

August 2, 1994

Docket Nos. 50-280
and 50-281

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Mr. J. P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

Dear Mr. O'Hanlon:

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: SURVEILLANCE
FREQUENCY OF CONTROL ROD MOTION TESTING (TAC NOS. M89327 AND M89328)

The Commission has issued the enclosed Amendment No. 192 to Facility Operating License No. DPR-32 and Amendment No. 192 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated April 19, 1994.

These amendments will modify the surveillance frequency of the control rod motion testing from monthly to quarterly.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

This completes our effort on this issue and we are, therefore, closing out TAC Nos. M89327 and M89328.

Sincerely,

(Original Signed By)
Bart C. Buckley, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 192 to DPR-32
2. Amendment No. 192 to DPR-37
3. Safety Evaluation

cc w/enclosures:

See next page

OFFICE	LA:PDII-2	PM:PDII-2	AD:PDII-2	OGC	
NAME	ETana <i>ETD</i>	BBuckley <i>BCB</i>	VMcCree <i>Jms</i>	<i>CRW</i>	
DATE	07/22/94	07/22/94	07/22/94	07/25/94	

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Mr. J. P. O'Hanlon
Virginia Electric and Power Company

Surry Power Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated April 19, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Bart C. Buckley for

Victor M. McCree, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 2, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 192
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated April 19, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 192, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Victor M. McCree for

Victor M. McCree, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 2, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Page

Insert Page

TS 4.1-9b

TS 4.1-9b

TABLE 4.1-2A

MINIMUM FREQUENCY FOR EQUIPMENT TESTS

<u>DESCRIPTION</u>	<u>TEST</u>	<u>FREQUENCY</u>	<u>FSAR SECTION REFERENCE</u>
1. Control Rod Assemblies	Rod drop times of all full length rods at hot conditions	Prior to reactor criticality: a. For all rods following each removal of the reactor vessel head b. For specially affected individual rods following any maintenance on or modification to the control rod drive system which could affect the drop time of those specific rods, and c. Each refueling shutdown.	7
2. Control Rod Assemblies	Partial movement of all rods	Quarterly	7
3. Refueling Water Chemical Addition Tank	Functional	Each refueling shutdown	6
4. Pressurizer Safety Valves	Setpoint	Per TS 4.0.3	4
5. Main Steam Safety Valves	Setpoint	Per TS 4.0.3	10
6. Containment Isolation Trip	* Functional	Each refueling shutdown	5
7. Refueling System Interlocks	* Functional	Prior to refueling	9.12
8. Service Water System	* Functional	Each refueling shutdown	9.9
9. Fire Protection Pump and Power Supply	Functional	Monthly	9.10
10. Primary System Leakage	* Evaluate	Daily	4
11. Diesel Fuel Supply	* Fuel Inventory	5 days/week	8.5
12. Boric Acid Piping Heat Tracing Circuits	* Operational	Monthly	9.1
13. Main Steam Line Trip Valves	Functional (Full Closure)	Before each startup (TS 4.7) The provisions of Specification 4.0.4 are not applicable	10

Amendment Nos. 192 and 192

TS 4.1-9b



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated April 19, 1994 Virginia Electric and Power Company (the licensee) requested changes to the Technical Specifications (TS) for the Surry Power Station Units 1 and 2. These changes will modify the surveillance frequency of the control rod motion testing from monthly to quarterly in accordance with Generic Letter 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation," dated September 27, 1993.

TS require periodic testing of each control and shutdown control rod assembly bank in the core during power operation to ensure that the control rod assemblies are trippable. This testing requires partial movement of each control rod assembly not fully inserted into the core. This is typically done at or near full power, one bank at a time.

2.0 EVALUATION

Control rod motion testing was reviewed as part of the evaluation of power testing requirements. NUREG-1366 determined that electrical problems with the control rod drive system were the major contributor to rod motion failure. Mechanical problems were found to be less common than electrical problems. Most stuck rods were discovered during rod drop testing or during plant startup after refueling. Based on the generic evaluation it was concluded in GL 93-05 that the surveillance interval for control rod motion testing could be increased to once every 92 days without any decrease in plant safety.

Neither Surry unit has identified any stuck rods during routine surveillance testing. However, both Surry units have experienced Rod Control Drive System failures during surveillance testing. In these cases the control rods could not be moved by the control rod system but they remained trippable and continued to be capable of performing their intended safety function.

Thus, from review of this operating experience, the proposed control rod surveillance test improvements specified in the NUREG are appropriate and applicable to both Surry units.

3.0 TECHNICAL SPECIFICATION CHANGES

Table 4.1-2A Change the frequency of the Control Rod Assemblies testing from monthly to quarterly.

4.0 SUMMARY

Based on the staff evaluation in Section 2.0 above, the staff concludes that the proposed TS changes are acceptable. These changes are all consistent with the Standard Technical Specifications.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

6.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding (59 FR 27070). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Chatterton

Date: August 2, 1994

DATED: August 2, 1994

AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1
AMENDMENT NO. 192 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

Distribution

Docket File

NRC & Local PDRs

PDII-2 Reading

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