

July 15, 1997

Mr. J.P. O'Hanlon
Senior Vice President Nuclear
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Blvd.
Glen Allen, Virginia 23060

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: ASME CODE
REQUIREMENTS FOR MAIN STEAM AND FEEDWATER SYSTEMS (TAC NOS.
M97908 AND M97909)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment No.212 to Facility Operating License No. DPR-32 and Amendment No. 212to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letters dated February 3, 1997, and March 18, 1997.

These amendments revise the TS to eliminate the inconsistency between the current approved Inservice Inspection Program and ASME Code (1989 Edition) and the Surry TS, as required by 10 CFR 50.55a(g)(5)(ii).

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

This completes our effort on this issue and we are, therefore, closing out TAC Nos. M97908 and M97909.

Sincerely,

Original signed by: [Signature]
Gordon Edison, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosures:

1. Amendment No.212 to DPR-32
2. Amendment No.212 to DPR-37
3. Safety Evaluation

cc w/enclosures:

See next page

Distribution: See next page

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OFFICE	PDII-1	LA:PDII-1	EMEB	PM:PDII-1	OGCS	AD:PDII-1
NAME	TEaton	Dunnington	DTerao	GEdison	R Bachmann	R. Eaton
DATE	7/1/97	7/1/97	7/1/97	7/2/97	7/8/97	7/14/97
COPY	(Yes/No)	(Yes/No)	(Yes/No)	(Yes/No)	Yes/No	Yes/No

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Mr. J. P. O'Hanlon
Virginia Electric and Power Company

Surry Power Station

cc:

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DATED: July 15, 1997

AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1
AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

~~Public File~~

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ACRS

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. DPR-32

- I. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 3, 1997, and March 18, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 212, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon E. Edison, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 15, 1997



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 212
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 3, 1997, and March 18, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

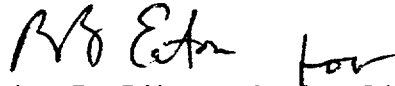
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 212, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Gordon E. Edison, Acting Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 15, 1997

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-32
AMENDMENT NO. 212 TO FACILITY OPERATING LICENSE NO. DPR-37
DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 4.15-1

TS 4.15-3

TS Fig. 4.15

Insert Pages

TS 4.15-1

TS 4.15-3

TS Fig. 4.15

AUGMENTED INSERVICE INSPECTION PROGRAM FOR HIGH ENERGY LINES OUTSIDE OF CONTAINMENT

Applicability

Applies to welds in piping systems or portions of systems located outside of containment where protection from the consequences of postulated ruptures is not provided by a system of pipe whip restraints, jet impingement barriers, protective enclosures and/or other measures designed specifically to cope with such ruptures.

For Surry Units 1 and 2, this specification applies to welds in the main steam and main feedwater lines in the main steam valve house of each unit.

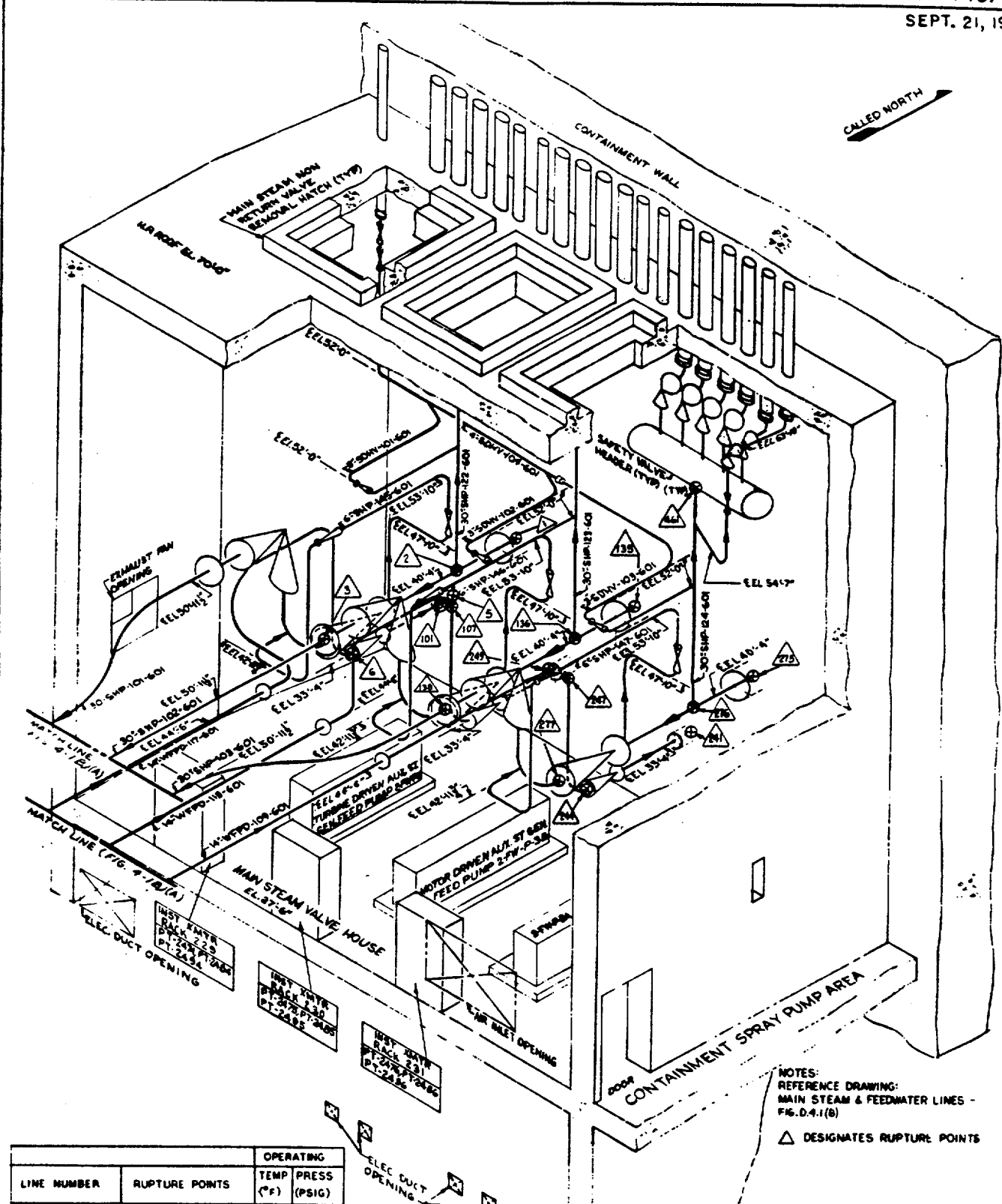
Objective

To provide assurance of the continued integrity of the piping systems over their service lifetime.

Specifications

- A. For the 20 welds identified in TS Figure 4.15:
1. At the first refueling outage period a volumetric examination shall be performed with 100 percent inspection of welds in accordance with the requirement of ASME Section XI Code, Inservice Inspection of Nuclear Power Plant Components, to

4. In the event repairs of any welds are required following any examination during successive inspection intervals, the inspection schedule for the repaired welds will revert back to the first 10 year inspection program.
- B. For all welds other than those identified in TS Figure 4.15:
1. Welds in the main steam lines including the safety valve headers and in the feedwater lines in the main steam valve house shall be examined in accordance with the requirements of subsection IWC of the ASME Section XI Code.
- C. For all welds in the main steam valve house:
1. A visual inspection of the surface of the insulation at all weld locations shall be performed on a weekly basis for detection of leaks. Any detected leaks shall be investigated and evaluated. If the leakage is caused by a through-wall flaw, either the plant shall be shutdown, or the leaking piping isolated. Repairs shall be performed prior to return of this line to service.
 2. Repairs, reexamination and piping pressure tests shall be conducted in accordance with the rules of ASME Section XI Code.



NOTES:
REFERENCE DRAWING:
MAIN STEAM & FEEDWATER LINES -
FIG. D.4.1(B)
△ DESIGNATES RUPTURE POINTS

LINE NUMBER	RUPTURE POINTS	OPERATING	
		TEMP (°F)	PRESS (PSIG)
14-WFPD-109-601	241, 244, 247 & 249	450	1032
14-WFPD-113-601	101 & 107	450	1032
14-WFPD-117-601	5 & 6	450	1032
30-SHP-103-601	275, 276 & 277	547	1005
30-SHP-102-601	135, 136 & 138	547	1005
30-SHP-101-601	1, 2, 2	547	1005
30-SHP-124-601	461 (TYP FOR SAFETY VALVE HEADERS)	547	1015

MAIN STEAM AND FEEDWATER LINES - UNIT 2
SURRY POWER STATION
UNITS 1 AND 2



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO.212TO FACILITY OPERATING LICENSE NO. DPR-32

AND AMENDMENT NO.212TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NUMBERS: 50-280 AND 50-281

1.0 INTRODUCTION

The regulations require that inservice examination of components and system pressure tests conducted during the first 10-year interval and subsequent intervals comply with the requirements in the latest edition and addenda of Section XI of the American Society of Mechanical Engineers (ASME) Code incorporated by reference in 10 CFR 50.55a(b) 12 months prior to the start of the 120-month interval, subject to the limitations and modifications listed therein. The applicable edition of Section XI of the ASME Code for Surry Power Station, Units 1 and 2, third 10-year inservice inspection (ISI) interval is the 1989 edition.

If the licensee determines that the revised inspection program for the facility conflicts with the Technical Specifications (TS) for the facility, then the licensee shall amend the TS to conform to the revised inspection program as required by 10 CFR 50.55a(g)(5)(ii), at least 6 months prior to the period during which the provisions become applicable. In a letter dated February 3, 1997, Virginia Electric and Power Company (licensee) submitted to the NRC a proposed TS change to delete a specific reference to the ASME Section XI Code reference and replace it with a general reference to the ASME Section XI Code for Class 2 welds (IWC) in Section 4.15 of the TS. Also, in a supplemental letter dated March 18, 1997, the licensee requested to reincorporate specific pages TS 4.15-1, TS 4.15-3, and TS FIG 4.15. It was discovered prior to this submittal, that these three pages may have been inadvertently omitted in a previous amendment to the Surry Unit 1 and 2 TS (Amendments 39 and 40 on April 13, 1978).

2.0 EVALUATION

The staff has evaluated the information provided by the licensee, in submittals dated February 3, 1997, and March 18, 1997, in support of proposed changes to the TS.

For ISI of the main steam and feedwater lines at Surry, Units 1 and 2, the TS require the licensee to meet Section XI requirements of the ASME Boiler and Pressure Vessel Code up to and including the Winter 1972 Addenda. However, pursuant to 10 CFR 50.55a, licensees are required to update the ASME Code, Section XI requirements every 10 years (120 months) to the latest edition and addenda of Section XI incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the start of the 10-year interval subject to the limitations and modifications therein.

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The latest version of the ASME Code, Section XI referenced in 10 CFR 50.55a(b) is the 1988 Addenda and editions through the 1989 Edition. The ISI program for Surry, Units 1 and 2, is currently in its third 10-year interval, and the licensee is committed to meet the requirements of the 1989 Edition. The staff has reviewed the Surry, Units 1 and 2, ISI program plans for the third 10-year interval and has found them acceptable.

The staff has previously approved the ISI program plans which include the 1989 Edition of the ASME Code, Section XI. By deleting the TS reference to the 1972 Winter Addenda of the ASME Section XI Code, the TS will be made consistent with the current approved ISI program. This represents an administrative change because the 1989 Code Edition in the ISI program has been implemented at the Surry Station.

Additionally, subsequent to the February 3, 1997 submittal, it was identified that pages TS 4.15-1, TS 4.15-3, and TS Fig. 4.15 were inadvertently omitted in previous amendments (Amendments 39 and 40) to the Surry, Unit 1 and Unit 2 TS. Therefore, for the purpose of clarity, in addition to the specific proposed change noted above, TS 4.15 has been revised to include those pages which were inadvertently omitted.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia state official was notified of the proposed issuance of amendments. The State official had no comment.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (62 FR 17242). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such

activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Eaton

Date: July 15, 1997