then reload the bus according to 0POP02-EE-0001, "ESF DC Distribution."
- B. Remove loads from the 125VDC Bus E2A11, swap loads to one of the ESF Buses with DC control power available, when Bus E2A11 is available then load the bus according to 0POP02-EE-0001, "ESF DC Distribution."
- C. Remove loads from the 125VDC Bus E2A11, start an emergency diesel generator to energize the bus, then load the bus according to 0POP02-EE-0001, "ESF DC Distribution."
- D. Remove loads from the 125VDC Bus E2A11, cross-connect the bus to the 125 VDC 1B bus , then reload the bus according to 0POP02-EE-0001, "ESF DC Distribution."

Answer:	В	Lesson Plan:	LOT201.37.HO, Rev.6
			LOT201.33.HO.01

Reference: 0POP04-DJ-0001 Rev 9, "Loss of class1E 125 VDC Power", Addendum 1, pg. 13

Question Source:	Bank #	
	Modified Bank #	
	New	X

 Question Cognitive Level:
 Memory or Fundamental Knowledge

 Comprehension or Analysis

Level	RO	SRO
K/A #	063K4.02	
Importance Rating	2.9	3.2
Tier #	2	2
Group #	2	1
10 CFR 55.43(b)		

Question #66

The following conditions exist on Unit 2 following a refueling outage:

- Start up in progress, Reactor at 22%, RCS pressure at 2235psig, RCS temp at 572.0°F
- Three control rods drop into the core (all in close proximity)
- Power decreases to 15% on one power range monitor, 19% on the other three channels

Which ONE of the following statements is correct concerning the actions that will occur or be required given the Unit 2 conditions?

- A. Manually trip the reactor, manually trip the turbine, remove heat through steam dumps to bring RCS temperature to 567°F in the Steam Pressure Mode.
- B. Automatic reactor trip, automatic trip of the turbine due to the P-10 permissive, remove heat through the steam dumps to bring RCS temperature to 567°F in the Tavg Mode.
- C. Manually trip the reactor, the turbine does not trip, remove RCS heat through the turbine and condenser to 567°F.
- D. Automatic reactor trip, do NOT trip the turbine, remove RCS heat through the turbine and condenser to 567°F.

Answer:	С	Lesson Plan:	LOT201.20.HO.01, Rev. 13
			LOT202.09.LP Rev. 11
			LOT201.20.HO, Rev. 13

Reference:

Question Source:	Bank #	
	Modified Bank #	
	New	X

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

_X__

Level	RO	SRO
K/A #	045K5.18	
Importance Rating	2.7	3.2
Tier #	2	2
Group #	3	3
10 CFR 55.43(b)		

Question # 67

The following conditions exist on Unit 1:

- Reactor power at 25% and decreasing, RCS Temp is 573.25°F and Press is 2235psig
- Control rods in MANUAL
- Steam dump system is in Tavg Mode

While in this condition the turbine trips off line.

Concerning the steam dump system, which one of the following statements correctly describes the system and actions under the these conditions?

- A. C-7 Load Reject signal is received which allow the Load Reject Controller to control the steam dumps, Transfer to Steam Pressure Mode as per 0POP04-TM-003, "Turbine Trip Below P-9" and maintain RCS temperature within 1.5°F of Program Tavg.
- B. C-8 Turbine Trip signal received which allows the Turbine Trip Controller to control the steam dumps, Transfer to Steam Pressure Mode as per 0POP04-TM-003, "Turbine Trip Below P-9" and maintain RCS temperature within 1.5°F of Program Tavg.
- C. C-8 Turbine Trip signal received which allows the Turbine Trip Controller to control the steam dumps, RCS temperature will be decreased to the No-Load setpoint of 567°F, Transfer to Steam Pressure Mode as per 0POP04-TM-003, "Turbine Trip Below P-9" and maintain RCS temperature at 567°F.
- D. C-7 Load Reject signal is received which allow the Load Reject Controller to control the steam dumps, RCS temperature will be decreased according to the difference in Tref and Tavg, Transfer to Steam Pressure Mode as per 0POP04-TM-003, "Turbine Trip Below P-9" and maintain RCS temperature at 567°F.

Answer: B Lesson Plan: LOT202.09.HO.01 Rev. 11, pg 16 LOT202.09. Rev. 11, pg 26-31

Reference: 0POP04-TM-0003, "Turbine trip below P-9"

Question Source:	Bank #	
	Modified Bank #	
	New	X

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

__X___

Level	RO-ONLY	SRO
K/A #	041K4.16	
Importance Rating	2.6	
Tier #	2	
Group #	3	
10 CFR 55.43(b)		

Question #68

Answer:

D

In accordance with 0POP08-FH-0009, "Core Refueling", which ONE of the following conditions would require that core alterations be immediately suspended and who would approve commencement of core alterations?

- A. RHR loop inlet temperature decreased by 8°F between successive readings; Unit Operations Manager
- B. Unexplained decrease in boron concentration of 20ppm; Core Loading Supervisor
- C. While placing a new fuel assembly into the core, the count rate on both source range detectors increases by a factor of 2.4; Unit Operations Manager
- D. Two trains of Standby Diesel Generators inoperable; Core Loading Supervisor

Reference: 0POP08-FH-0009, "Core Refueling", pg. 2-4 T.S. 3.8.1.2, "AC Sources--Shutdown"

Lesson Plan:

 Question Source:
 Bank #

 Modified Bank #

 New

Question Cognitive Level:	Memory or Fundamental Knowledge	X
	Comprehension or Analysis	

Level	RO	SRO
K/A #	034G2.2.22	
Importance Rating	3.4	4.1
Tier #	2	2
Group # 10 CFR 55.43(b)	3	2

Question # 69

Unit 1 is operating at 100% with no LCOs. Due to an unplanned maintenance activity, all MSIVs have closed shut. Concerning just the steam generators, what will occur without operator action?

- A. Rapid pressure increase causes steam generator levels to increase, steam generator PORVs lift relieving the pressure, steam generator levels decrease and feed water level control increases feed to regain level.
- B. Rapid pressure increase causes steam generator levels to increase, steam generator PORVs lift relieving the pressure, steam generator levels decrease and aux. feed water will feed to regain level.
- C. Rapid pressure increase causes steam generator levels to decrease, steam generator safety relief valves lift to relieve the pressure, steam generator levels decrease and aux. feed water will feed to regain level.
- D. Rapid pressure increase causes steam generator levels to decrease, steam generator PORVs lift relieving the pressure, steam generator levels decrease and feed water level control increases feed to regain level.

Answer: C Lesson Plan:

Reference:

Question Source:	Bank # Modifie New	ed Bank #	X	
Question Cognitive Level	:	Memory or Fu Comprehensio	ndamental Knowledge on or Analysis	X
Examination Outline Cros	s-refere	ence:		

Level	RO	SRO
K/A #	035K6.01	
Importance Rating	3.2	3.6
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 70

Unit 1 is operating at 43% power. Two level indicators on one steam generator have failed low. Which one of the following correctly describes the action that will occur barring any operator action?

- A. The "level error output" will dominate the "flow error output" causing the MFRV to open, level will increase and cause the P-14 setpoint to be exceeded, feedwater isolation, main turbine trip and reactor trip will occur.
- B. The "flow error output" will dominate the "level error output" causing the MFRV to close, level will decrease and cause a reactor trip once level falls below 33%, start signal sent to auxiliary feedwater pumps, and main turbine will trip.
- C. The "level error output" will dominate the "flow error output" causing the MFRV to open, level will increase and cause the P-14 setpoint to be exceeded, feedwater isolation and main turbine trip will occur.
- D. The "flow error output" will dominate the "level error output" causing the MFRV to open, level will increase and cause the P-14 setpoint to be exceeded, feedwater isolation, main turbine trip and reactor trip will occur.
- Answer: C Lesson Plan: LOT202.15.HO. Rev. 7, pg. 3,4,7,8

Reference: LOT201.20.HO.01 Rev. 13, pg. 40, 49 LOT202.13.HO.01 Rev. 7, pg. 27-28

Question Source:	Bank #	
	Modified Bank #	
	New	>

Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

_X___

Level	RO	SRO
K/A #	035K1.12	
Importance Rating	3.7	3.9
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question #71

The following has occurred on Unit 1:

- A complete loss of load from 100% power.
- The main turbine did not trip (a direct reactor trip was not generated).
- The reactor tripped on high pressurizer pressure.
- Pressure in the PRT is at 35 psig

Which ONE of the following would be the approximate temperature of the Pressurizer PORV tailpipe following this transient?

Α.	228 °F				
В.	281 °F				
C.	655 °F				
D.	659 °F				
Answe	r:	С	Lesson Plan: LOT2	01.04.HO.01, Rev 7 and Stea	am Tables
Refere	nce:				
Questi	on Sour	rce:	Bank # Modified Bank # New	X	
Questi	on Cogr	nitive Level:	Memory or F Comprehens	undamental Knowledge ion or Analysis	X
Examir	nation C	Outline Cross-r	eference:		
		Level K/A #	tance Rating	RO 010K5.02 2.6	SRO
		Tier #		2	2
		Group 10 CF	o # R 55.43(b)	2	2

Question #72

The following conditions exist on Unit 2:

- Reactor at 75% power
- In a 14-day LCO for T.S. 3.8.1.1.b---SBDG 22 inoperable
- "SG 2A LVL LO-LO ALERT" annunciator has actuated
- ESF Auxiliary transformers E2C, and E2B have failed

Given the above conditions, which one of the following statements correctly states the resulting condition of the unit?

- A. The motor-driven AFW pump #23 will NOT start and the corresponding S/G will not receive water.
- B. The motor-driven AFW pump #22 will NOT start and the corresponding S/G will not receive water.
- C. The motor-driven AFW pump #21 will NOT start and the corresponding S/G will not receive water.
- D. All motor driven AFW pumps will start, due in part to SBDG actuation, and all S/G will receive water.

Answer: B Lesson Plan: LOT202.28.HO.01 Rev. 8, pg 7 LOT201.36.HO.01 Rev. 4, pg 2-8

Reference: DWG: 0-E-AAAA-01, "Single line diagram, Main one line diagram, Unit 1&2" 0POP09-AN-06M3, Rev. 14, pg. 33,34,41,42

Question Source:	Bank #	
	Modified Bank #	
	New	X

Question Cognitive Level:Memory or Fundamental Knowledge
Comprehension or Analysis

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	061K2.02	
Importance Rating	3.7	3.7
Tier #	2	2
Group #	1	1
10 CFR 55.43(b)		

Х
Question #73

Given the following:

A spent fuel assembly is being raised from its slot in the storage pool for return to the reactor. Gas bubbles are coming to the surface of the pool. Radiation levels in the Spent Fuel Pool area are increasing.

Which ONE of the following actions is required of personnel in the Fuel Handling Building?

- A. Notify the control room to sound the Containment Evacuation Alarm.
- B. Immediately evacuate all personnel from the Fuel Handling Building.
- C. Notify the Core Loading Supervisor, then replace the damaged fuel assembly in its storage location.
- D. Move the fuel assembly into the RCB and notify the control room to initiate containment isolation.

Answer:	В	Lesson Plan: STP: OPOP04-FH-0001, p. 2, 6.
		STP: LOT504.02, LO# 3.

Reference:

Question Source:

Bank # ____X___ Modified Bank # _____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

Х

Level	RO	SRO
K/A #	000036K202	
Importance Rating	3.4	3.9
Tier #	1	1
Group #	3	3
10 CFR 55.43(b)		

Question #74

The plant is operating at 100% power. The following conditions exist:

-Xenon is at equilibrium. -Group D rod position is 225 steps. -Rods are in manual.

A downpower to 50% is commenced. Boration is in progress and T avg. remains in accordance with program. With NO ROD MOTION, which one of the following is the expected response of Axial Flux Difference (AFD) to the downpower?

- Α. Becomes more positive.
- Β. Becomes more negative.
- C. Does not change.
- D. Deviates between channels.
- Answer: Α Lesson Plan:

Reference:

Que	estion	Sol	irce:			Bank # Modifie New	≠ ed Ba	ank	#	_	X		
~		~						_					

Memory or Fundamental Knowledge Question Cognitive Level: Comprehension or Analysis

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	001A3.03	
Importance Rating	3.6	3.8
Tier #	2	2
Group #	1	1
10 CFR 55.43(b)		

Х

Question #75

Which one of the following describes the method used at STP to check for Estimated Critical Condition inaccuracies during a unit startup in accordance with 0POP03-ZG-0004 "Reactor Startup"?

- A. The count rate is monitored at a MAXIMUM of every 50 steps.
- B. The startup rate is maintained GREATER THAN 0.5 dpm when pulling rods.
- C. The RCS temperature is monitored every 30 minutes to be GREATER THAN 561 degrees F until criticality is achieved.
- D. All control bank positions are monitored during control bank movement to ensure that the step counters and DRPI agree within a MAXIMUM of 6 steps.

Answer: A Lesson Plan: LOT506.01 LOT506.03

Reference:

Question Source:

	Modified Bank #X	
Question Cognitive Level:	Memory or Fundamental Knowled Comprehension or Analysis	dgeX

Rank #

Level	RO-ONLY	SRO
K/A #	001A4.10	
Importance Rating	3.5	
Tier #	2	
Group #	1	
10 CFR 55.43(b)		

Question # 76

Which ONE of the following provides the power source for the train B Engineered Safety Features load sequencer?

- A. 120Vac Nonvital Instrument bus DP004.
- B. 125Vdc Vital switchboard E1B11.
- C. 48Vdc master relay power supply.
- D. 120Vac Vital Instrument bus DP006.

10 CFR 55.43(b)

Answer:	В	Lesson Plan:	LOT201.37	Class 1E 125 vdc	
Reference:					
Question Sou	urce:	Bank ; Modifi New	# ed Bank #	 x	
Question Co	gnitive Level:	Memo Comp	ory or Fundame rehension or A	ental Knowledge Analysis	X
Examination	Outline Cross	-reference:			
	Leve K/A ; Impo Tier ; Grou	el # prtance Rating # up #	<u>RO-C</u> 013K 3.6 2 1	DNLY 2.01	SRO

Question #77

A loss of offsite power in conjunction with a small break Loss of Coolant Accident has occurred, resulting in a reactor trip. All three Emergency Diesel Generators start and sequence on to the vital buses, restoring vital power. A subsequent Safety Injection signal is received and all of the Reactor Containment Fan Coolers are shed as expected. However, due to a common mode problem within the sequencer circuitry, none of the Reactor Containment Fan Coolers are sequenced back on the vital bus.

Given the following containment history:

Time	Ctmt Temp.	Ctmt Press	Ctmt. Humidity	Ctmt Radiation
0815	178 Deg F	2 psig	90%	9.0 X 10E2 R/Hr
0830	180 Deg F	4 psig	100%	7.3 X 10E3 R/Hr
0845	183 Deg F	6 psig	100%	9.5 X 10E4 R/Hr
0900	185 Deg F	10 psig	100%	2.0 X 10E5 R/Hr

Which ONE of the following describes the EARLIEST time at which adverse containment should have been declared?

- A. 0815
- B. 0830
- C. 0845
- D. 0900

Answer: C Lesson Plan:

Reference:

Question Source:

Bank # _____X____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis



Level	RO-ONLY	SRO
K/A #	022K3.02	
Importance Rating	3.0	
Tier #	2	
Group #	1	
10 CFR 55.43(b)		

Question # 78

Under what condition is entry into OPOP05-EO-FRZ2, "Response to Containment Flooding," warranted?

- A. Containment water level still below 69 inches.
- B. Containment Radiation is greater than 2.0 + 03E R/HR.
- C. Containment Critical Safety function tree is in an Orange Condition.
- D. Containment Pressure is greater than 9.5 psig.

Answer: C Lesson Plan:

Reference: OPOP05-EO-FRZ2

Question Source:

Bank # ____X___ Modified Bank # _____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

Х

Level	RO	SRO
K/A #	W/E 15 A2.02	
Importance Rating	2.9	3.3
Tier #	1	1
Group #	3	3
10 CFR 55.43(b)		

Question # 79

A reactor trip occurs after 100 days of full power operation. (Initiated when both 13.8kV buses trip on faults.)

Which ONE of the following is a positive indication that natural circulation heat removal is occurring? Assume thirty minutes have elapsed since the trip.

- A. Core exit thermocouples are decreasing.
- B. Narrow range T HOT temperature is decreasing.

Group #

10 CFR 55.43(b)

- C. Reactor coolant system subcooling is 10 degrees F and decreasing
- D. T SAT for steam pressure is falling faster than primary temperatures.

Answer:	A	Lesson Plan:		
Reference:				
Question Sou	rce:	Bank # Modified Bank # New	x	
Question Cog	nitive Level:	Memory or Comprehen	Fundamental Knowledge sion or Analysis	X
Examination (Outline Cross-r	eference:		
	Lev K/A Imp	vel # portance Rating	<u>RO-ONLY</u> 017K3.01 3.5	SRO
	lie	r #	2	

1

Question # 80

The Incore Thermocouple output provides input to the QDPS where the signals are processed and used for:

- A. information display and feedback to the reference T/C junction box.
- B. information display and reference temperature for the COMS.
- C. information display and temperature compensation of RVWL.
- D. information display only.

Answer: D Lesson Plan: LOT201.17.HO, rev. 2

Reference:

Question Source:

Bank # Modified Bank # New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

Х

Examination Outline Cross-reference:

Level	RO		SRO
K/A #	017A4.01		
Importance Rating	3.8	4.1	
Tier #	2		2
Group #	1	1	
10 CFR 55.43(b)			

Х

Question #81

Given the following:

- Unit 1 is in Mode 6 with 50 assemblies off-loaded to the Spent Fuel Pool
- Refueling operations have been suspended due to a stuck fuel assembly in the reactor vessel core region.
- While preparing to free the stuck assembly, the Core Load Supervisor determines that the HOIST RAISE, +OVTL, ENG O/L (S-17) interlock bypass switch must be placed in the ON position.

In accordance with 0POP08-FH-0001, Refueling Machine Operating Instruction, which of the following attachment(s) is/are REQUIRED to be used by the Core Load Supervisor under these circumstances?

- Α. Form 1, Stuck Fuel Assembly Removal Instructions
- Β. Form 2, Bypass Mode Operation Justification and Authorization
- C. Form 1, Stuck Fuel Assembly Removal Instructions AND Form 2, Bypass Mode Operation Justification and Authorization
- D. Form 1, Stuck Fuel Assembly Removal Instructions OR Form 2, Bypass Mode **Operation Justification and Authorization**

Answer:	А	Lesson	Plan:
Answer.	~	LESSON	Fian.

Reference: 0POP08-FH-0001

Question Source:	Bank #	X-593
	Modified Bank #	
	New	

Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

Examination Outline Cross-reference:

Level K/A # Importance Rating Tier # Group # 10 CFR 55.43(b)

RO-ONLY SRO G2.2.30 3.5

3

Question # 82

Which ONE of the statements below most correctly completes the following concerning RCP seal injection? The #1 seal bypass valve may be used to provide additional cooling flow to:

- A. the radial bearing when RCS pressure is below 1000 psig.
- B. the #1 seal during transients at normal temperature and pressure.
- C. the #2 and #3 seals if #1 seal flow is inadequate.
- D. the #1 seal with a containment phase A isolation signal present.

Answer: A Lesson Plan:

Reference:

Question Source:

Bank # _____ Modified Bank # _____ New

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

__X__

Examination Outline Cross-reference:

Level K/A # Importance Rating Tier # Group # 10 CFR 55.43(b)

<u>RO-ONLY</u>	SRO
003K6.02	
2.7	
2	
1	

Question #83

The following conditions exist on Unit 2:

- Reactor power is at 35% at EOL (end of core life)
- All systems in automatic
- No significant LCOs or maintenance in progress
- Main turbine trips off line, Main condenser is available

Steam dump system initially operates as intended, steam dump valves modulate open to match Tref and Tavg for the power given. Shortly after that all dump valves modulate to the full open position.

Which one of the following statements correctly describes what happens to the plant without operator assistance?

- A. Steam generator levels immediately decrease, RCS temperature will initially decrease, reactor power increases due to moderator temperature coefficient, pressurizer level decreases, reactor trips.
- B. Steam generator levels immediately increase, RCS temperature will initially increase, reactor power decreases due to total power defect, pressurizer level decreases, reactor trips.
- C. Steam generator levels immediately increase, RCS temperature will initially decrease, reactor power increases due to moderator temperature coefficient, pressurizer level initially decreases, reactor power stabilizes at approximately 40%.
- D. Steam generator levels immediately decrease, RCS temperature will initially increase, reactor power initially decreases due to total power defect, pressurizer level initially decreases, reactor power stabilizes at approximately 40%.

Answer: C Lesson Plan:

Reference: Total power defect curve (EOL) Moderator temperature coefficient curve (according to PPM of Boron)

Question Source:	Bank # Modified Bank # New	X

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

_X__

Level	RO	SRO
K/A #	039K5.08	
Importance Rating	3.6	3.6
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 84

Given the following Unit 1 conditions:

- A reactor startup is in progress
- Shutdown Bank A is fully withdrawn
- Withdrawing Shutdown Bank B (DRPI indication is 234 steps)

Which ONE of the following correctly identifies the expected condition of the indicated annunciator?

- A. ROD CONT URGENT ALARM window is ILLUMINATED.
- B. PR CHANNEL DEV window is ILLUMINATED.

Group #

10 CFR 55.43(b)

- C. ROD BOTTOM window is CLEAR.
- D. SR SHUTDN FLUX HI ALM BLKD window is CLEAR.

Answer:	D	Lesson I	Plan:				
Reference:	0POP03-ZG-(0004					
Question Sou	rce:	E N N	3ank # ∕lodified Bank New	# _	X		
Question Cog	nitive Level:	N C	Memory or Fur Comprehensio	ndament on or Ana	tal Knowledge alysis)	X
Examination C	Outline Cross-r	eference:					
	Level K/A # Import Tier #	ance Rat	ing	<u>RO</u> (001)G2 3.5 3	2.4.46		<u>SRO</u>

Question #85

The following conditions exist on Unit 1:

- Reactor at 100%
- NI-0042 is tagged out---LCO 3.3.1 entered
- All systems are in automatic

Which of the following statements correctly states what will happen if there is a Loss of 120VAC to distribution panel DP1202?

- A. Reactor trip, "SSPS TROUBLE ALARM TRN R(S)" alarm annunciator actuation, power is lost to one of the 48 VDC and 15 VDC power supplies for logic train S on SSPS.
- B. Reactor does NOT trip, "SSPS TROUBLE ALARM TRN R(S)" alarm annunciator actuation, power is lost to one of the 48 VDC and 15 VDC power supplies for logic train R on SSPS.
- C. Reactor trip, "SSPS DC PWR LOSS TRN R(S)" alarm annunciator actuation, power is lost to one of the 48 VDC and 15 VDC power supplies for logic train R on SSPS.
- D. Reactor does NOT trip, "SSPS TROUBLE ALARM TRN R(S)" alarm annunciator actuation, power is lost to one of the 48 VDC and 15 VDC power supplies for logic train S on SSPS.

Answer: A Lesson Plan: LOT201.20.HO.01 Rev 13, pg. 17 LOT201.20.HO.02 Rev 13, pg. 10 2POP09-AN-03M2 Rev 10, pg. 16

Reference:

Question Source:	Bank # Modified Bank # New	X	
Question Cognitive Level:	Memory or Fundan Comprehension or	nental Knowledge Analysis	_>
Examination Outline Cross-referer	ice:		

10 CFR 55.43(b)

LevelROK/A #012K2.01Importance Rating3.3Tier #2Group #2

SRO

3.7

Question #86

Given the following conditions:

- Pressurizer (Pzr) level control selector switch is in the L465/467 position •
- Pressurizer level control is in automatic. ٠
- The REFERENCE LEG for LT-465 develops a small leak

Which ONE of the following correctly describes the INITIAL instrument and plant response?

	LI-465 (Pzr level)	LI-467 (Pzr level)
A.	Increasing	Increasing
В.	Increasing	Decreasing
C.	Decreasing	Increasing
D.	Decreasing	Decreasing

Lesson Plan: LOT201.14, obj#04416 Answer: В

Reference: LOT201.14

estion S Q

uestion Source:	Bank # Modified Bank #	
	New	<u>X</u>

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

Х

Level	RO	SRO
K/A #	016A3.02	
Importance Rating	2.9	2.9
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 87

Given the following:

- An RCS break has occurred.
- The crew is currently in 0POP05-EO-ES12, Post LOCA Cooldown and Depressurization, attempting to isolate SI Accumulators.
- It is discovered that SI Accumulator 'B' outlet valve, MOV-0039B, will not close.
- All attempts to close the valve electrically have failed.

Which ONE of the following describes how the operators should address the stuck open valve?

- A. Dispatch an operator to close the valve locally.
- B. Continue the cooldown and allow the SI Accumulator to discharge.
- C. Vent the SI Accumulator to the RCB.
- D. Isolate the SI Accumulator by closing the Cold Leg Injection Valve, MOV-0031B

Answer: C Lesson Plan: LOT 504.12, obj. CRO 92172

Reference: 0POP05-EO-ES12, Rev. 8, Step 20d RNO

Question Source:

Bank # _____ Modified Bank # ____ New

Comprehension or Analysis

Question Cognitive Level:

<u>X</u>

Examination Outline Cross-reference:

Level	RO-ONLY	SRO
K/A #	W/E 03K2.01	
Importance Rating	3.6	
Tier #	1	
Group #	2	
10 CFR 55.43(b)		

Memory or Fundamental Knowledge

Х

Question # 88

The Plant is operating at full power with all systems in normal alignment.

• Feedwater Pump Discharge Pressure instrument PT-558 fails off scale high.

Which ONE of the following is the expected INITIAL plant response?

- A. Feedwater pump speed increases.
- B. Feedwater pump speed decreases.
- C. Main Feedwater Regulating Valves open.
- D. Main Feedwater Regulating Valves close.

Answer: B Lesson Plan: LOT505.01, obj# 38635

Reference: LOT202.14, POP04-FW-0002

Question Source:	Bank # Modified Bank #	<u> 650 </u>
	New	

Question Cognitive Level:	Memory or Fundamental Knowledge
-	Comprehension or Analysis

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	059A1.07	
Importance Rating	2.5	
Tier #	2	
Group #	1	
10 CFR 55.43(b)		

Х

Question #89

Unit 1 is operating at 100% power when a Large Break LOCA occurs. Five minutes later, the Primary Operator notices the following:

- ECW Pump 1B is running
- ECW Train 1B Blowdown Isolation Valve is closed
- ECW Train 1B Screen Wash Booster Pump is running
- ECW Pump 1B Discharge Valve indicates intermediate position (red AND green lights lit)
- ECW Trains A and C are operating normally
- The yard watch reports the ECW Pump 1B Discharge Valve is 50% open

Which ONE of the following is true concerning ECW Train 1B?

- A. Safety Injection actuation has blocked the trip of the pump to allow the train to operate. The pump will continue to run even if the discharge valve is partially closed.
- B. Safety Injection Train B was reset prior to the discharge valve reaching full open. The discharge valve will open fully when the control switch is taken to OPEN.
- C. Safety Injection Train B did not actuate. Manually actuating Safety Injection will open the discharge valve fully.
- D. ECW Pump 1B did not receive a start signal from the sequencer. The pump was running prior to the Large Break LOCA.

Answer:	А	Lesson Plan:	LOT201.13, obj# 91193
---------	---	--------------	-----------------------

Reference: Electrical drawings 9E-EW01-01, 9E-EW04-02

Question Source:	Bank #	19
	Modified Bank #	
	New	

Question Cognitive Level:	Memory or Fundamental Knowledge	
-	Comprehension or Analysis	<u>X</u>

Level	RO	SRO
K/A #	000062A1.06	
Importance Rating	2.9	2.9
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question # 90

While performing a liquid waste release, RT-8038, LWPS Monitor, reaches the HIGH alarm setpoint. Which of the following will occur?

Α.	LWPS Waste Monitor Tank Pump TRIPS							
В.	LWPS	Discharge Val	ve, WL	-FV-407	7, goes to	REC	CIRC	
C.	LWPS	Waste Monito	r Tank (Outlet V	alve, WL-F	=V-5	019, goes CLOSED	
D.	LWPS	Surge Tank P	ump TF	RIPS				
Answei	1	В	Lessor	n Plan:	LOT203.1	1, oł	oj# 92083	
Refere	nce:	POP04-RA-00	01					
Questic	on Soui	rce:		Bank # Modifie New	d Bank #		<u> 532 </u> 	
Question Cognitive Level: Memory or Fundamental KnowledgeX Comprehension or Analysis								
Examination Outline Cross-reference:								

Level	RO	SRO
K/A #	000059A2.04	
Importance Rating	3.2	3.5
Tier #	1	1
Group #	2	1
10 CFR 55.43(b)		

Question # 91

Given the following:

- Unit 1 Reactor power is 90%.
- Steam Flow on each SG is 4.1 E6 lbm/hr.
- RCS Tavg is stable at 589.6°F on all 4 loops
- RCS pressure is stable at 2235 psig
- 1C SG Feed Flow is pegged HIGH
- 1C SG Main FW Reg Valve is full OPEN
- 1C SG pressure is STABLE
- 1C SG level is DECREASING.
- Containment Humidity and Pressure are INCREASING

Which ONE of the following events is in progress?

- A. Steam Line Break INSIDE Containment.
- B. Feed Flow Indicator failed HIGH.
- C. Feed Line Break INSIDE Containment.
- D. Main Feed Pump trip.

Answer: C Lesson Plan: LOT501.16, obj# 501165

Reference: LOT501.16

Question Source:	Bank #	786
	Modified Bank # New	

Question Cognitive Level:	Memory or Fundamental Knowledge	_
	Comprehension or Analysis	_

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	059K1.05	
Importance Rating	3.1	3.2
Tier #	2	2
Group #	1	1
10 CFR 55.43(b)		

Х

Question # 92

0POP02-AF-0001, Auxiliary Feedwater, requires the following AFW Pump 14 steam supply line drain valves to be OPEN:

- 1(2)-MD-0928, AFPT TO CNDSR 13 (23) ISOL
- 1(2)-MS-0515, MAIN STEAM DRAIN TO CONDENSER FIRST ISOLATION VALVE
- 1(2)-MS-0516, MAIN STEAM DRAIN TO CONDENSER SECOND ISOLATION VALVE

Failure to have these valves in their required position could result in:

- A. A hydraulic transient or AFW turbine overspeed on startup.
- B. An inadequate AFW turbine exhaust path preventing the turbine from reaching rated speed.
- C. Excessive thermal stresses on startup because AFW turbine temperature is not equalized with Main Steam temperature.
- D. Overpressurization of the AFW turbine casing during standby conditions.

Answer:	А	Lesson Plan:	LOT 202.28,	obj. CRO 43201
---------	---	--------------	-------------	----------------

Reference: 0POP02-AF-0001, Auxiliary Feedwater, Note & Precaution 4.8

Question Source:

Bank # _____ Modified Bank # _____ New >>

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

<u>X</u>

Level	RO	<u>SRO</u>
K/A #	061K1.11	
Importance Rating	2.7	
Tier #	2	
Group #	1	
10 CFR 55.43(b)		

Question # 93

Which ONE of the following describes the release pathway of a Waste Monitor Tank (WMT) to the reservoir?

Waste Monitor Tank (WMT) to the:

- Α. Circulating Water (CW) discharge piping into the outfall structure.
- Open Loop Cooling (OC) discharge piping to Circulating Water (CW) discharge piping Β. into the outfall structure.
- C. Floor Drain Tank (FDT) into the outfall structure.

D. Open Loop Cooling (OC) discharge piping into the outfall structure.

Answer:	В	Lesson Plan: LOT203.11, obj# 92082	
Reference:	P&IDs 9F900	P&IDs 9F90001, 9F00033, 2F00032 sh 2/5	
Question Sou	Irce:	Bank # Modified Bank # NewX	
Question Coo	gnitive Level:	Memory or Fundamental Knowledge Comprehension or Analysis	<u>_X</u>

Level	RO	SRO
K/A #	075K1.02	
Importance Rating	2.9	3.1
Tier #	2	2
Group # 10 CFR 55.43(b)	2	2

Question #94

Unit 1 is in Mode 4 with the following conditions:

- RCS Pressure: 600 psig
- Cold Leg Temperatures: 345°F
- Cold Overpressure Protection (COMS): ARMED

Wide Range Cold Leg temperature element TE-414 fails low resulting in the following alarms:

- ♦ 4M08/E-5 PRZR PORV OPEN COMMAND
- 5M02/B-7 RCS COLD OVERPRESS ALERT - TRN B

Which ONE of the following describes the expected response?

- Α. PORV 655A is open
- Β. PORV 656A is open
- C. Both PORV 655A and 656A are open
- D. Neither PORV 655A or 656A are open
- Answer: В Lesson Plan: LOT201.14, obj# 80409

Reference: POP04-RP-0005

Question Source:

Modified Bank #

Question Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis Х

Bank #

New

Level	RO	SRO
K/A #	006A4.11	
Importance Rating	4.2	4.3
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 95

Given the following:

- A Reactor Trip occurs due to a Loss of Offsite Power
- The ESF Diesel Generators have all started and restored power to their ESF buses
- The Control Room crew has just completed the Immediate Actions of 0POP05-EO-EO00, Reactor Trip or Safety Injection

Which ONE of the following correctly predicts the RCB Temperature trend and the reason for the trend?

- A. Temperature is slowly decreasing (1) because the RCFCs are running with CCW flowing through the cooling coils.
- B. Temperature is slowly increasing (1) because the RCFCs are running with NO cooling flow through the cooling coils.
- C. Temperature is slowly decreasing (1) because the RCFCs are running with Chilled Water flowing through the cooling coils.
- D. Temperature is slowly increasing (1) because the RCFCs are NOT running.

Answer:	В	Lesson Plan:	LOT202.33,	obj# 4967
			,	

Reference: Logics 9-Z-42041, 9-Z-42042, 9-Z-41630

Question Source:	Bank #	
	Modified Bank #	
	New	X

Question Cognitive Level:	Memory or Fundamental Knowledge	
-	Comprehension or Analysis	<u>X</u>

Level	RO	SRO
K/A #	103A1.01	
Importance Rating	3.7	4.1
Tier #	2	2
Group #	3	2
10 CFR 55.43(b)		

Question # 96

Given the following:

- The Control Room has been evacuated
- Auxiliary Feed Pump 14 is in service feeding the 'D' SG
- 'D' SG level is being maintained 14-50% in the Narrow Range

Which of the below correctly represents the reason for maintaining level in this band AND the control location used?

	Reason for maintaining SG level 14 – 50 % NR	Location for control of 'D' SG AFW Reg Valve
Α.	Maintains a heat sink	Locally at the valve
В.	Prevents moisture carryover to the Terry Turbine	Aux Shutdown Panel
C.	Maintains a heat sink	Aux Shutdown Panel
D.	Prevents moisture carryover to the Terry Turbine	Locally at the valve

Answer: C Lesson Plan: LOT 505.01, obj. CRO 92110

Reference: 0POP04-ZO-0001, Rev. 14, Bases step 26

Question Source:	Bank # Modified Bank #	
	New	<u>X</u>

Question Cognitive Level:	Memory or Fundamental Knowledge	<u>X</u>
	Comprehension or Analysis	

Level	RO	SRO
K/A #	000068K3.07	
Importance Rating	4.0	4.3
Tier #	1	1
Group #	1	1
10 CFR 55.43(b)		

Question # 97

Given the following:

- Spent Fuel Pool level is decreasing
- Fuel assemblies are beginning to uncover

The FHB HVAC system will automatically:

- A. Go into EMERGENCY MODE if either RT-8090 or RT-8091, FHB 68' Area Monitors, reach the High Alarm setpoint.
- B. Shutdown completely if either RT-8090 or RT-8091, FHB 68' Area Monitors, reach the High Alarm setpoint.
- C. Go into EMERGENCY MODE if either RT-8035 or RT-8036, FHB Exhaust Monitors, reach the High Alarm setpoint.
- D. Shutdown completely if either RT-8035 or RT-8036, FHB Exhaust Monitors, reach the High Alarm setpoint.

Answer:	С	Lesson Plan:	LOT 202.41, c	obj. CRO 92122	
Reference:	0POP04-RA-0	0001, Rev. 7, A	ddendum 10		
Question Sour	ce:	Bank # Modifie New	ed Bank #	 X	
Question Cogr	nitive Level:	Memor Compr	ry or Fundamer ehension or Ar	ntal Knowledge nalysis	X

Level	RO	SRO
K/A #	033K3.02	
Importance Rating	2.8	3.2
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Question # 98

Unit 1 is at 15% power and commencing to raise turbine load at 10%/hr. Which ONE of the following describes how the temperature of the components cooled by the Closed Loop Auxiliary Cooling Water (CL-ACW) System will be controlled?

- A. The Open Loop Auxiliary Cooling Water (OL-ACW) System TCV on the outlet of the SW/FW Heat Exchanger will modulate open to maintain CL-ACW temperature.
- B. The Closed Loop Auxiliary Cooling Water (CL-ACW) System TCV on the outlet of the SW/FW Heat Exchanger will modulate open to maintain CL-ACW temperature.
- C. The individual component TCVs will modulate open to maintain component temperature.
- D. The TGB Watch will manually throttle CL-ACW from the SW/FW Heat Exchanger to maintain CL-ACW temperature.

Answer:	С	Lessor	n Plan:	LOT202.2	4, ot	oj# T20224	
Reference:	P&ID 9F0003	4					
Question Sou	rce:		Bank # Modifie New	d Bank #		X	
Question Cog	nitive Level:		Memor Compre	y or Funda ehension c	imen or An	tal Knowledge alysis	 <u>X</u>

Level	RO-ONLY	SRO
K/A #	076A1.0	
Importance Rating	2.6	
Tier #	2	
Group #	3	
10 CFR 55.43(b)		

Question # 99

Which ONE of the following pieces of equipment/indications is used to detect "identified leakage?"

- Α. Pressurizer relief tank
- Β. Reactor coolant drain tank temperature
- C. Containment sumps
- D. Reactor coolant drain tank level

Answer: D Lesson Plan:

Reference: UFSAR 5.2.5.2 "Leak Detection Methods" pg. 5.2-27, 28

New

Question	Source:
Question	oource.

Bank # Modified Bank #

Memory or Fundamental Knowledge

Х

Question Cognitive Level:

Examination Outline Cross-reference:

Level	RO-ONLY	SRO
K/A #	002K4.05	
Importance Rating	3.8	4.2
Tier #	2	2
Group #	2	2
10 CFR 55.43(b)		

Comprehension or Analysis

Question # 100

Unit 1 is conducting a reactor startup per 0PGP03-ZG-0004, Reactor Startup, with the following conditions:

- Rod Control is selected to MANUAL
- Full Out Position (FOP) is 249 steps
- Bank overlap is 112 steps
- Control Bank "B" is 173 steps withdrawn

Assuming the rod control and position indicating systems are operating properly, which ONE of the following is the expected step counter position for Control Bank "C"?

- A. 23 steps
- B. 36 steps
- C. 49 steps
- D. 62 steps
- Answer: B Lesson Plan: LOT201.18, obj# 3160

Reference: LOT201.18

 Question Source:
 Bank #

Modified Bank #

New
 X

Question Cognitive Level:Memory or Fundamental Knowledge
Comprehension or Analysis

Examination Outline Cross-reference:

Level	RO	SRO
K/A #	014A4.01	
Importance Rating	3.3	3.1
Tier #	2	2
Group #	2	1
10 CFR 55.43(b)		

Х

Question # 101

The following Unit 2 conditions exist:

- The plant is in Mode 6
- CRDM latching is in progress following core reload
- Steam generator secondary manways are removed for sludge lancing
- Steam generator primary side manways are open, with nozzle dams installed, in preparation for Eddy Current testing
- Refueling cavity level is 24 feet above the reactor vessel flange with two trains of RHR operable

Which ONE of the following activities requires a suspension of Core Alterations?

- A. Two trains of the FHB Exhaust System are declared inoperable.
- B. One of the operable trains of RHR cooling is taken out of service to test motor operator valve settings.
- C. Eddy Current testing of the steam generators is commenced.

Lesson Plan

- D. Steam generator safety valves are removed for bench testing of lift setpoint.
- Answer: D

(As available)

TS 3.9.4

Question Source:	Bank # Modified Bank # New	#189

Question Cognitive Level:	Memory or Fundamental Knowledge	
-	Comprehension or Analysis	Χ

Level	<u>R0</u>	SRO
K/A #		2.1.12
Importance Rating		4.0
Tier #		3
Group #		1
10 CFR 55.43(b)(2)		

Question # 102

In accordance with the Technical Specifications, which ONE of the following conditions represents a loss of Primary CONTAINMENT INTEGRITY?

- A. While in MODE 1, an operator enters containment but leaves the inner airlock door OPEN.
- B. While in MODE 3, during an inspection of an equipment hatch, it is determined that the equipment hatch is NOT sealed.
- C. While in MODE 4, Containment internal pressure is found to be 1.6 psig.
- D. While in MODE 5, during performance of the Overall Integrated Containment Leakage Rate Test, containment leakage exceeds the maximum allowable Technical Specification leakage rates.

Answer: B Lesson Plan (As available)

T/S 3.6.1 and the Basis

Question Source:	Bank #	740
	Modified Bank # New	

Question Cognitive Level:	Memory or Fundamental Knowledge	X
-	Comprehension or Analysis	

Level	RO	SRO
K/A #		2.1.33
Importance Rating		4.0
Tier #		3
Group #		1
10 CFR 55.43(b)(2)		

Question # 103

Given the following conditions:

- Troubleshooting under a work package is in progress on Circulating Water Pump 11 to determine why the pump will not start.
- Several leads in the start permissive circuitry for Circulating Water Pump 11 have been lifted and pump start verified during troubleshooting.
- Due to increasing bearing temperature on Circulating Water Pump 14, Plant Management has decided to operate Circulating Water Pump 11 with the lifted leads and secure Circulating Water Pump 14 in order to keep the unit in operation.

Which ONE of the following indicates how Circulating Water Pump 11 would be restored to operation?

- A. Return the pump to operation maintaining the work package open. Continue troubleshooting when the pump can be secured.
- B. Close the existing work package and return the pump to operation. Then write a new work package to continue troubleshooting when the pump can be secured.
- C. Install a temporary modification for the lifted leads, then return the pump to operation.
- D. Return the pump to operation, then approve a design change to incorporate the lifted leads.

Answer: C Lesson Plan (As available)

0PGP03-ZO-0003 Rev 18 step 1.4

Question Source: Bank # 121____ Modified Bank # _____ New

 Question Cognitive Level:
 Memory or Fundamental Knowledge

 Comprehension or Analysis
 X_____

Level	RO	SRO
K/A #		2.2.11
Importance Rating		3.4
Tier #		3
Group #		2
10 CFR 55.43(b)(3)		

Question # 104

- Assuming a steady state reactor-to-secondary generator leakage rate of 150 gpd per S/G, what is basis for limiting the specific activity of the reactor coolant?
- A. Ensures the resulting 2-hour dose at the EXCLUSION AREA will not exceed an appropriately small fraction of the 10 CFR Part 100 dose guideline values following a S/G tube rupture accident.
- B. Ensures the resulting 2-hour dose at the EXCLUSION AREA will not exceed an appropriately small fraction of the 10 CFR Part 20 dose guideline values following a S/G tube rupture accident.
- C. Ensures the resulting 2-hour dose at the SITE BOUNDARY will not exceed an appropriately small fraction of the 10 CFR Part 100 dose guideline values following a S/G tube rupture accident.
- D. Ensures the resulting 2-hour dose at the SITE BOUNDARY will not exceed an appropriately small fraction of the 10 CFR Part 20 dose guideline values following a S/G tube rupture accident.

Answer:	С		Lessor	n Plan	(As available)	
T/S Basis 3.4	1.8					
Question Sou	urce:	Bank : Modified Ban New	# K #	 x		
Question Co	gnitive Le	evel: Memo Comp	ry or Fu rehensio	ndamental Kno on or Analysis	owledge	x
Examination Outline Cross-reference:						
	Level K/A # Importa Tier # Group	ance Rating #		<u>RO</u>		<u>SRO</u> 2.2.25 3.7 3 2

10 CFR 55.43(b)(2)

Question # 105

Unit 2 is in the process of core refueling and the conditions are as follows:

- RHR Train 2A is in service maintaining RCS temperature at 97 °F
- RHR Train 2A loop boron concentration was 2870 ppm on the last sample
- Source Range N-31 indicates 4 cps
- Source Range N-32 indicates 3 cps and is selected for audible count rate indication

In accordance with 0POP08-FH-0009, "Core Refueling," which ONE of the following conditions requires suspension of Core Alterations?

- A. Both Source Range channel indications unexpectedly increase to 7 cps.
- B. Direct communication is lost between the RCB and the FHB Fuel Handling personnel.
- C. RHR Train 1A boron concentration is 2900 ppm on the next successive sample.
- D. The Core Loading Supervisor delegates his duties to the refueling machine operator who is a qualified Reactor Operator.

Answer:	D	Lesson Plan	(As available)		
Procedure 0	P0P08-FH-000	9, "Core Refueling," F	Revision 18		
Question Sc	ource:	Bank # Modified Bank # New	#61		
Question Cognitive Level:		Memory or Fundamental Knowledge X_ Comprehension or Analysis		X	
Examination	Outline Cross-	reference:			
Level K/A # Importance Rating Tier #		RO	SF 2.2 3. ⁻ 3.	<u>RO</u> 2.26 7	
	Group	o #		2	

10 CFR 55.43(b)(7)

Question # 106

Unit 2 was cooling down for a forced outage when a loss of offsite power occurred. All equipment is available for service. The following conditions existed at the time offsite power was lost:

- RCS Temperature: 225 °F
- RCS Pressure: 300 psig
- RHR Trains A and B in service
- SG Pressures: Approximately 5 psig
- SG NR Levels: 55-60%

Which ONE of the following procedures would the crew use to stabilize the unit?

- Α. 0POP04-RH-0001, "Loss of Shutdown Cooling"
- Β. 0POP03-ZG-0007, "Plant Cooldown"
- C. 0POP02-RH-0001, "Residual Heat Removal System Operation"
- 0POP05-EO-EC00, "Loss of All AC Power" D.

Answer:	А	Lesson Plan	(As available)	
Question Sou	rce:	Bank # Modified Bank # New	289	
Question Cog	nitive Level:	Memory or Fundamental Knowledge Comprehension or Analysis		x
Examination Outline Cross reference:				

Level	RO	SRO
K/A #		2.4.9
Importance Rating		3.9
Tier #		3
Group #		4
10 CFR 55.43(b)(5)		

Question # 107

When determining an Emergency Classification from the Fission Product Barrier Degradation Initiating Condition Matrix, which ONE of the following conditions constitutes a LOSS of RCS in accordance with 0ERP01-ZV-IN01, "Emergency Classification?"

- A. Integrity CSFST Red
- B. Core Cooling CSFST Yellow with subcooling < 0°F
- C. SGTR with primary-to-secondary leakrate > capacity of one CCP
- D. SG tube leak > 500 gpd with direct secondary side leak to atmosphere

Answer: B	Lesson Plan	(As available)		
Question Source:	Bank # Modified Bank # New	589 		
Question Cognitive Level Memory or Fundamental Knowledge				

stion Cognitive Level: Memory or Fundamental Knowledge Comprehension or Analysis

Examination Outline Cross-reference:

Level	RO	SRO
K/A #		2.4.41
Importance Rating		4.1
Tier #		3
Group #		4
10 CFR 55.43(b)5		

X____

Question # 108

Unit 2 has experienced the following casualty:

- A Steam Line Break inside containment has occurred.
- 0POP05-EO-EO00, "Reactor Trip or Safety Injection" was entered at Step 1 but the reactor failed to trip.
- The crew then implemented 0POP05-EO-FRS1, "Response to Nuclear Power Generation/ATWS."
- A loss of an ESF bus has occurred requiring local manual operation to isolate AFW to the faulted SG.

At Step 15, "Verify Reactor Subcritical", of FRS1, the following conditions exist:

- Extended Range NIs indicate 0%
- Extended Range NIs SUR is + 0.2 dpm

Which ONE of the following actions is required based on the given conditions?

- A. Return to procedure and step in effect (E000, Step 1).
- B. Continue Emergency Boration and continue actions per FRS1.
- C. Continue Emergency Boration and return to procedure and step in effect (EO00, Step 1).
- D. Continue with FRS1 and implement actions of other Optimal Recovery Guidelines (ORPs) which do not cooldown or add positive reactivity.

Answer:	В	Lesson Plan	(As available)		
0POP05-E	O-FRS1, "Res	ponse to Nuclear	Generation - ATWS," Rev 9		
Question Source: Bank # Modifie New		Bank # Modified Bank # New	735		
Question Cognitive Level:		: Memory o Compreh	or Fundamental Knowledge ension or Analysis	x	
Examination Outline Cross-reference:					
	Level K/A #		RO	<u>SRO</u> 029EA2.01	

4.7

1

1

K/A # Importance Rating Tier # Group # 10 CFR 55.43(b)(5)
Question # 109

Unit 2 has experienced a loss of all AC power and both diesel generators failed to load. What procedure would be used to restore power to the vital buses and how would the restoration of electrical power be accomplished?

- A. Procedure 0POP05-EO-EC00, "Loss of all AC Power," directs you in part to verify power is available from the emergency transformer; disconnect the normal power source from the ESF buses; and, connect the emergency bus to one ESF bus.
- B. Procedure 0POP05-EO-EC00, "Loss of all AC Power," directs you in part to verify power is available from the emergency transformer; disconnect the normal power source from the ESF buses; and, connect the emergency bus to the three ESF buses.
- C. Procedure 0POP05-EO-EC00, "Loss of all AC Power," references Procedure 0POP04-AE-0001, "Loss of any 13.8 KV or 4.16 KV Bus," which directs you in part to verify power is available from the emergency transformer; disconnect the normal power source from the ESF buses; and, connect the emergency bus to one ESF bus.
- D.
- E. Procedure 0POP05-EO-EC00, "Loss of all AC Power," references Procedure 0POP04-AE-0001, "Loss of any 13.8 KV or 4.16 KV Bus," which directs you in part to verify power is available from the emergency transformer; disconnect the normal power source from the ESF buses; and, connect the emergency bus to the three ESF buses.

Answer: A Lesson Plan (As available)

Procedure 0POP05-EO-EC00, "Loss of All AC Power," Rev 11 and Lesson Plan LOT201.36 Obj 92399

Question Source: Bank # Modified Bank # New

k # _____

Question Cognitive Level:

Memory or Fundamental Knowledge Comprehension or Analysis

Level	RO	<u>SRO</u>
K/A #		55EA2.06
Importance Rating		4.1
Tier #		3
Group #		1
10 CFR 55.43(b)5		

Question # 110

Unit 1 is operating at 100% power when an inadvertent safety injection actuation occurs. Five minutes later, the Primary Operator notices the following:

- ECW Pump 1B is running
- ECW Train 1B Blowdown Isolation Valve is closed
- ECW Train 1B Screen Wash Booster Pump is running
- ECW Pump 1B Discharge Valve indicates intermediate position (red AND green lights lit)
- ECW Trains A and C are operating normally
- The yard watch reports the ECW Pump 1B Discharge Valve is 50% open.

Which ONE of the following is true concerning the status of the ECW system?

- A. The safety injection actuation blocked the trip of the pump to allow the train to operate. The pump will continue to run even if deadheaded by a closed discharge valve. T/S 3.7.4 requires that ECW Train 1B be restored within 7 days or be in HOT SHUTDOWN within the next 6 hours.
- B. Safety Injection Train B was reset prior to the discharge valve reaching full open. The discharge valve will open fully when the control switch is taken to OPEN. The three loops of ECW remain operable and T/S 3.7.4 action statement is satisfied.
- C. The safety injection actuation blocked the trip of the pump to allow the train to operate. The pump will continue to run even if deadheaded by a closed discharge valve. T/S 3.7.4 requires that ECW Train 1B be restored within 7 days of be in HOT STANDBY within the next 6 hours.
- D. Safety Injection Train B was reset prior to the discharge valve reaching full open. The discharge valve will open fully when the control switch is taken to OPEN. T/S 3.7.4 requires that ECW Train 1B be restored within 7 days of be in HOT STANDBY within the next 6 hours.

C	Lesson Plan	(As available)	
E-EW01-01 R	ev 8, 9E-EW04-02 R	ev 5???????	
e:	Bank # Modified Bank # New	19	
itive Level:	Memory or Fundame Comprehension or A	ntal Knowledge nalysis	x
utline Cross-re Level K/A # Importa Tier # Group	eference: ance Rating # 2 55 43(b)(2)	<u>RO</u>	<u>SRO</u> APE062AK3.02 3.9 1 1
	C E-EW01-01 R e: itive Level: utline Cross-re Level K/A # Importa Tier # Group 10 CFF	C Lesson Plan E-EW01-01 Rev 8, 9E-EW04-02 R e: Bank # Modified Bank # New itive Level: Memory or Fundame Comprehension or A utline Cross-reference: Level K/A # Importance Rating Tier # Group # 10 CFR 55.43(b)(2)	C Lesson Plan (As available) E-EW01-01 Rev 8, 9E-EW04-02 Rev 5?????? Se: Bank # Modified Bank # 19 New 19 itive Level: Memory or Fundamental Knowledge Comprehension or Analysis utline Cross-reference: RO K/A # Importance Rating Tier # Group # 10 CFR 55.43(b)(2)

Question # 111

Unit 2 has experienced a reactor trip and Safety Injection. While performing 0POP05-EO-EO00, "Reactor Trip or Safety Injection," a fire breaks out in CP-004. The Control Room must be evacuated.

Which ONE of the following must be performed when the Control Room personnel arrive at the Aux Shutdown Panel?

- A. 0POP05-EO-EO00 should be continued where left off, and 0POP04-ZO-0001, "Control Room Evacuation" initiated when 0POP05-EO-EO00 is completed.
- B. 0POP05-EO-EO00 and 0POP04-ZO-0001, "Control Room Evacuation" are to be performed concurrently, to the degree possible.
- C. 0POP04-ZO-0001, "Control Room Evacuation" must be performed exclusively.
- D. 0POP04-ZO-0001, "Control Room Evacuation" is to be performed, unless a CSF Orange or Red condition exists. At which time implementation of the associated FRG is required.

Answer: C	L	esson Plan	(As available)		
Procedure:	0POP-01 Section 7	I-ZA-0018, "Emerg 7.1	ency Operating Proced	ure Use	er's Guide," Rev 14,
Question Sou	ırce: B M N	ank # lodified Bank # lew	1012 		
Question Cog	gnitive Level:	Memory or F Comprehens	undamental Knowledge ion or Analysis	e	x
Examination	Outline Cross-	reference:			
L K Ir T G	evel /A # nportance Rat ïer # Group #	<u>RO</u>		<u>SRO</u> APE06 4.5 1	58AK3.18 1

10 CFR 55.43(b)(5)

Question # 112

You are currently performing procedure 0POP05-EO-EO10, "Loss of Reactor or Secondary Coolant" and Unit 1 plant conditions are as follows:

- RCS pressure = 425 psig
- Pressurizer level = 10%
- Containment pressure = 3.0 psig
- RCS subcooling = 34°F
- Containment radiation levels = 10.5E5 R/hr

Given the above conditions, what actions are required?

- A. Continue with Procedure 0POP05-EO-EO10 and when RWST level decreases below 75,000 gallons, transition to Procedure 0POP05-EO-ES13, "Transfer to Cold Leg Recirculation."
- B. Continue with Procedure 0POP05-EO-EO10 and lineup the plant for hot leg recirculation and then transition to Procedure 0POP05-EO-ES14, "Transfer to Hot Leg Recirculation."
- C. Transition to Procedure 0POP05-EO-ES02, "Natural Circulation and Cooldown."
- D. Transition to Procedure 0POP05-EO-ES12, "Post LOCA Cooldown and Depressurization."

Answer:	D	Lesson Plan	(As available)
---------	---	-------------	----------------

Procedure 0POP05-EO-EO10, "Loss of Reactor or Secondary Coolant," Rev 10

Question Source:	Bank # Modified Bank # New	 X	
Question Cognitive Level:	Memory or Fundamental Knowledge Comprehension or Analysis		x
Examination Outline Cross-	reference:		
Level K/A #	<u>RO</u>		SRO W/E03EA2.02

Importance Rating Tier # Group # 10 CFR 55.43(b)(5) RO SRO W/E038 4.1 1 2

Question # 113

Unit 2 is in a Train B Outage when a reactor trip occurs followed by a loss of offsite power (LOOP). Currently, the operators are implementing Procedure 0POP05-EO-EO00, Reactor Trip and Safety Injection."

Current conditions are:

- Containment Pressure: 5.5 psig
- Containment Radiation: 10.1 R/hr
- Core Exit Thermocouples: 584 °F
- RCS Pressure: 1352 psig
- SG A WR Level: 31%
- SGs B-D WR Levels: 5 8%
- AFW Pump 21 failed to start
- Total AFW flow is approximately 500 gpm

(With adverse containment and AFW flow <576 gpm, this would make the Heat Sink CSF Red - should immediately go to FRH-1 - remove this comment before making exam final)

What actions are required?

- A. Immediately transition to Procedure 0POP-EO-FRH1, "Response to Loss of Secondary Heat Sink," and establish feed and bleed.
- B. Immediately transition to Procedure 0POP-EO-EO10, "Loss of Reactor or Secondary Coolant," and depressurize the intact SGs to 1000 psig.
- C. Immediately transition to Procedure 0POP-EO-FRH1, "Response to Loss of Secondary Heat Sink," dump steam to the condenser to reduce SG pressures.
- D. Perform Procedure 0POP-EO-FRH1, "Response to Loss of Secondary Heat Sink," concurrently with Procedure 0POP05-EO-EO00, Reactor Trip and Safety Injection."

Answer:	A	Lesson Plan	(As available)	
Procedure: Procedure:	0POP-EO-FR 0POP-01-ZA-(H1, "Response to Loss 0018, "Emergency Ope	s of Secondary Heat S erating Procedure Use	ink," and r's Guide," Rev 14
Question Sour	rce:	Bank # Modified Bank # New	 x	
Question Cogi	nitive Level:	Memory or Fundamer Comprehension or Ar	ntal Knowledge nalysis	x
Examination C	Dutline Cross-re Level K/A # Imports Tier # Group 10 CFf	eference: ance Rating # R 55.43(b)(5)	RO	<u>SRO</u> W/E05EK2.2 4.2 1 2

Question # 114

Unit 2 experienced a LOCA and the crew has just transitioned to 0POP05-EO-ES13, "Transfer To Cold Leg Recirculation." The Reactor Operator notes and reports that Train A DID NOT automatically swap suction to the containment sump.

Which ONE of the following indications would justify the failure? (consider each condition separately)

- A. STATUS MONITORING PANEL 1M25: BYP/INOP RWST OUTL MOV-0001A ON
- B. STATUS LAMPBOX 5M2-3: AUTO RECIRC NOT RESET TRAIN A OFF
- C. RWST LVL LI-931 = 70,000 gallons; LI-932 = 73,000 gallons
- D. HHSI Pump 1A MIN FLOW ISOL MOV-0011A OPEN; MOV-0012A CLOSED

Answer:	В	Lesson Plan	(As available)	
References P	&IDs 9-Z-4211	4, 9-Z-42001		
Question Sou	rce:	Bank # Modified Bank # New	123 	
Question Cog	nitive Level:	Memory or Fundamental Knowledge Comprehension or Analysis		x
Examination Outline Cross-reference:				

Level	RO	SRO
K/A #		004A2.13
Importance Rating		3.9
Tier #		2
Group #		1
10 CFR 55.43(b)(5)		

Question # 115

The following conditions exist on Unit 2:

- Reactor power is 65%
- Rod Deviation alarm is lit
- Rod Bottom alarm is lit
- Power Range Channel Deviation alarm is lit
- Rod Bottom LEDs are lit for Shutdown Bank A Rods P4 and D2

Which ONE of the following describes the Technical Specification requirement and the required response to these conditions?

- A. T/S 3.2.4, "Quadrant Power Tilt Ratio," requires that the axial flux difference and quadrant power tilt ratio be checked.
- B. T/S 3.1.3.1, "Moveable Control Assemblies," is applicable and procedures require you to trip the reactor and perform 0POP05-EO-EO00, Reactor Trip or Safety Injection.
- C. T/S 3.1.3, "Moveable Control Assemblies," is applicable and up to two rods are allowed to be restored per 0POP04-RS-0001, "Control Rod Malfunction."
- D. T/S 3.1.3.5, "Shutdown Rod Insertion Limit," is applicable and procedures allow the rods to be restored per 0POP04-RS-0001, "Control Rod Malfunction", then verify operability by performing 0PSP03-RS-0001, "Monthly Control Rod Operability."

Answer:	В	Lesson Plan	(As available)
Procedure 0P0	OP04-RS-0001	, "Control Rod Malfund	ction," Rev 12
Question Sour	ce:	Bank # Modified Bank # New	1008
Question Cogr	nitive Level:	Memory or Fundamental Knowledge Comprehension or Analysis X	

Level	RO	SRO
K/A #		014A2.03
Importance Rating		4.1
Tier #		2
Group #		1
10 CFR 55.43(b)(2) ar	nd (5)	

Question # 116

Unit 1 was at 90% power with the following plant conditions:

- Steam flow on each SG = 4.1E6 lbm/hr
- RCS Tavg on all 4 loops = 587°F
- RCS press = 2235 psig (stable)
- 1C SG feedwater flow is pegged high
- 1C SG main feedwater regulating valve is full open
- 1C SG pressure is stable
- 1C SG level is decreasing

A reactor scram and safety injection actuation occur. The plant conditions at this time are as follows:

- RCS pressure = 2200 psig (slowly decreasing) (make sure this is an accurate condition with a feedline break downstream inside containment - one SG blowing down)
- 1A, 1B, and 1 D SG pressures and levels are stable
- 1C SG pressure and level are decreasing rapidly

Based on the conditions before and after the reactor scram and safety injection, what procedure would Procedure 0POP05-EO-EO00, "Reactor Trip or Safety Injection," direct you to use to address this event and what is the most likely cause of the event?

- A. Procedure 0POP05-EO-EO20, "Faulted Steam Generator Isolation," and feedwater line break inside containment.
- B. Procedure 0POP05-EO-EO20, "Faulted Steam Generator Isolation," and feedwater line break outside containment.
- C. Procedure 0POP05-EO-EO20, "Faulted Steam Generator Isolation," and main steam line break inside containment.
- D. Procedure 0POP05-EO-EO10, "Loss of Reactor or Secondary Coolant," and feedwater line break inside containment.

Answer: A Lesson Plan (As available)

Procedure 0POP05-EO-EO00, "Reactor Trip or Safety Injection"

Question Source:	Bank Modifi New	# ed Bank # A	
Question Cognitive Lev	el:	Memory or Fundamental Knowledge Comprehension or Analysis	x
Examination Outline Cr	oss-refe	rence:	
Leve	I	RO	SRO

Level	RO	<u>SRO</u>
K/A #		059A2.05
Importance Rating		3.4
Tier #		1
Group #		3
10 CFR 55.43(b)(5)		

Question # 117

Unit 2 RCB High Range Area Monitor, RT-8050, is out of service and cannot be repaired for approximately 30 days. What action is required and what is the basis for the action?

- A. Unit 2 would be allowed to operate up to 60 days with 1 inoperable High Range Area Monitor as long as the RCB Low Range Area Monitors remained operational.
- B. Unit 2 would be allowed to operate indefinitely; however, if the inoperable channel could not be restored to operable within 30 days a special report would have to be submitted.
- C. Since this instrumentation is used for determination of routine radiation release permits, Unit 2 would have to shutdown or sample containment using temporary equipment.
- D. Since this instrumentation is used to monitor radiation releases associated with accidents and allows operators to determine actions to mitigate the consequences of the release, Unit 2 would have to shutdown.

Answer:	D	Lesson Plan	(As available)	
T/S 3.3.3.	6, Table 3.3-10			
Question \$	Source:	Bank # Modified Bank # New	 	
Question (Cognitive Level	: Memory or Fu Comprehensi	undamental Knowledge on or Analysis	x
Examination	on Outline Cros	ss-reference:		
	Level K/A # Import Tier # Group 10 CFI	ance Rating # R 55.43(b)(2) & (4)	<u>RO</u>	SRO 2.3.11 3.2 3 3

Question # 118

A Unit 1 RCS Loop 1 RTD failure has resulted in tripping of all bistables associated with the failure. Sometime later in the shift, Power Range Channel N-44 fails high. Following the Channel N-44 failure, Unit 1 is in Mode 2 at 3% power preparing to enter Mode 1. Which one of the following statements describes the appropriate action to be taken?

- P. Continue the startup. T/S 3.3.1 allows the startup to continue as long as the inoperable channel is placed in tripped condition within 6 hours.
- Q. Enter T/S 3.0.3 and begin a unit shutdown per Procedure 0POP03-ZG-0006, "Plant Shutdown from 100% to Hot Standby."
- R. Trip the reactor and enter Procedure 0POP05-EO-EO00, "Reactor Trip or Safety Injection."
- S. Hold the startup. T/S 3.3.1 prevents entry into Mode 1 until Power Range Channel N-44 is operable.

Need to check to make sure the reactor should trip - believe it will be on an OTDT based on the logic

Answer:	С	Lesson Plan	(As available)
---------	---	-------------	----------------

References: 0POP04-RP-0004, Rev 7; 0POP04-NI-0001, Rev 3; 0POP04-RP-0001, Rev 5

Question Source:	Bank #	
	Modified Bank #	538
	New	

Question Cognitive Level:Memory or Fundamental Knowledge Comprehension or Analysis

Α

Level	RO	SRO
K/A #		012A2.01
Importance Rating		3.9
Tier #		2
Group #		2
10 CFR 55.43(b)(5)		

Question # 119

Given the following on Unit 2:

- The Unit was operating at 4% reactor power.
- Two loops of Essential Cooling Water (ECW) were determined to be inoperable.
- The operators placed the Unit in HOT STANDBY exactly 4 hours after determining that the second ECW loop was inoperable.

Which ONE of the following time limits apply to place the Unit in HOT SHUTDOWN and then COLD SHUTDOWN?

- A. HOT SHUTDOWN must be achieved within 6 hours of reaching HOT STANDBY and COLD SHUTDOWN must be achieved within an additional 30 hours.
- B. HOT SHUTDOWN must be achieved within 6 hours of reaching HOT STANDBY and COLD SHUTDOWN must be achieved within an additional 24 hours.
- C. HOT SHUTDOWN must be achieved within 9 hours of reaching HOT STANDBY and COLD SHUTDOWN must be achieved within an additional 30 hours.
- D. HOT SHUTDOWN must be achieved within 9 hours of reaching HOT STANDBY and COLD SHUTDOWN must be achieved within an additional 24 hours.

Answer:	D	Lesson Plan	(As available)	
Reference:	TS 3.0.3			
Question Sou	urce:	Bank # Modified Bank # New	342	
Question Co	gnitive Level:	Memory or Fundam Comprehension or	iental Knowledge Analysis	C
Examination	Outline Cross-	reference:		
	Level		RO	SRO

Level	RU	SRU
K/A #		076G2.1.12
Importance Rating		4.0
Tier #		2
Group #		3
10 CFR 55.43(b)(2)		

Question # 120

During routine rounds, a plant operator called the control room and reported that he observed an individual running from the area of Valve MOV-0143, "AFWP 14 Turbine Steam Inlet." At approximately the same time the Bypass Inop Status light alarms and Valve MOV-0143 indicated not full open. Security declared a Security Alert. What actions are required?

- A. Enter Procedure 0POP01-ZO-0001, "Suspected Sabotage or Tampering Response;" immediately trip the reactor and enter Procedure 0POP05-EO-EO00, "Reactor Trip or Safety Injection;" and declare an Unusual Event.
- B. Enter Procedure 0POP01-ZO-0001, "Suspected Sabotage or Tampering Response;" have the auxiliary operator detain the individual; and, declare an Alert
- C. Enter Procedure 0POP01-ZO-0001, "Suspected Sabotage or Tampering Response;" with concurrence from the Security Force Supervisor, walkdown the area looking for damaged equipment and mispositioned valves; and, declare an Unusual Event.
- D. Enter Procedure 0POP01-ZO-0001, "Suspected Sabotage or Tampering Response;" immediately trip the reactor and enter Procedure 0POP05-EO-EO00, "Reactor Trip or Safety Injection;" and, declare an Alert.

Answer:	С	Lesson Plan	(As available)
---------	---	-------------	----------------

Procedure: 0POP01-ZO-0001, "Suspected Sabotage or Tampering Response;" and 0ERP01-ZV-IN01, "Emergency Classification Tables"

Question Source:	Bank #	
	Modified Bank #	
	New	X

 Question Cognitive Level:
 Memory or Fundamental Knowledge
 M_

 Comprehension or Analysis

Level	RO	SRO
K/A #		2.4.28
Importance Rating		3.3
Tier #		3
Group #		4
10 CFR 55.43(b)(5)		

Question # 121

The following conditions exist in Unit 2:

- Mode 3 following a manual reactor trip and SI
- RCS pressure 1490 psig and STABLE
- Core Exit TCs are 555°F
- Pressurizer level is 48% and INCREASING
- Containment pressure is 6.5 psig
- SG NR levels are: 32%, 30%, 30%, 36%
- Total AFW flow is 400 gpm

The Unit Supervisor is at Step 15 of Procedure 0POP05-EO-EO10, "Loss of Reactor or Secondary Coolant," and is evaluating whether safety injection should be terminated.

Which ONE of the following actions is appropriate for the above conditions?

- A. SI termination criteria is met if AFW flow is adjusted to > 576 gpm. Do NOT transition to Procedure 0POP05-EO-ES11, "SI Termination," until AFW flow is adjusted.
- B. SI termination criteria is NOT met since RCS subcooling is less than the required value and further actions in Procedure 0POP05-EO-EO10 need to be performed.
- C. SI termination criteria is met and transition should be made to Procedure 0POP05-EO-ES11, "SI Termination."
- D. SI termination criteria is NOT met since pressurizer level is still low and further actions in Procedure 0POP05-EO-EO10 need to be performed.

Applicants will neeAnswer:B	ed Steam T	ables - be sure availa Lesson Plan	able in the testing area (As available)	
Procedure 0POP0	5-EO-EO1	0, "Loss of Reactor	or Secondary Coolant,'	' Step 15
Question Source:		Bank # 230_ Modified Bank # New		
Question Cognitive	e Level:	Memory or Fundame Comprehension or A	ental Knowledge Analysis	A
Examination Outlin	ne Cross-re	eference:		
	Level K/A #		<u>R0</u>	<u>SRO</u> EPE009EA2.3

LEVEI	NO	0110
K/A #		EPE009EA2.34
Importance Rating		4.2
Tier #		1
Group #		2
10 CFR 55.43(b)(5)		

Question # 122

Unit 2 containment pressure is 0.4 psig and requires a containment vent. The following conditions exist:

- Mode 4
- Containment Radiation Monitor RT-8011 is out-of-service.
- Reactor Containment Purge Permit has expired.

What is the appropriate procedure and action(s) for the above plant conditions?

- A. Per Procedure 0POP02-HC-0003, "Supplementary Containment Purge," with Chemistry approval, a containment purge is allowed.
- B. Per Procedure 0POP02-HC-0003, "Supplementary Containment Purge," conduct a containment purge, then notify Chemistry to update the Reactor Containment Purge Permit.
- C. Per Procedure, 0PGP03-ZO-0024, "Reactor Containment Purge Permit," obtain a new purge permit and then perform the containment purge.
- D. Per Procedure, 0PGP03-ZO-0024, "Reactor Containment Purge Permit," a portable containment air sample must be obtained and analyzed prior to performing the containment purge.

Answer: A Lesson Plan (As available)

Procedure 0POP02-HC-0003, "Supplementary Containment Purge"

Question Source:	Bank # Modified Bank # New	 x	
Question Cognitive Level:	Memory or Fundam Comprehension or	nental Knowledge Analysis	F

Level	RO	SRO
K/A #		2.3.9
Importance Rating		3.4
Tier #		3
Group #		3
10 CFR 55.43(b)(5)		

Question # 123

Plant conditions are as follows:

- A large break LOCA has occurred
- Containment pressure = 9.2 psig
- Containment water level = 59 inches
- Containment radiation level = 2500 R/hr

Which procedure is appropriate for the above conditions:

- A. Procedure 0POP05-EO-FRZ1, "Response to High Containment Pressure"
- B. Procedure 0POP05-EO-FRZ2, "Response to Containment Flooding"
- C. Procedure 0POP05-EO-FRZ3, "Response to High Containment Radiation Level"
- D. Procedure 0POP05-EO-ES13, "Transfer to Cold Leg Recirculation"

Answer:	С	Lesson Plan	(As available)
Procedure 0POP05-EO-FRZ3, "Response to High Containment Radiation Level"			
Question Sou	rce:	Bank # Modified Bank # New	
Question Cog	nitive Level:	Memory or Fundamental KnowledgeM Comprehension or Analysis	
Examination Outline Cross-reference:			

Level	RO	<u> </u>
K/A #		W/E16EA2.1
Importance Rating		3.3
Tier #		1
Group #		2
10 CFR 55.43(b)(5)		

Question # 124

Maintaining the minimum AFW Storage Tank volume required by Technical Specifications ensures that enough inventory is available to provide the ability to maintain ...

P. Hot Shutdown for 6 hours followed by a controlled cooldown to RHR entry conditions.

- Q. Hot Standby for 4 hours followed by a controlled cooldown to RHR entry conditions
- R. Hot Standby for 6 hours followed by a controlled cooldown to RHR entry conditions
- S. Hot Shutdown for 4 hours followed by a controlled cooldown to Cold Shutdown

Answer:	В	Lesson Plan	(As available)	
Reference:	T/S Basis 3/4 Feedwater; L	.7.1.3 and STP UFSAF OT202.28	R, Accident Analysis, 15.2.7, a	and Loss of Normal
Question Sou	rce:	Bank # Modified Bank # New	569	
Question Cog	nitive Level:	Memory or Fundamental KnowledgeF Comprehension or Analysis		
Examination C	Dutline Cross-re	eference:		
Level K/A # Importance Rating Tier # Group #		<u>RO</u>	<u>SRO</u> 061A1.04 3.9 2 1	

10 CFR 55.43(b)(2)

Question # 125

Given the following conditions:

- Unit 1 is in Mode 2, performing a startup
- "PRZR Level Cont Sel" Switch is selected to L465/467
- Pressurizer Level Channels indicate as follows:

Channel 465 = 26% Channel 466 = 26% Channel 467 = 0% Channel 468 = 25%

What actions are required to satisfy procedural requirements and Technical Specifications?

- A. Switch the "PRZR Level Cont Sel" Switch to L465/466; trip Channel 467 pressurizer high level bistable within 6 hours; and, then continue with startup.
- B. Switch the "PRZR Level Cont Sel" Switch to L465/466; trip Channel 467 pressurizer high and low level bistables within 6 hours; and, then continue with startup.
- C. Switch the "PRZR Level Cont Sel" Switch to L465/466; trip Channel 467 pressurizer high level bistable within 6 hours; place excess letdown in service; and restore Channel 467 before entering Mode 1.
- D. Switch the "PRZR Level Cont Sel" Switch to L465/466; trip Channel 467 pressurizer high and low level bistables within 6 hours; place excess letdown in service; and restore Channel 467 before entering Mode 1.

Answer: A Lesson Plan (As available)

Reference: T/S 3.3.1; Procedure 0POP04-RP-0002, "Loss Of Automatic Pressurizer Level Control"; and, Lesson LOT201.14.01

Comprehension or Analysis

Level	RO	SRO
K/A #		011A2.03
Importance Rating		3.9
Tier #		2
Group #		2
10 CFR 55.43(b)(2)&(5)		