

July 11, 1995

Mr. J.P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

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SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: NUMBER OF
CELLS IN BATTERIES (TAC NOS. M91813 AND M91814)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment No. 201 to Facility Operating License No. DPR-32 and Amendment No. 201 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application transmitted by letter dated November 22, 1994.

These amendments change the Technical Specifications to delete unnecessary descriptive phrases regarding the number of cells in the station and emergency diesel generator batteries.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Bart C. Buckley, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 201 to DPR-32
2. Amendment No. 201 to DPR-37
3. Safety Evaluation

cc w/enclosures:

See next page

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Mr. J. P. O'Hanlon
Virginia Electric and Power Company

Surry Power Station
Units 1 and 2

cc:

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DATED: July 11, 1995

AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1
AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

Docket File

NRC & Local PDRs

PDII-2 Reading

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 201
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 22, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 201 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 11, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 201
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated November 22, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 201, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 11, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 4.6-3

TS 4.6-4

Insert Pages

TS 4.6-3

TS 4.6-4

C. Station Batteries**1. Tests and Frequencies**

- a. The specific gravity, electrolytic temperature, cell voltage of the pilot cell in each battery, and the D.C. bus voltage of each battery shall be measured and recorded weekly.
- b. Each month the voltage of each battery cell in each battery shall be measured to the nearest 0.01 volts and recorded.
- c. Every 3 months the specific gravity of each battery cell, the temperature reading of every fifth cell, the height of electrolyte of each cell, and the amount of water added to any cell shall be measured and recorded.
- d. Twice a year, during normal operation, the battery charger shall be turned off for approximately 5 min and the battery voltage and current shall be recorded at the beginning and end of the test.
- e. During the normal refueling shutdown each battery shall be subjected to a simulated load test without battery charger. The battery voltage and current as a function of time shall be monitored.
- f. During the refueling outages connections shall be checked for tightness and anti-corrosion coating shall be applied to interconnections.

2. Acceptance Criteria

- a. Each test shall be considered satisfactory if the new data when compared to the old data indicate no signs of abuse or deterioration.

- b. The load test in (d) and (e) above shall be considered satisfactory if the batteries perform within acceptable limits as established by the manufacturers discharge characteristic curves.

D. EMERGENCY DIESEL GENERATOR BATTERIES

1. Tests and Frequencies

- a. The specific gravity, electrolytic temperature, cell voltage of the pilot cell in each battery and the D.C. bus voltage of each battery shall be measured and recorded weekly.
- b. Each month the voltage of each battery cell in each battery shall be measured to the nearest 0.01 volts and recorded.
- c. Every 3 months the specific gravity of each battery cell, the temperature reading of every fifth cell, the height of electrolyte of each cell, and the amount of water added to any cell shall be measured and recorded.
- d. At refueling each battery shall be subjected to a normal load or simulated load test without battery charger. The battery voltage and current as a function of time shall be monitored.
- e. Each refueling connections shall be checked for tightness and anti-corrosion coating shall be applied to inter-connections.

2. Acceptance Criteria

- a. Each test shall be considered satisfactory if the new data when compared to the old data indicate no signs of abuse or deterioration.
- b. The load test in (d) above shall be considered satisfactory if the batteries perform within acceptable limits as established by the manufacturers discharge characteristic curves.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated November 22, 1994, Virginia Electric and Power Company proposed changes to Technical Specifications (TS) 4.6.C.1.a and b and 4.6.D.1.a and b for the Surry Power Station, Units 1 and 2. These specifications use the descriptive phrases "60 cell battery" and "56 cell battery" to refer to the station battery and the emergency diesel generator (EDG) battery, respectively. The purpose of this revision is to delete unnecessary descriptive phrases regarding the number of cells in the station and EDG batteries. Deletion of the descriptive reference will eliminate the need for an interpretation regarding operation of a battery with less than the total number of installed cells operable.

2.0 EVALUATION

The TS for Surry Power Station include surveillance requirements for the station and EDG batteries to ensure that the batteries are capable of performing their intended safety functions. These specifications use the descriptive phrases "60 cell battery" and "56 cell battery" to refer to the station battery and the emergency diesel battery, respectively. The licensee proposes to delete descriptive phrases regarding the number of cells in station and EDG batteries. Deletion of the descriptive reference will preclude future misunderstandings if a situation arises that requires plant operation with less than the total number of installed cells in a particular battery. The licensee states that the reference to "60 cell" or "56 cell" is merely a descriptive phrase. The number of cells is not intended as a technical requirement, nor is it relevant in determining battery operability. If the voltage and capacity of the batteries are adequate to perform the intended safety function, the batteries are considered operable. Operability of individual cells is determined by appropriate surveillance tests and acceptance criteria in accordance with the TS.

The specific changes to the TS are as follows:

TS 4.6.C.1.a and b are revised to delete the words "60 cell" before the word "battery" in each specification; and TS 4.6.D.1.a and b are revised to delete the words "56 cell" before the word "battery" in each specification.

On the basis of this information, the staff concludes that deletion of the descriptive references will eliminate the need for an interpretation regarding operation of a battery with less than the total number of installed cells operable. As long as the voltage and capacity of the batteries are adequate to perform their safety function, the batteries are considered operable. Therefore, the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (60 FR 18630). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: O. Chopra

Date: July 11, 1995