

March 1, 1994

Docket Nos. 50-280, 50-281  
and 50-338, 50-339

**DISTRIBUTION:**  
See next page

Mr. W. L. Stewart  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY POWER STATION UNITS 1 AND 2 AND NORTH ANNA POWER STATION UNITS  
1 AND 2 - ISSUANCE OF AMENDMENTS RE: REVIEW AND AUDIT FREQUENCIES  
(TAC NOS. M87225, M87226, M87142, AND M87143)

The Commission has issued the enclosed Amendment No.188 to Facility Operating License No. DPR-32 and Amendment No. 188 to Facility Operating License No. DPR-37 for the Surry Power Station, Units 1 and 2, respectively. Also, the Commission has issued the enclosed Amendment No. 180 to Facility Operating License No. NPF-4 and Amendment No. 161 to Facility Operating License No. NPF-7 for the North Anna Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated July 20, 1993.

These amendments delete the TS requirement for Station Nuclear Safety and Operating Committee review of the Emergency and Security Plans. This requirement remains in the respective plans. The audit frequencies are also being deleted from the TS.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

9403100277 940301  
PDR ADOCK 05000280  
P PDR

Sincerely,

(Original Signed By)

(Original Signed By)

Bart C. Buckley, Sr. Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II

Leon B. Engle, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II

Enclosures:

1. Amendment No. 188 to DPR-32
2. Amendment No. 188 to DPR-37
3. Amendment No. 180 to NPF-4
4. Amendment No. 161 to NPF-7
5. Safety Evaluation

080035

REC FILE CONTROL COPY

cc w/enclosures: See next page

OFC	LA:PDII-2	PE:PDII-2	PM:PDII-2	PM:PDII-2
NAME	ETana <i>ET</i>	RCroteau <i>RC</i>	BBuckley <i>BB</i>	LEngle <i>LE</i>
DATE	12/10/93	12/14/93	12/15/93	12/16/93
OFC	D:PDII-2	OGC		
NAME	HBerkow <i>HB</i>	S.Hom		
DATE	12/10/93	12/21/93		

*DF01*

Virginia Electric & Power Company

cc:

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Mr. W. L. Stewart  
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Mr. Michael R. Kansler, Manager  
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Mr. Sherlock Holmes, Chairman  
Board of Supervisors  
of Surry County  
Surry County Courthouse  
Surry, Virginia 23683

DATED: March 1, 1994

AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1  
AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2  
AMENDMENT NO. 180 TO FACILITY OPERATING LICENSE NO. NPF-4-NORTH ANNA UNIT 1  
AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NO. NPF-7-NORTH ANNA UNIT 2

Distribution

~~Bracket File~~

NRC & Local PDRs  
PDII-2 Reading  
S. Varga, 14/E/4  
G. Lainas, 14/H/3  
H. Berkow  
E. Tana (2)  
R. Croteau  
B. Buckley  
L. Engle  
OGC  
D. Hagan, 3302 MNBB  
G. Hill (4), P-137  
C. Grimes, 11/F/23  
ACRS (10)  
OPA  
OC/LFMB  
M. Sinkule, R-II



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY  
DOCKET NO. 50-280  
SURRY POWER STATION, UNIT NO. 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 188  
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated July 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

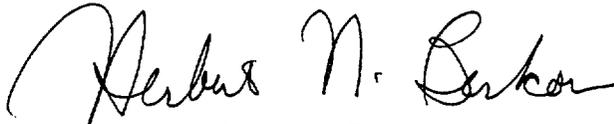
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 188 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 1, 1994



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 188  
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated July 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

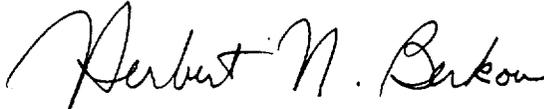
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 188, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 1, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 188 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

6.1-8  
6.1-11  
6.1-12  
6.4-6  
6.5-3

Insert Pages

6.1-8  
6.1-11  
6.1-12  
6.4-6  
6.5-3

9. Deleted.
10. Deleted.
11. Review of every unplanned onsite release of radioactive material to the environs exceeding the limits of Specification 3.11, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Operations and to the Management Safety Review Committee.
12. Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

g. Authority

The SNSOC shall:

1. Provide written approval or disapproval of items considered under (1) through (3) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
2. Render determinations in writing with regard to whether or not each item considered under (1) through (5) above constitutes an unreviewed safety question.
3. Provide written notification within 24 hours to the Vice President - Nuclear Operations and to the Management Safety Review Committee of disagreement between SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.A above.

h. Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Operations and to the Management Safety Review Committee.

5. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
6. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
7. Events requiring written notification to the Commission.
8. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
9. Reports and meeting minutes of the SNSOC.

**h. Audits**

Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

1. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
2. The performance, training and qualifications of the entire facility staff.
3. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.

4. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
5. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
6. The Fire Protection Program and implementing procedures.
7. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
8. The radiological environmental monitoring program.
9. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
10. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive waste.

- H. Practice of site evacuation exercises shall be conducted annually, following emergency procedures and including a check of communications with off-site report groups. |
- I. Deleted. |
- J. Deleted. |
- K. Systems Integrity |

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

9. Records of the service lives of all hydraulic and mechanical snubbers on safety-related systems, including the data at which the service life commences and associated installation and maintenance records.
10. Records of the Station Emergency Plan and implementing procedures audits.
11. Records of the Station Security Plan and implementing procedures audits.
12. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 180  
License No. NPF-4

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated July 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

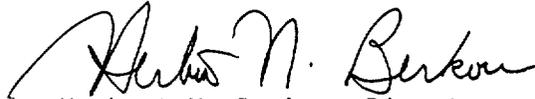
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.D.(2) of Facility Operating License No. NPF-4 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 180, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 1, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 180

TO FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

6-6  
6-9  
6-10  
6-13  
6-22

Insert Pages

6-6  
6-9  
6-10  
6-13  
6-22

## ADMINISTRATIVE CONTROLS

### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specifications 6.8.1, 6.8.2 and 6.8.3 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

## ADMINISTRATIVE CONTROLS

- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the SNSOC.

## AUDITS

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.

## ADMINISTRATIVE CONTROLS

- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.

### AUTHORITY

6.5.2.9 The MSRC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

### RECORDS

6.5.2.10 Records of MSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
- b. Reports of reviews with safety significant findings encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

## ADMINISTRATIVE CONTROLS

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- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f and changes thereto, shall be reviewed and approved by the SNSOC prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant supervisory staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed, and approved by the SNSOC within 14 days of implementation. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

## ADMINISTRATIVE CONTROLS

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### b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for monitoring, and
- (iii) Provisions for maintenance of sampling and analysis equipment.

### c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- (i) Identification of a sampling schedule for the critical variables and control points for these variables,
- (ii) Identification of the procedures used to measure the values of the critical variables,
- (iii) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser leakage,
- (iv) Procedures for the recording and management of data,
- (v) Procedures defining corrective actions for all control point chemistry conditions, and
- (vi) A procedure identifying (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

### d. Post-Accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- (i) Training of personnel,
- (ii) Procedures for sampling and analysis,
- (iii) Provisions for maintenance of sampling and analysis equipment.

**SPECIAL REPORTS**

6.9.2 Special reports shall be submitted to the Regional Administrator, Region II, within the time period specified for each report. These reports shall be submitted pursuant to the requirement of the applicable specification:

- a. Inservice Inspection Reviews, Specification 4.0.5, shall be reported within 90 days of completion.
- b. MODERATOR TEMPERATURE COEFFICIENT. Specification 3.1.1.4.
- c. RADIATION MONITORING INSTRUMENTATION. Specification 3.3.3.1, Table 3.3-6, Action 35.
- d. SEISMIC INSTRUMENTATION. Specifications 3.3.3.3 and 4.3.3.3.2.
- e. METEOROLOGICAL INSTRUMENTATION. Specification 3.3.3.4.
- f. Deleted.
- g. LOOSE PARTS MONITORING SYSTEMS. Specification 3.3.3.9.
- h. REACTOR COOLANT SYSTEM SPECIFIC ACTIVITY. Specification 3.4.8.
- i. OVERPRESSURE PROTECTION SYSTEMS. Specification 3.4.9.3.
- j. EMERGENCY CORE COOLING SYSTEMS. Specification 3.5.2 and 3.5.3.
- k. SETTLEMENT OF CLASS 1 STRUCTURES. Specification 3.7.12.
- l. GROUND WATER LEVEL - SERVICE WATER RESERVOIR. Specification 3.7.13.
- m. Deleted.
- n. RADIOACTIVE EFFLUENTS. As required by the ODCM.
- o. RADIOLOGICAL ENVIRONMENTAL MONITORING. As required by the ODCM.
- p. SEALED SOURCE CONTAMINATION. Specification 4.7.11.1.3.
- q. REACTOR COOLANT SYSTEM STRUCTURAL INTEGRITY. Specification 4.4.10. For any abnormal degradation of the structural integrity of the reactor vessel or the Reactor-Coolant System pressure boundary detected during the performance of Specification 4.4.10, an initial report shall be submitted within 10 days after detection and a detailed report submitted within 90 days after the completion of Specification 4.4.10.

## ADMINISTRATIVE CONTROLS

- r. **CONTAINMENT STRUCTURAL INTEGRITY.** Specification 4.6.1.6. For any abnormal degradation of the containment structure detected during the performance of Specification 4.6.1.6, an initial report shall be submitted within 10 days after the completion of Specification 4.6.1.6. A final report, which includes (1) a description of the condition of the liner plate and concrete, (2) inspection procedure, (3) the tolerance on cracking, and (4) the corrective actions taken, shall be submitted within 90 days after the completion of Specification 4.6.1.6.

### 6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE EVENTS and Special Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.
- i. Records of the Station Emergency Plan and implementing procedures audits.
- j. Records of the Station Security Plan and implementation procedures audits.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

OLD DOMINION ELECTRIC COOPERATIVE

DOCKET NO. 50-339

NORTH ANNA POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161  
License No. NPF-7

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company et al., (the licensee) dated July 20, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-7 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 161 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 1, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 161

TO FACILITY OPERATING LICENSE NO. NPF-7

DOCKET NO. 50-339

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Remove Pages

6-7  
6-10  
6-11  
6-14  
6-22

Insert Pages

6-7  
6-10  
6-11  
6-14  
6-22

## **ADMINISTRATIVE CONTROLS**

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### **MEETING FREQUENCY**

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### **QUORUM**

6.5.1.5 A quorum of the SNSOC consists of the Chairman or Vice-Chairman and two members including alternates.

### **RESPONSIBILITIES**

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specifications 6.8.1, 6.8.2 and 6.8.3 and changes thereto, 2) all programs required by Specification 6.8.4 and changes thereto, 3) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- d. Review of all proposed changes to Appendix "A" Technical Specifications and Appendix "B" Environmental Protection Plan. Recommended changes shall be submitted to the Station Manager.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and the MSRC.
- f. Review of all REPORTABLE EVENTS and Special Reports.
- g. Review of facility operations to detect potential nuclear safety hazards.
- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee or Station Manager.
- i. Deleted.
- j. Deleted.

## ADMINISTRATIVE CONTROLS

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### COMPOSITION

6.5.2.2 The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANSI/ANS 3.1-1979 Rev. 1 (Draft).

### ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### CONSULTANTS

6.5.2.4 Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

### MEETING FREQUENCY

6.5.2.5 The MSRC shall meet at least once per calendar quarter.

### QUORUM

6.5.2.6 The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

### REVIEW

6.5.2.7 The MSRC shall be responsible for the review of:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications or this Operating License.

## **ADMINISTRATIVE CONTROLS**

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- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
- g. Events requiring written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
- i. Reports and meetings minutes of the SNSOC.

## **AUDITS**

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
- b. The performance, training and qualifications of the entire facility staff.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50.
- e. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
- f. The Fire Protection Program and implementing procedures.
- g. An independent fire protection and loss prevention inspection and audit shall be performed utilizing an outside qualified fire consultant.
- h. The Radiological Environmental Monitoring Program and the results thereof.

## ADMINISTRATIVE CONTROLS

- i. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures.
- j. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes.

## AUTHORITY

6.5.2.9 The MSRC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

## RECORDS

6.5.2.10 Records of MSRC activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
- b. Reports of reviews with safety significant findings encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

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## ADMINISTRATIVE CONTROLS

### 6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the SNSOC and the results of this review shall be submitted to the Vice President-Nuclear Operations and the MSRC.

### 6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The facility shall be placed in at least HOT STANDBY within one hour.
- b. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within one hour. The Vice President-Nuclear Operations and the MSRC shall be notified within 24 hours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the Vice President-Nuclear Operations and the MSRC within 14 days of the violation.

### 6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- b. Refueling operations.

## ADMINISTRATIVE CONTROLS

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- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. PROCESS CONTROL PROGRAM implementation.
- h. OFFSITE DOSE CALCULATION MANUAL implementation.
- i. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of 6.8.1 above, except 6.8.1.d, 6.8.1.e, and 6.8.1.f and changes thereto, shall be reviewed and approved by the SNSOC prior to implementation and reviewed periodically as set forth in administrative procedures. Procedures of 6.8.1.d, 6.8.1.e, and 6.8.1.f shall be reviewed and approved as set forth in the facility's Security Plan, Emergency Plan, and section 6.5.1.6.m of the Technical Specifications, respectively. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant supervisory staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.
- c. The change is documented, reviewed, and approved by the SNSOC within 14 days of implementation. SNSOC approval shall be certified in writing by an Assistant Station Manager.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the recirculation spray, safety injection, chemical and volume control, gas stripper, and hydrogen recombiners. The program shall include the following:

- (i) Preventive maintenance and periodic visual inspection requirements, and
- (ii) Integrated leak test requirements for each system at refueling cycle intervals or less.

## **ADMINISTRATIVE CONTROLS (Continued)**

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### **6.10 RECORD RETENTION**

In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.1 The following records shall be retained for at least five years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. ALL REPORTABLE EVENTS and Special Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to Operating Procedures.
- f. Records of radioactive shipments.
- g. Records of sealed source leak tests and results.
- h. Records of annual physical inventory of all sealed source material of record.
- i. Records of the Station Emergency Plan and implementing procedures audits.
- j. Records of the Station Security Plan and implementation procedures audits.

6.10.2 The following records shall be retained for the duration of the Facility Operating License:

- a. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of facility radiation and contamination surveys.
- d. Records of radiation exposure for all individuals entering radiation control areas.
- e. Records of gaseous and liquid radioactive material release to the environs.
- f. Records of transient or operational cycles for those facility components identified in Table 5.7-1.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

TECHNICAL SPECIFICATION CHANGE

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA UNITS 1 & 2 AND SURRY UNITS 1 & 2

DOCKET NOS. 50-338/339 AND 50-280/281

1.0 INTRODUCTION

By letter dated July 20, 1993, Virginia Electric and Power Company (Virginia Power) requested changes to the Administrative Controls Section of the Technical Specifications (TS) for North Anna Units 1 & 2 and Surry Units 1 & 2. The substance of the changes is to delete the audit frequencies and Station Nuclear Safety and Operating Committee (SNSOC) review of the Emergency and Security Plans from the TS. The following description and evaluation apply to both Surry Units 1 & 2 and North Anna Units 1 & 2, except where noted, since the wording of the TS for both sites is nearly identical, except for Section numbers.

2.0 EVALUATION

a. Sections 6.1.C.1.f.9 & 10 (Surry), Sections 6.5.1.6.i & j (North Anna) - SNSOC Responsibilities - Virginia Power proposes to delete the requirement for the SNSOC to review the Plant Security Plan and the Emergency Plan (EP) and their implementing procedures.

The requirement to review the facilities' EP and implementing procedures and submit recommended changes to the Station Manager is maintained in Section 8.2.1 of each facility's EP.

For this and the following proposed changes, where a requirement is being deleted from the Surry or North Anna TS because it is redundant to the facility's EP, we note that 10 CFR 50.54(q) states in part "A licensee ... shall follow and maintain in effect emergency plans ..." ; "...licensee may make changes to these plans without Commission approval only if the changes do not decrease the effectiveness of the plans and the plans, as changed, continue to meet ..." ; and "Proposed changes that decrease the effectiveness of the approved emergency plan may not be implemented without application to and approval by the Commission."

The staff finds these changes acceptable as these requirements are in the facilities' Emergency Plan and Physical Security Plans and these changes are consistent with Section 5.5.1 of the Improved Standard Technical Specifications (ISTS).

b. Sections 6.1.C.2.h.1, 6.1.C.2.h.2, 6.1.C.2.h.11, 6.1.C.2.h.12 and 6.1.C.2.h.13 (Surry), Sections 6.5.2.8.a, 6.5.2.8.b, 6.5.2.8.k, 6.5.2.8.l and 6.5.2.8.m (North Anna) - Management Safety Review Committee (MSRC) Audits - Virginia Power proposes to delete the audit frequency of "at least once per 12 months." Sections 6.1.C.2.h.3 (Surry) and 6.5.2.8.c (North Anna) - MSRC Audits - Virginia Power proposes to delete the audit frequency of "at least once per 6 months." Sections 6.1.C.2.h.4 and 6.1.C.2.h.8 (Surry) and 6.5.2.8.d and 6.5.2.8.h (North Anna) - MSRC Audits - Virginia Power proposes to delete the audit frequency of "at least once per 24 months." Sections 6.1.C.2.h.9 (Surry) and 6.5.2.8.i (North Anna) - MSRC Audits - Virginia Power proposes to delete the term "annually" and reword these sections to also capture the substance of Sections 6.1.C.2.h.10 (Surry) and 6.5.2.8.j (North Anna). Section 6.1.C.2.h.10 (Surry) - MSRC Audits - Virginia Power has proposed to delete the phrase "at intervals no greater than 3 years," and combine this section with Section 6.1.C.2.h.9. Section 6.5.2.8.j (North Anna) - MSRC Audits - Virginia Power proposes to delete this section and combine the inspection and audit requirements, less the audit frequency, with Section 6.5.2.8.i.

The staff finds these changes acceptable as they are consistent with Section 5.5.2 of the ISTS.

c. Sections 6.1.C.2.h.5, 6.1.C.2.h.6 (Surry) and 6.5.2.8.e, 6.5.2.8.f (North Anna) - MSRC Audits - Virginia Power proposes to delete these audits which are with respect to the Emergency Plan and the Security Plan since these audits are specified in the respective Plans.

The TS requirement to audit the EP and implementing procedures at least once per 12 months is redundant to 10 CFR 50.54(t) which states an independent review and audit of the emergency program is to be conducted at least every 12 months. Additionally, Section 8.9 of each facility's EP states, in part, that an independent review of the emergency preparedness program shall be conducted at least once every 12 months in accordance with 10 CFR 50.54(t).

We find these changes acceptable as the requirement for the independent review and audit of the Emergency Program is redundant to 10 CFR 50.54(t) and is maintained in each facility's EP. In addition, the proposed changes are consistent with Section 5.5.2 of the ISTS.

d. As a consequence of deletions and the combining of sections, the sections have been renumbered and/or relettered.

The staff finds these changes acceptable as they are editorial in nature.

e. Section 6.8.2 (North Anna) - Procedures - Virginia Power proposes to revise this section to require approval of the facility's Security Plan and Emergency Plan as stated within the Plans in lieu of approval by the SNSOC. In Section 8.2.1 of the North Anna EP, the review and approval of the EP and implementing procedures is specified as discussed in 2.a above.

We find the change acceptable for the same reason set forth in 2.a above and it is consistent with Section 5.5.2 of the ISTS.

f. Sections 6.5.B.10, 6.5.B.11 (Surry) and 6.10.1.i, 6.10.1.j (North Anna) - Records Retention - Virginia Power proposes to revise this section to delete the phrase "of the annual audit" and to add the word "audits."

Under Section 6.5, "Station Operating Records," Section 6.5.B of the Surry TS, it is specified that records relative to specific items as listed in this section are to be retained for the life of the plant. One of these items, TS 6.5.B.10, states that this includes records of the annual audit of the Station Emergency Plan and implementing procedures. The proposed change deletes "annual audit of the" and the word "audit" is to be inserted at the end of the sentence such that it would read:

"10. Records of the Station Emergency Plan and implementing procedures audit."

In 10 CFR 50.54(t), which provides for the independent review of the emergency preparedness program, it is specified that "... The results of the review, ... shall be documented ... and retained for a period of five years." The Surry TS indicate that these records will be retained for the life of the plant.

The deletion of the word "annual" indicates that all reviews of the emergency preparedness program are to be retained for the life of the plant. Additionally, the requirement to retain records of the review of the Emergency Preparedness program for 5 years is specified in Section 8.9 of the Surry EP. Under Section 6.10 "Record Retention," of the North Anna TS, these same changes are proposed for TS 6.10.1.i. However, North Anna TS Subsection 6.10.1 indicates that the records of the EP and implementing procedures review are records to be retained for at least 5 years. Section 8.9 of the North Anna EP specifies that the following records shall be filed in Station Records and maintained for 5 years:

- a) The review results and recommended improvements
- b) The answers to the recommended improvements
- c) A description of the corrective actions taken

We find the changes in the Surry TS acceptable because the requirement for the retention of the review of the Emergency Preparedness program for the life of the plant is more stringent than that specified in the regulations. We find the changes in the North Anna TS acceptable because the requirement for retention of the Emergency Plan and implementing procedures review records for 5 years is commensurate with the regulatory requirement. In addition, the changes are acceptable as they reflect the other changes that deleted annual audits.

g. In Section 6.4.H of the Surry TS, the sentence "An annual review of the Emergency Plan will be performed." is to be deleted. Section 8.2.1 of the Surry EP states that the Station Coordinator, Emergency Planning, shall review the Surry EP and its implementing procedures at least every 12 months and certify that they are adequate and current.

We find this change acceptable as the requirement for the annual review of the EP is maintained in the Surry EP.

h. Surry TS Section 6.4.I required implementation of an industrial security program and performance of an annual review of the program. The licensee has proposed deleting TS 6.4.I since the requirements for these activities are stated in the CFR and are included in the Surry Power Station Security Plan.

We find this change acceptable as the requirements for the activities are stated in the CFR and are included in the Surry Power Station Security Plan.

i. Surry TS Section 6.4.J required implementation of the fire protection program and implementing procedures. This section also required a review of the program at least once every two years. The licensee has proposed deleting TS 6.4.J since the requirements exist in other documents and, therefore, the requirements are neither altered nor diminished.

We find this change acceptable since the requirements for the activities are stated in the CFR and Surry license condition 3.I.

### 3.0 CONCLUSION

We find the requested changes acceptable as they reflect the ISTS and the appropriate part of Section 17.3 of NUREG-0800, the Standard Review Plan. In many cases, the requirements that are being deleted from TS exist outside of TS and changes to the requirements are controlled by means acceptable to the staff.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

### 5.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR

51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments (58 FR 46242).

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