LICENSE AUTHORITY FILE COPY

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DO NOT REMOVE

January 7, 1976

Posted Amn-14 to DPR-37

Docket Nos.: 50-280
and 50-281

Virginia Electric & Power Company
ATTN: Mr. Stanley Ragone
Senior Vice President
P. O. Box 26666
Richmond, Virginia 23261

Gentlemen:

The Commission has issued the enclosed Amendment No. 14 to Facility Operating Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications for each license and are in response to your request dated February 18, 1975.

The amendments incorporate into the Surry Units 1 and 2 Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The technical specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated December 8, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This Guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your technical specifications. Reports that are required by the regulations have not been repeated in your technical specifications.

Please note that we have discontinued the use of separate identifying numbers for changes to technical specifications. Sequential amendment numbers will be continued as in the past.

Copies of related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing

Enclosures:
1. Amendment No. 14
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/encls: See next page

cc w/ enclosures:
Michael W. Maupin, Esquire
Hunton, Williams, Gay & Gibson
P. O. Box 1535
Richmond, Virginia 23213

Swem Library
College of William & Mary
Williamsburg, Virginia 23185

Mr. Sherlock Holmes
Chairman
Board of Supervisors of Surry County
Surry County Courthouse
Surry, Virginia 23683

cc w/enclosures & incoming:
Ms. Susan T. Wilburn
Commonwealth of Virginia
Council on the Environment
P. O. Box 790
Richmond, Virginia 23206

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC & POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric & Power Company (the licensee) dated February 18, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-37 is hereby amended to read as follows:

   B. Technical Specifications

   The Technical Specifications contained in Appendix A, as revised are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment becomes effective 30 days after the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance:
January 7, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 14

FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Revise Appendix A as follows:

Remove Pages:	Insert Pages:
ii	ii
1.0-5 - 1.0-7	1.0-5 - 1.0-7
6.6-1 - 6.6-13	6.6-1 - 6.6-17

5. All automatic containment isolation valves are operable or are locked closed under administrative control.

6. The uncontrolled containment leakage satisfied Specification 4.4.

I. Reportable Occurrence

1. Definition: Refer to Technical Specification 6.6, Station Reporting Requirements for the definitions and examples of the two categories of Reportable Occurrence Reports

a. Prompt Notification With Written Followup.

b. Thirty Day Written Reports

J. Quadrant Power Tilt

The quadrant power tilt is defined as the ratio of the maximum upper excore detector current to the average of the upper excore detector currents or the ratio of the maximum lower excore detector current to the average of the lower excore detector currents whichever is greater. If one excore detector is out of service, the three in-service units are used in computing the average.

K. Low Power Physics Tests

Low power physics tests are tests conducted below 5% of rated power which measure fundamental characteristics of the reactor core and related instrumentation.

Revised page submitted 1-15-76

6.6 Station Reporting Requirements

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

1. Routine Reports

   a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

   Startup reports shall be submitted within (1) 90 days following completion of the startup test program,

(2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e.. initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

b. Annual Operating Report.[1/] Routine operating reports covering the operation of the unit during the previous calendar year should be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

Each annual operating report shall include:

(1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 1.b.(2)(e) below.

(2) For each outage or forced reduction in power[2/] of over twenty percent of design power level where

the reduction extends for greater than four hours:

(a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);

(b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;

(c) corrective action taken to reduce the probability of recurrence, if appropriate;

(d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,[3/] use the generator off-line hours; for forced reductions in power, use the approximate duration of operation at reduced power);

(e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and

(f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts

for more than 10% of the allowable annual values in 10 CFR 20.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions,[4/] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, with a copy to the appropriate Region Office, to arrive no later than the tenth of each month following the calendar

month covered by the report.

2. Reportable Occurrences

Reportable occurrences, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

(2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

(3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

(4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to 1% $\Delta k/k$; a calculated reactivity balance indicating a shutdown margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than 0.5% $\Delta k/k$; or occurrence of any unplanned criticality.

(5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the FSAR.

(6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the FSAR.

Note: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

(7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.

(8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.

(9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event.

The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain

radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

3. Unique Reporting Requirements

a. In-service Inspection Evaluation. Special summary technical report shall be submitted to the Director of Reactor Licensing, Office of Nuclear Reactor Regulation, NRC, Washington, D. C. 20555, after five (5) years of operation. This report shall include an evaluation of the results of the in-service inspection program and will be reviewed in light of the technology available at that time.

b. Reporting of Radioactivity in Effluent Releases, Solid Waste, and Fuel Shipments.

(1) Effluent releases - Data shall be reported on an annual basis but within 60 days after January 1st of each year in the form given in Appendix A of U. S. NRC Safety Guide No. 21 for water cooled nuclear power plants, entitled "Measuring and Reporting of Effluents from Nuclear Power Plants," dated December 29, 1971, or in equivalent form. Effluent data shall be summarized on a monthly basis except that when the majority of the activity is released as batches and there are less than 3 batches per month, each batch shall be reported. Estimates of the error associated with each six month total shall be reported. Specifically, the following

data shall be reported.

(a) Gaseous Releases

(i) total radioactivity (in curies) releases of noble and activation gases.

(ii) maximum noble gas release rate during any one-hour period.

(iii) total radioactivity (in curies) releases, by nuclide, based on representative isotopic analyses performed.

(iv) percent of technical specification limit.

(b) Iodine Releases

(i) total (I-131, I-133, I-135) radioactivity (in curies) released.

(ii) total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(iii) percent of technical specification limit.

(c) Particulate Releases

(i) gross radioactivity (β,γ) released (in curies) excluding background radioactivity.

(ii) gross alpha radioactivity released (in curies) excluding background radioactivity.

(iii) total radioactivity released (in curies) of nuclides with half-lives greater than eight days.

(iv) percent of technical specification limit.

(d) Liquid Releases

(i) gross radioactivity (β,γ) released (in curies) and average concentration released to the unrestricted area.

(ii) total tritium and alpha radioactivity (in curies) released and average concentration released to the unrestricted area.

(iii) total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted

(iv) total volume (in liters) of liquid waste released.

(v) total volume (in liters) of dilution water used prior to release from the restricted area.

(vi) the maximum concentration of gross radioactivity (β,γ) released to the unrestricted area (averaged over the period of release).

(vii) total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(viii) percent of technical specification limit for total activity released.

(2) Solid Radioactive Waste (Summarized Monthly) - Data shall be reported on an annual basis but within 60 days after January 1 of each year.

(a) Total amount of solid waste packaged (in cubic feet).

(b) Estimated total radioactivity (in curies) involved.

(c) Dates of shipment and disposition (if shipped off-site)

(3) Fuel Shipments - Information relative to each shipment of new and spent fuel shall be provided, including the following:

(a) Date of shipments

(b) Number of elements shipped

(c) Identification number and enrichment of elements shipped.

(d) Activity level at surface of each shipping cask containing spent fuel.

c. Radiological Environmental Monitoring. Data shall be reported to the NRC on an annual basis, but within 60 days after January 1 of each year. This report will include:

(1) Descriptive material covering the off-site environmental surveys performed during the reporting period including information on:

(a) The number and types of samples taken; e.g., air, soil, fish, etc.

(b) The number and types of measurements made; e.g. dosimetry.

(c) Location of the sample points and monitoring stations.

(d) The frequency of the surveys.

(e) A summary of survey results, including:

(i) number of locations at which activity levels are found to be significantly greater than local backgrounds.

(ii) highest, lowest, and the annual average concentrations or levels of radiation for the sampling point with the highest average and description of that point with respect to the site.

(2) If levels of station contributed radioactive materials in environmental media indicate the likelihood of public intakes in excess of 3% of those that could result from

continuous exposure to the concentration values listed in Appendix B, Table II of 10 CFR 20, estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.

(3) If a particular sample or measurements indicate statistically significant levels of radioactivity above established or concurrent backgrounds, the following information shall be provided:

(a) The type of analysis performed; e.g., alpha, beta, gamma and/or isotopic.

(b) The minimum sensitivity of the monitoring system.

(c) The measured radiation level or sample concentration.

(d) The specific times when samples were taken and measurements were made.

(e) An estimate of the likely resultant exposure to the public if it exceeds 10 mrem.

d. Non-Radiological Environmental Monitoring. Data shall be reported to the NRC on an annual basis, but within 60 days after January 1 of each year. This report will satisfy the requirements of TS 4.13.D and TS 4.13.E.

e. Containment Leak Rate Test. Each containment integrated leak rate test shall be the subject of a summary technical report. Upon completion of the initial containment leak rate test specified by proposed Appendix J to 10 CFR 50, a special report shall, if that Appendix is adopted as an effective rule, be submitted to the Director, Division of Reactor Licensing, USNRC, Washington, D. C. 20555, and other containment leak rate tests specified by Appendix J that fail to meet the acceptance criteria of the appendix, shall be the subject of special summary technical reports pursuant to Section V.B of Appendix J.

(1) "B. Report of Test Results - The initial Type A test shall be subject of a summary technical report submitted to the Commission approximately 3 months after the conduct of the test. This report shall include a schematic arrangement of the leakage rate measurement system, the instrumentation used, the supplemental test method, and the test program selected as applicable to the initial test, and all subsequent periodic tests. The report shall contain an analysis and interpretation of the leakage rate test data to the extent necessary to demonstrate the acceptability of the containment's leakage rate in meeting the acceptance criteria."

"For periodic tests, leakage rate results of Type A, B, and C tests that meet the acceptance criteria of Sections III.A.7, III.B.3, respectively, shall be reported in the licensee's periodic operating report. Leakage test results of Type A, B, and C tests that fail to meet the acceptance criteria of Sections III.A.7, III.B.3, and III.C.3, respectively, shall be reported in a separate summary report that includes an

analysis and interpretation of the test data, the least swuares fit analysis of the test data, the instrument error analysis, and the structural conditions of the containment or components, if any, which contributed to the failure in meeting the acceptance criteria. Results and analyses of the supplemental verification test employed to demonstrate the validity of the leakage rate test measurements shall also be included."

f. Initial Containment Structural Test. A special summary technical report shall be submitted to the Director, Division of Reactor Licensing, USNRC, Washington, D. C. 20555, within 3 months after completion of the test. This report will include a summary of the measurements of deflections, strains, crack width, crack patterns observed, as well as comparisons with predicted values of acceptance criteria.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

2. The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this section.

3. The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

4. This tabulation supplements the requirements of §20.407 of 10 CFR Part 20.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NO. 14 TO LICENSES NOS. DPR-32 AND DPR-37

VIRGINIA ELECTRIC & POWER COMPANY

SURRY POWER STATION UNITS 1 & 2

DOCKETS NOS. 50-280 AND 50-281

Introduction

By letter dated February 18, 1975, Virginia Electric & Power Company proposed changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station Units 1 & 2. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and an abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrence" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to these facilities. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with the licensee's staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date:
January 7, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-280 AND 50-281

VIRGINIA ELECTRIC & POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 14 to Facility Operating Licenses Nos. DPR-32 and DPR-37 issued to Virginia Electric & Power Company which revised Technical Specifications for operation of the Surry Power Station, Units 1 and 2, located in Surry County, Virginia. The amendments become effective 30 days after the date of issuance.

The amendments revise the reporting requirements of the Technical Specifications for the Surry Power Station, Units 1 and 2.

The application for the amendments dated February 18, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated February 18, 1975, (2) Amendments No. 14 to Licenses Nos. DPR-32 and DPR-37, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Swem Library, College of William & Mary, Williamsburg, Virginia.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland this 7th day of January, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Reactor Licensing

Posted

Amd-15 to DPR-37

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 26, 1976

Dockets Nos. 50-280
and 50-281

Virginia Electric & Power Company
ATTN: Mr. Stanley Ragone
Senior Vice President
P. O. Box 26666
Richmond, Virginia 23261

Gentlemen:

The Commission has issued the enclosed Amendments No. 15 to Facility Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station, Units 1 and 2. The amendments consist of changes to your Technical Specifications for each license and are in response to your request dated July 23, 1975, as completely revised November 4, 1975.

The amendments revise the provisions of Section 6.0 of the Technical Specifications related to plant organization, plant personnel qualifications, and the organization of the System Nuclear Safety and Operating Committee.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

Robert W. Reid

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Enclosures:
1. Amendment No. 15 to DPR-32
2. Amendment No. 15 to DPR-37
4. Safety Evaluation
4. Federal Register Notice

cc w/enclosures:
See next page

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC & POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric & Power Company (the licensee) dated November 4, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the
Technical Specifications

Date of Issuance:
January 26, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NO. 50-281

Revise Appendix A as follows:

Remove Pages:	Insert Pages:
6.1-1 - 6.1-10	6.1-1 - 6.1-12
Figures 6.1-1 - 6.1-4	Figures 6.1-1 - 6.1-3
6.4-7a	6.4-7a

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, SAFETY AND OPERATION REVIEW

Specification

A. The Station Manager shall be responsible for the safe operation of the facility. The Station Manager shall report to the Director-Nuclear Operation. The relationship between this Director and other levels of company management is shown in TS Figures 6.1-1 and 6.1-2.

B. The station organization shall conform to the chart as shown in TS Figure 6.1-3.

1. Qualifications with regard to education and experience and the technical specialties of key supervisory personnel will meet the minimum acceptable levels described in American National Standard 18.1 "Selection and Training of Nuclear Power Plant Personnel" dated March 8, 1971.

The key supervisory personnel are as follows:

a) Manager

b) Superintendent-Station Operations

c) Operating Supervisor

d) Supervisor-Electrical Maintenance

e) Supervisor-Mechanical Maintenance

f) Supervisor-Engineering Services

g) Chemistry and Health Physics Supervisor

h) Shift Supervisor

2. Retraining and replacement training of station personnel shall be in accordance with American National Standard 18.1 "Selection and Training of Nuclear Power Plant Personnel" dated March 8, 1971 and 10 CFR 55, Appendix A.

3. The following requirements supplement the applicable regulations of 10 CFR 50.54:

	Condition	Minimum Complement
1.	One unit operating	1 SLO, 2 LO, 2 AO
2.	One unit fueled and shutdown**	1 SLO, 1 LO, 1 AO
3.	One unit operating and one unit shutdown	1 SLO*, 3 LO, 2 AO
4.	Both units fueled and shutdown**	1 SLO, 2 LO, 1 AO
5.	Both units operating	2 SLO, 3 LO, 2 AO

Note: SLO = Senior Licensed Operator as defined by 10 CFR 55.4(e)
LO = Licensed Operator as defined by 10 CFR 55.4(d)
AO = Auxiliary Operator
* When the shutdown unit is undergoing refueling or startup, 1 additional SLO will be added to this shift complement to ensure supervision of these activities.
** A LO for each fueled unit shall be in the control room and a SLO shall be on site. For each SLO in the control room, the requirement to have a LO in the control room shall be waived.

C. Organization units to provide a continuing review of the operational and safety aspects of the nuclear facility shall be constituted and have the authority and responsibilities outlined below:

1. Station Nuclear Safety and Operating Committee

a. Membership

1. Chairman - Station Manager

2. Vice Chairman - Superintendent-Station Operations

3. Operating Supervisor

4. Supervisor-Electrical Maintenance

5. Supervisor-Mechanical Maintenance

6. Supervisor-Engineering Services

7. Chemistry and Health Physics Supervisor

b. Qualifications: The qualifications of the regular members of the Station Nuclear Safety and Operating Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level at least equal to those described in Section 6.1.B.1 of these Specifications.

c. Meeting frequency: As called by the Chairman but not less than monthly.

d. Quorum: Chairman or Vice Chairman, Chemistry and Health Physics Supervisor, and three others to provide a quorum of five members. The Chairman or Vice Chairman may appoint a similiarly qualified designee to represent a member other than the Chairman or Vice Chairman.

e. Responsibilities

1. Periodically review all proposed normal, abnormal, and emergency operating procedures and all proposed maintenance procedures. Review proposed changes to those procedures, and any other proposed procedures or changes thereto as determined by the Station Manager which affect nuclear safety.

2. Review all proposed test and experiment procedures and results thereof when applicable.

3. Review proposed changes to Technical Specifications.

4. Review all proposed changes or modifications to systems or equipment that would require a change in established procedures, or which would constitute a design change.

5. Periodically review all operations to detect any potential safety hazards.

6. Investigate all reported instances of departure from Technical Specification limits, such investigations to include review, evaluation and recommendations to prevent recurrence, to the Station Manager, Director Nuclear Operation and to the Chairman of the System Nuclear Safety and Operating Committee.

7. The Station Nuclear Safety and Operating Committee shall make determinations as to whether or not proposals considered by the Committee involve unreviewed safety questions. This determination shall be subject to review by the System Nuclear Safety and Operating Committee.

8. Review all reportable occurrence reports.

9. Perform special reviews and investigations and render reports thereon as requested by the Chairman of the System Nuclear Safety and Operating Committee.

10. Initiate periodic drills to test the effectiveness of the emergency procedures.

f. Authority

1. The Station Nuclear Safety and Operating Committee shall advise the Manager on all matters affecting the safe operation of the facility.

2. The Station Nuclear Safety and Operating Committee shall recommend to the Station Manager approval or disapproval of proposals under items e(1) through (4) above.

a) In the event of disagreement between the recommendations of the Station Nuclear Safety and Operating Committee and the actions contemplated by the Station Manager, the course determined by the Station Manager will be followed with immediate notification to the Director Nuclear Operation and the Chairman of the System Nuclear Safety and Operating Committee.

g. Records

Minutes shall be kept of all meetings of the Station Nuclear

Safety and Operating Committee and copies shall be sent to the Director Nuclear Operation and to all members of the Station and System Nuclear Safety and Operating Committees.

h. Procedures

Written administrative procedures for committee operation shall be prepared and maintained describing the method of submission, and the content of presentations to the committee, provisions for the use of subcommittees; review and approval by members of written committee evaluations and recommendations; the distributions of minutes; and, such other matters as may be appropriate.

2. System Nuclear Safety and Operating Committee (SyNSOC)

a. Composition

1. The SyNSOC shall be composed of the: Chairman and four other members. Membership shall be composed of the Managers of the Power Station Engineering, Nuclear Fuel Services, Production Operation and Maintenance Departments, and Executive Manager Licensing and Quality Assurance or qualified designees from these departments and a fifth qualified member selected by the four other members.

b. Qualifications

The minimum qualifications of the members of the SyNSOC will be: An engineering graduate or equivalent with combined nuclear and conventional experience in power station design and/or operation

of eight years, with at least two years involving the direction of nuclear operations or design activity. Related post graduate formal training may be counted as experience on a one for one basis, except that no more than three years may be considered experience.

c. Consultants

Consultants shall be utilized as determined by the SyNSOC Chairman to provide expert advice to the SyNSOC.

d. Quorum

A quorum for formal meetings of the SyNSOC shall consist of not less than a majority of the members or duly appointed alternates and shall be subject to the following constraints:

1. The Chairman or Vice Chairman shall be present for all meetings.
2. No more than a minority of the quorum shall have line responsibility for operation of the stations.
3. A motion-carrying vote must consist of no less than three (3) votes.
4. No more than a minority of a quorum may be alternates.

e. Meeting Frequency

The SyNSOC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once every six months thereafter.

f. Alternates

Alternate members shall be appointed in writing by the SyNSOC Chairman to serve on a temporary basis. The use of alternates should be restricted to legitimate absences of members.

g. Function

The SyNSOC shall function to provide independent review and audit of designated activities in the areas of:

1. Nuclear power station operations
2. Nuclear engineering
3. Chemistry and radiochemistry
4. Metallurgy
5. Instrumentation and control
6. Radiological safety
7. Mechanical and electrical engineering
8. Quality assurance practices
9. Other appropriate fields as deemed necessary by the SyNSOC

h. Review

The following subjects shall be reviewed by the SyNSOC:

1. Written safety evaluations of changes in the stations as described in the Safety Analysis Report, changes in procedures as described in the Safety Analysis Report and tests or experiments not described in the Safety Analysis Report which are completed without prior NRC approval under the provisions of 10 CFR 50.59(a)(1). This review is to verify

that such changes, tests or experiments did not involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2), and is accomplished by review of minutes of the Station Nuclear Safety and Operating Committee and the design change program.

2. Proposed changes in procedures, proposed changes in the station, or proposed tests or experiments, any of which may involve a change in the technical specifications or an unreviewed safety question as defined in 10 CFR 50.59(a)(2). Matters of this kind shall be referred to the SyNSOC by the Station Nuclear Safety and Operating Committee following its review prior to implementation.

3. Changes in the technical specifications or license amendments relating to nuclear safety prior to implementation except in those cases where the change is identical to a previously reviewed proposed change.

4. Violations and reportable occurrences such as:

   (a) Violations of applicable codes, regulations, orders, technical specifications, license requirements or internal procedures or instructions having safety significance;

   (b) Significant operating abnormalities or deviations from normal or expected performance of station safety-related structures, systems, or components; and

(c) Reportable occurrences as defined in the station technical specifications.

Review of events covered under this paragraph shall include the results of any investigations made and the recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

5. Any other matter involving safe operation of the nuclear power station which a duly appointed subcommittee or committee member deems appropriate for consideration, or which is referred to the SyNSOC by the Station Nuclear Safety and Operating Committee.

i. Audits

Audits of station activities shall be performed under the cognizance of the SyNSOC. These audits shall encompass:

1. The conformance of station operation to all provisions contained within the Technical Specifications, and applicable license conditions at least once per year.
2. The performance, training and qualifications of the entire station staff at least once every year.
3. The results of all actions taken to correct deficiencies occurring in station equipment, structures, systems or

method of operation that affect nuclear safety at least once every six (6) months.

4. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix B, 10 CFR 50, at least once every two (2) years.

5. The station Emergency Plan and implementing procedures at least once every two (2) years.

6. The station Security Plan and implementing procedures at least once every two (2) years.

7. Any other area of station operation considered appropriate by the SyNSOC or the Vice President-Power Supply and Production Operations.

j. Authority

The SyNSOC shall report to and advise the Vice President-Power Supply and Production Operations on those areas of responsibility specified in Sections 6.1.C.2.h and 6.1.C.2.i

k. Records

Minutes of SyNSOC meetings and recommendations shall be prepared, maintained and disseminated promptly to:

1. Senior Vice President - Power
2. Vice President - Power Supply and Production Operations
3. Nuclear Power Station Managers

4. Director Nuclear Operations

5. Members of the SyNSOC

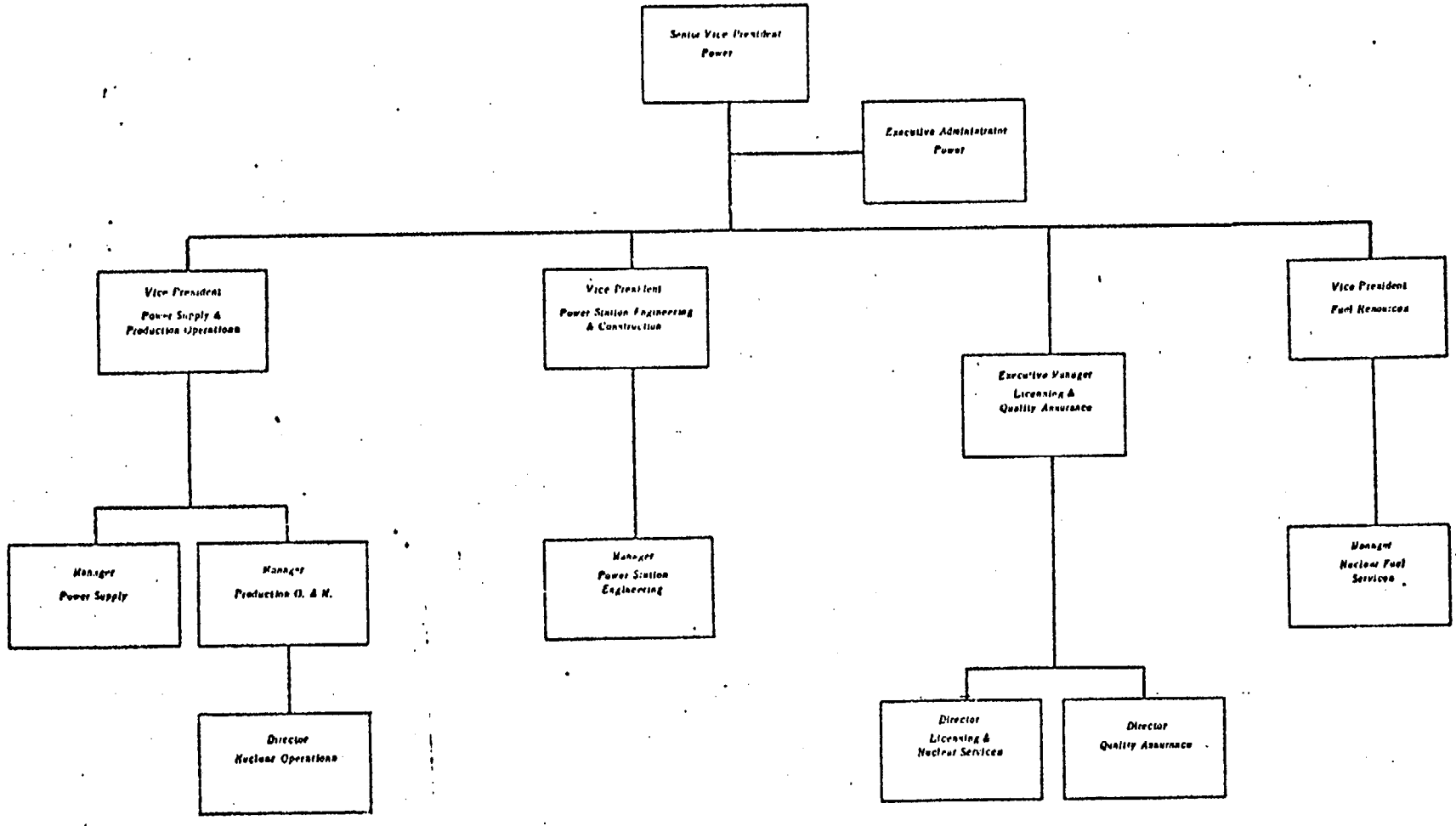
6. Others that the Chairman of the SyNSOC may designate.

1. Procedures

Written bylaws and administrative procedures for committee operation shall be maintained describing the method of submission and the content of presentations to the committee; provisions for use of subcommittee evaluations and recommendations; distribution of minutes, and, such other matters as may be appropriate.

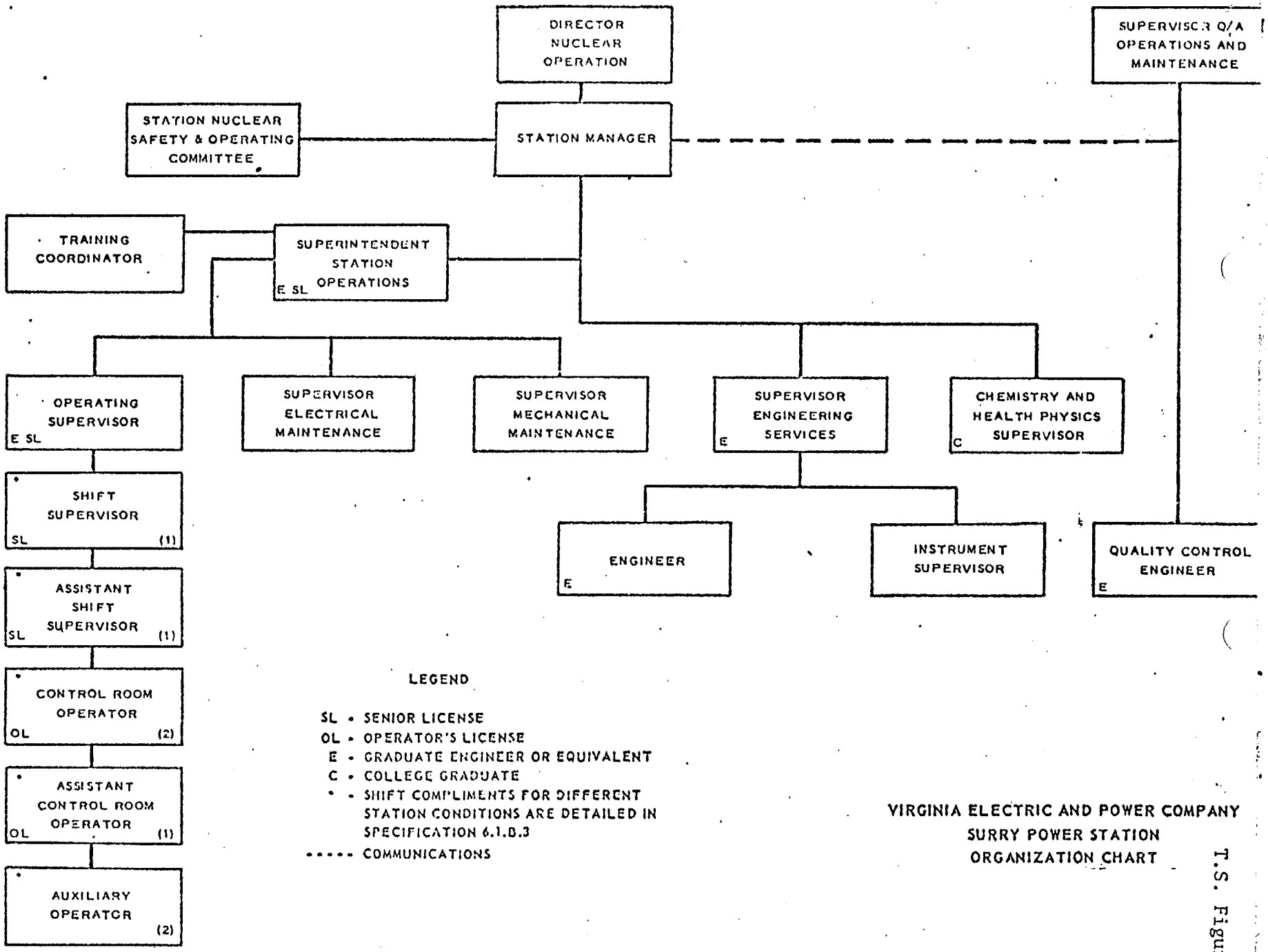
Amendment No. 15

VIRGINIA ELECTRIC AND POWER COMPANY
ORGANIZATION CHART
POWER



October 1, 1975

T.S. Fig. 6.1-1

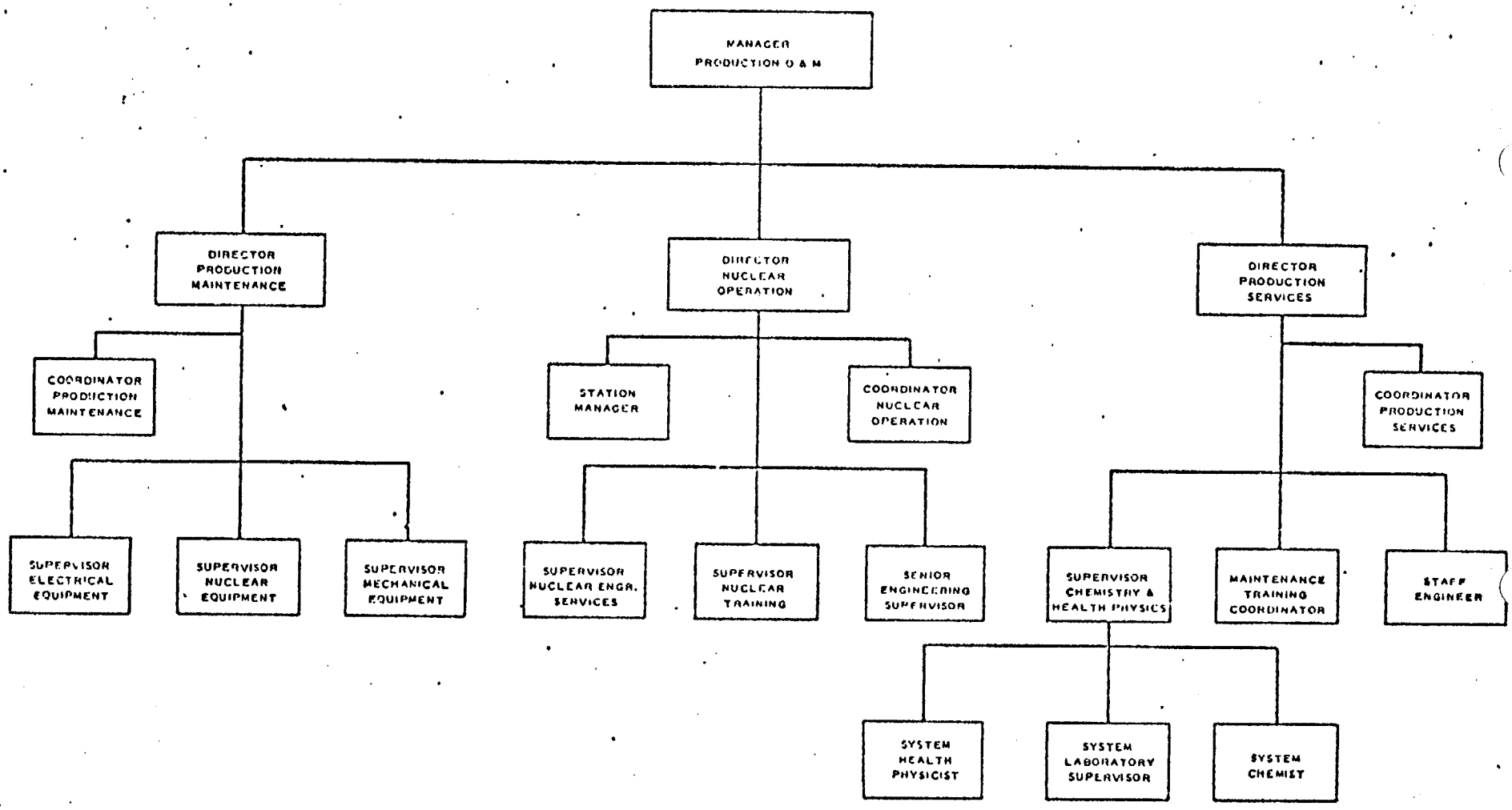


VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION
ORGANIZATION CHART

T.S. Figure 6.1-

Amendment No. 15

VIRGINIA ELECTRIC AND POWER COMPANY
ORGANIZATION CHART
PRODUCTION O & M DEPARTMENT ORGANIZATION
AVAILABLE STAFF SUPPORT



Amendment No. 15

TS FIG 6.1-2

D. All procedures described in A and B above shall be followed.

E. Temporary changes to procedures described in A and B above which do not change the intent of the original procedure may be made, provided such changes are approved prior to implementation by the person designated below based on the type of procedure to be changed:

1.	Administrative	Station Manager
2.	Abnormal	Shift Supervisor
3.	Annunciator	Shift Supervisor
4.	Chemistry and Health Physics	*Health Physist, Chemist
5.	Emergency	Shift Supervisor
6.	Electrical Maintenance	*Electrical Foreman
7.	Mechanical Maintenance	*Mechanical Foreman
8.	Operating	Shift Supervisor
9.	Periodic Test	*Cognizant Supervisor
10.	Start-up Test	*Supervisor-Engineering Services
11.	Special Test	*Supervisor-Engineering Services
12.	Quality Control	Quality Control Engineer

* In addition, these procedures must have the approval of a licensed Senior Reactor Operator.

Such changes will be documented and subsequently reviewed by the Station Nuclear Safety and Operating Committee and approved by the Station Manager within seven days.

F. Temporary changes to procedures described in A and B above which change the intent of the original procedure may be made, provided such changes are approved prior to implementation by the person designated below based on the type of procedure to be changed.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NO. 15 TO LICENSES NOS. DPR-32 AND DPR-37

VIRGINIA ELECTRIC & POWER COMPANY

SURRY POWER STATION UNITS 1 & 2

DOCKETS NOS. 50-280 AND 50-281

INTRODUCTION

By letter dated July 23, 1975 and as completely amended November 4, 1975, Virginia Electric & Power Company (the licensee) requested amendments to Facility Operating Licenses Nos. DPR-32 and DPR-37. The purpose of the request is to revise the Surry Units 1 and 2 Technical Specifications in order to implement miscellaneous changes to the Administrative Controls.

DISCUSSION

The licensee requested changes in order to update both the Corporate and Surry Power Station Organizational Charts; and in addition, to reorganize the System Nuclear Safety and Operating Committee (SyNSOC). Experience and training requirements of plant personnel have been expanded, and the personnel who must approve certain temporary changes to procedures have been identified.

EVALUATION

The changes to the Corporate and Surry Station organizational charts are acceptable in that they provide the required corporate technical support to the station.

The reorganization of the SyNSOC is acceptable as our review indicates that all of the changes follow the guidance of ANSI 18.7, Draft 5 of Revision 1, dated November 26, 1974, "Independent Review Program".

The expansion of experience and training requirements are acceptable as it follows the guidance of the latest ANSI 18.1 Standard, dated March 8, 1971.

None of the above changes involve any increase in the consequences of an accident nor significantly affect the probability of an accident. No revision of safety limits is involved.

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date:
January 26, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-280 AND 50-281

VIRGINIA ELECTRIC & POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 15 to Facility Operating Licenses Nos. DPR-32 and DPR-37 issued to Virginia Electric & Power Company which revised Technical Specifications for operation of the Surry Power Station, Units 1 and 2, located in Surry County, Virginia. The amendments are effective as of the date of issuance.

The amendments revise the provisions in the Technical Specifications relating to miscellaneous changes in the Administrative Controls section.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated November 4, 1975, (2) Amendments No. 15 to Licenses Nos. DPR-32 and DPR-37, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C and at the Swem Library, College of William and Mary, Williamsburg, Virginia.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Dated at Bethesda, Maryland this 26th day of January, 1976.