1	y: FENOC BVPS Unnation Level (circle	
	dministrative Topic/Subject Description	Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions
A.1	Conduct of	Verify Natural Circulation Flow [JPM]  K/A 193008 K1.21 3.9
	Operations	K/A 2.1.7 3.7
	Conduct of	Perform QPTR Calculation [JPM]
	Operations	K/A 015 A1.04 3.5
		K/A 015 A4.02 3.9
A.2	Equipment	Prepare a Clearance Tagout [JPM]
	Control	K/A 2.2.13 3.6
A.3	Radiation	Determine 10CR20 Exposure Limits [Q]
	Control	K/A 2.3.1 2.6
		Knowledge of Radiological Key Control [Q]
		K/A 2.3.2 2.5
A.4	Emergency	Knowledge of ERO Responsibilities [Q]
	Preparedness	K/A 2.4.12 3.4
		Knowledge of Response Facilities Activation [Q]
		K/A 2.4.29 2.6

II -	r: FENOC BVPS Unation Level (circle	······································
T .	dministrative opic/Subject Description	Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions
A.1	Conduct of Operations	Verify Natural Circulation Flow [JPM]  K/A 193008 K1.21 4.2  K/A 2.1.7 4.4
	Conduct of Operations	Perform an ECP Calculation [JPM]  K/A 192008 K1.07 3.6  K/A 2.1.23 4.0
A.2	Equipment Control	Review a Clearance Tagout [JPM]  K/A 2.2.13 3.8
A.3	Radiation Control	Determine 10CFR20 Exposure Limits [Q]  K/A 2.3.1 3.0  Knowledge of Radiological Key Control [Q]  K/A 2.3.2 2.9
A.4	Emergency Preparedness	Classify an EPP Event [JPM]  K/A 2.4.41 4.1

## Control Room Systems and Facility Walk-Through Test Outline

Facility: FENOC BVPS Unit 1 Date of Examination: 8/6/01 Exam Level (circle one): RO / SRO(I) / SRO(U) Operating Test No.: 1LOT4

## **B.1 Control Room Systems**

System / JPM Title	Type Code*	Safety Function
b. Start 1RC-P-1A, 'A' Reactor Coolant Pump	D, A, S, L	4
b. Fill 1SI-TK-1A, Accumulator 1A	D, S	2
c. Emergency Borate Reactor Coolant System	D, A, S	1
d. Manually Actuate CIB	D, A, S	5
e. Offsite to Onsite Power Breaker Alignment Verification 7F 5-Inchronize the Main Generator	N, S	6
f. LHSI Boric Acid Flow Path Verification 7/5/01 Depressurize the RCS using PORVs	N, S, TF 8/3/01	3
g. Secure Unnecessary Auxiliary Feedwater Pumps	D, S	4
B.2 Facility Walk-Through		
a. Start the Hydrogen Recombiners	D, R	5
b. Locally Start the No. 1 EDG	D, A	6
c. Locally Makeup to the Spent Fuel Pool	D	8

 $<sup>^{\</sup>star}$  Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (À)Iternate path, (C)ontrol room, (S)imulator, (L)ow-Power, (R)CA

racility:	FENOC BVPS Unit I	Scenario I	No.: _1_
Examiner	s:		rators:
Turnover:	itions: IC-187.  The plant is at 27% power, Equility	brium Xe, BOL, RO	CS boron at 1550 PPM. CB 'D' is at 145 steps. FW-P-3A is d will not be returned this shift. Severe thunderstorms are
Event No.	Malfunction No.	Event Type*	Event Description
0	N/A	N/R	Raise power to 100%.
1	PRS08D	I .	PZR pressure transmitter PT-RC-444 fails high.
2	MSS16B	I	SG control pressure transmitter fails low.
3	SIS05A	С	Running charging pump trips, manually start STBY pump.
4	RCS10A	С	RCP 1A high vibration with slow increase in #1 seal leakoff flow.

 $\mathbf{C}$ 

 $\mathbf{C}$ 

M

ATWS, reactor trip failure, both automatic &

AFW pumps fail to start automatically, can be

"B" SG steam break in CNMT. Steam break

occurs after transition back to E-0 from FR-S.1.

manual, FR-S.1 required.

started manually.

RCS05A

CRF12A

CRF12B

INH20

INH21 INH35

MSS01B

5

6

7

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Facility:	FENOC BVPS Unit 1	Sco	enario No.: <u>2</u>	Op-Test No.: 1L	<u>.OT4</u>
Examiner	3:		Operators:		<del></del>
	· ·				<del></del>
Initial Cond	itions: IC-171.				

Turnover: The plant is at 75% power, Equilibrium Xe, BOL, RCS boron at 1346 PPM. CB 'D' is at 181 steps. FW-P-3A is OOS and will not be returned this shift. The #1 EDG is OOS and will not be returned this shift. Severe thunderstorms are forecasted.

Event No.	Malfunction No.	Event Type*	Event Description
0	N/A	N/R	Raise power to 100%.
1	RCS03A	С	Small tube leak SG "A". Plant remains on line.
2	CHS20B	I	VCT level transmitter fails high.
3	TUR18A	I	Selected 1 <sup>st</sup> stage pressure transmitter fails low.
4	FWM09A MAL05	С	Main feed reg. valve oscillates in automatic, can be controlled in manual.
5	TUR15	M	Load rejection due to governor valve position limiter failure.
6	EPS18	C/M	Main transformer failure causes generator/turbine and reactor trip. Automatic reactor trip fails, manual trip functions, FW-P-3B fails and cannot be started, FW-P-2 trips on S/U but can be reset.
7	MSS18C	М	Main steam line break "C" SG outside CNMT, upstream of MSIV. Auto SI fails, manual SI functions.

<sup>\* (</sup>N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Facility: FENOC BV	PS UNIT 1		D	ate of	Exa	ım: 8	3/06/0	01			E	xam l	_evel: RO
Tier	Group				K//	A Ca	tegor	y Poi			Point Total		
		K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	
1.	1	2	1	5				3	4			1	16
Emergency & Abnormal Plant	2	3	3	4				4	2			1	17
Evolutions	3	1	0	1				1	0			0	3
	Tier Totals	6	4	10				8	6			2	36
2.	1	3	0	2	4	1	1	4	4	2	1	1	23
Plant Systems	2	3	1	2	2	1	1	0	3	2	4	1	20
	3	1	1	1	0	0	0	0	2	1	2	0	8
	Tier Totals	7	2	5	6	2	2	4	9	5	7	2	51
3. Generic Kn	owledge and	ties		Ca	ıt 1	Ca	t 2	Са	t 3	Ca	t 4		
					4	4	4	ļ	2	2	3	3	13

Note: 1.

- 1. Ensure that at least two topics from every K/A category are sampled within each tier (i.e., the "Tier Totals" in each K/A category shall not be less than two).
- 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate ±1 from that specified in the table based on NRC revisions. The final exam must total 100 points.
- 3. Select topics from many systems; avoid selecting more than two or three K/A topics from a given system unless they relate to plant-specific priorities.
- 4. Systems/evolutions within each group are identified on the associated outline.
- 5. The shaded areas are not applicable to the category/tier.
- 6.\* The generic K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.
- 7. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings for the RO license level, and the point totals for each system and category. K/As below 2.5 should be justified on the basis of plant-specific priorities. Enter the tier totals for each category in the table above.

ES-401	<del></del>	E	merger	ncy and	PWR Abnorn	RO Ex	ramination Outline nt Evolutions - Tier 1/Group 1	Form ES	;-401-4
E/APE # / Name / Safety Function	K1	K2	кз	A1	A2	G	K/A Topic(s)	Imp.	Points
000005 Inoperable/Stuck Control Rod / 1								mip.	Points
000015/17 RCP Malfunctions / 4		1					AK2.07 Interrelationship between RCP malfunctions and RCS seals	2.9	<del>                                     </del>
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4							p = 1.05, 7(0) mananadono una 700 seas	2.5	
000024 Emergency Boration / 1					2		AA2.02 Use of manual boration valve	3.9	1
000026 Loss of Component Cooling Water / 8					26		AA2.02 Cause of possible CCW loss	2.9	1
000027 Pressurizer Pressure Control System Malfunction / 3	27					99	AK1.02/2.1.32 Expansion of liquid/PZR P&L	2.8/3.4	2
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4				28			AA1.04 Isolation of main steam lines	4.3	1
CE/A11; W/E08 RCS Overcooling - PTS / 4					3		EA2.2 Adherence to procedures and operation within limits	3.5	1
*000051 Loss of Condenser Vacuum / 4			29				AK3.01 Steam dump vacuum	2.8	1
000055 Station Blackout / 6			30		31		EK3.02/EA2.03 Actions for loss of power / Actions to restore power	4.3/3.9	2
*000057 Loss of Vital AC Elec. Inst. Bus / 6			32				AK3.01 Actions for loss of instrument bus	4.3/3.9	1
*000062 Loss of Nuclear Service Water / 4	$oldsymbol{ol}}}}}}}}}}}}}}}}}$		34				AK3.03 Actions for loss of service water	4.0	1 1
000067 Plant Fire On-site / 9	35						AK1.02 Fire fighting	3.1	1
000068 (BW/A06) Control Room Evac. / 8			36				AK3.18 Actions for control room evacuation	4.2	1
000069 (W/E14) Loss of CTMT Integrity / 5				37			AA1.01 Monitor isolation valves	3.5	1
000074 (W/E06&E07) Inad. Core Cooling / 4				38			EA1.06 Operate RCPs	3.6	1
BW/E03 Inadequate Subcooling Margin / 4	1							3.0	
*000076 High Reactor Coolant Activity / 9									
BW/A02&A03 Loss of NNI-X/Y / 7									
K/A Category Totals:	2	1	5	3	4	1	Group Point Total:		16

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ES-401			Emerger	P) ncy and A	WR RO E	xaminat Plant Ev	tion Outline volutions - Tier 1/Group 2	Form ES-401	1-4
E/APE # / Name / Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Points
000001 Continuous Rod Withdrawal / 1							10.1.1000(0)	mip.	1 Ollits
000003 Dropped Control Rod / 1									1
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1									
BW/A01 Plant Runback / 1									<del>-</del>
BW/A04 Turbine Trip / 4									<b>+</b>
000008 Pressurizer Vapor Space Accident / 3			39				AK3.02 PORV/Code safety exit temperature	3.6	1 1
000009 Small Break LOCA / 3				40			EA1.01 Monitor RCS press./temp.	4.4	1
000011 Large Break LOCA / 3		41					EK2.02 LOCA and pump relationship	2.6	
W/E04 LOCA Outside Containment / 3				42			EA1.1 Components and Functions	4.0	1
BW/E08; W/E03 LOCA Cooldown/Depress. / 4				4			EA1.2 Operating characteristics of facility	3.7	1
W/E11 Loss of Emergency Coolant Recirc. / 4					43		EA2.2 Adherence to procedures	3.4	1
W/EO1 & E02 Rediagnosis & SI Termination / 3								3.4	<u>'</u>
000022 Loss of Reactor Coolant Makeup / 2				44			AA1.08 VCT level	3,4	1
000025 Loss of RHR System / 4	45						AK1.01 Loss of RHR during all modes	3.9	1
000029 Anticipated Transient w/o Scram / 1							an modes		<del>  '</del> -
000032 Loss of Source Range NI / 7		46					AK2.01 Switch positions	2.7	1
000033 Loss of Intermediate Range NI / 7	47						AK1.01 Effects of voltage changes	2.7	1
*00037 Steam Generator Tube Leak / 3						48	2.2.25 Tech. Spec. reason	2.5	1 1
000038 Steam Generator Tube Rupture / 3			49				EK3.06 Bases for actions of EOPs	4.2	1
000054 (CE/E06) Loss of Main Feedwater / 4								7.2	<del>                                     </del>
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	50						EK1.2 Loss of heat sink EOP actions	3.9	1
000058 Loss of DC Power / 6					5		AA2.01 Determine substitute power source	3.7	1
000059 Accidental Liquid RadWaste Rel. / 9		33					AK2.01 radioactive liquid monitors	2.7	1
000060 Accidental Gaseous Radwaste Rel. / 9								4./	<del> </del>
*00061 ARM System Alarms / 7			51				AK3.02 Guidance in alarm response	3.4	1
W/E16 High Containment Radiation / 9			52				EK3.1 Reasons for operating characteristics	2.9	1
CE/E09 Functional Recovery							The state of the s	2.9	<del>                                     </del>
K/A Category Point Totals:	3	3	4	4	2	1	Group Point Total:		17

ES-401		<del>,</del>	Emergency and Abnormal Plant Evolutions - Tier 1/Group 3													
E/APE # / Name / Safety Function	K1	K2	кз	A1	A2	G	K/A Topic(s)	Imp.	Points							
000028 Pressurizer Level Malfunction / 2				6			AA1.03 RCP and seal water system	2.9	1							
000036 (BW/A08) Fuel Handling Accident / 8							,	2.9	<del>                                     </del>							
000056 Loss of Off-site Power / 6																
000065 Loss of Instrument Air / 8			7				AK3.08 Actions in EOPs	3,7	1							
BW/E13&E14 EOP Rules and Enclosures						1		3.1								
BW/A05 Emergency Diesel Actuation / 6									<del>                                     </del>							
BW/A07 Flooding / 8									<del> </del>							
CE/A16 Excess RCS Leakage / 2																
W/E13 Steam Generator Over-pressure / 4	53						EK1.2 Normal, Abnormal and EOPs	3.0								
W/E15 Containment Flooding / 5								3.0	1							
		<u> </u>														
K/A Category Point Totals:	1	0	1	1	0	0	Group Point Total:		3							

ES-401	PWR RO Examination Outline Plant Systems - Tier 2/Group 1  System # / Name  K4 K2 K2 K2 K3 K4 K7														
System # / Name	K1	K2	КЗ	K4	K5	K6	A1	A2	АЗ	A4	G	K/A Topic(s)	Imp.	Point	
001 Control Rod Drive			8				98					K3.02/A1.01 RCS relationship/Predict Tave response	3.4/3.8	2	
003 Reactor Coolant Pump			54_					55				K3.01/A2.02 RCS / Abnormal shutdown of RCP	3.7/3.7	2	
004 Chemical and Volume Control		ļ			<u> </u>	56		57				K6.07/A2.18 HX & Cond / High VCT level	2.7/3,1	2	
013 Engineered Safety Features Actuation			<u> </u>	58			9					K4.13/A1.02 MFW Isol. / CNMT press. & temp.	3.7/3.9	2	
015 Nuclear Instrumentation	59								10			K1.01/A3.02 RPS / Annunciator		1	
017 In-core Temperature Monitor					60							K5.02 Saturation & subcooling	4.1/3.7	2	
022 Containment Cooling							61					A1.01 Monitor CNMT temp.	3.7	1	
025 Ice Condenser												74.5 FWOMO ONE TEMP.	3.6	1	
056 Condensate								62				A2.04 Loss of condensate pumps	1 26	<del></del>	
059 Main Feedwater				63						64		K4.08/A4.12 Mismatch S/F signal/Initiation of FWI	2.5/3.4	1 2	
061 Auxiliary/Emergency Feedwater	65							66		<u> </u>				-	
068 Liquid Radwaste	67							00			11	K1.01/A2.04 SG system / Pump failure	4.1/3.4	2	
*071 Waste Gas Disposal				68					12		11	K1.07/2.3.11 Sources of input/Control releases	2.7/2.7	2	
*072 Area Radiation Monitoring				69			13		12			K4.01/A3.02 Decay Tank / Press. regulator K4.01/A1.01 Cont. isol. / Rad. levels	2.6/2.8	2	
												R4.0 I/A 1.01 Cont. Isol. / Rad. levels	3.3/3.4		
														_	
														<del> </del>	
	_				_										
											-				
K/A Category Point Totals:	3	0	2	4	1	1	4	4	2	1	1	Group Point Total:		23	

ES-401	Plant Systems - Tier 2/Group 2													
System # / Name	K1	K2	КЗ	K4	K5	K6	A1	A2	_A3	A4_	G	K/A Topic(s)	Imp.	Points
*002 Reactor Coolant				70_		14						K4.02/K6.03 Monitor level / Loss of level	3.5/3.1	2
006 Emergency Core Cooling					74					71		K5.02/A4.08 Accum. press. & level/Operate ESF Systems	2.8/4.0	2
010 Pressurizer Pressure Control			16							72		K3.01/A4.01 Loss of PCS on RCS / Monitor spray valves	3.8/3.7	2
011 Pressurizer Level Control	73											K1.03 PZR PCS	3.7	1
012 Reactor Protection			<u> </u>											
*014 Rod Position Indication										17		A4.01 Rod selection control	3.3	1
016 Non-nuclear Instrumentation								15				A2.02 Loss of power supply	2.9	1
026 Containment Spray	18											K1.02 CSS and cooling water	4.1	1
029 Containment Purge														
*033 Spent Fuel Pool Cooling								75_				A2.02 Loss of SFPCS	2.7	1
035 Steam Generator			76									K3.03 Effect on secondary systems	3.0	1
039 Main and Reheat Steam				77								K4.02 Use of Tave	3.1	1
055 Condenser Air Removal									78			A3.03 Automatic diversion to containment	2.5	1
062 AC Electrical Distribution						<u> </u>		79				A2.05 Re-energize dead bus	2.9	1
063 DC Electrical Distribution	19		ļ									K1.03 125VDC connection	2.9	1
064 Emergency Diesel Generator		20							21			K2.02/A3.01 Fuel oil pump / Autostart of EDG	2.8/4.1	2
073 Process Radiation Monitoring										80		A4.02 Rad. monitor control panel	3.7	1
075 Circulating Water											81	2.1.32 Explain system P&Ls	3.4	1
079 Station Air														
086 Fire Protection														
K/A Category Point Totals:	3	1	2	2	1	1	0	3	2	4	1	Group Point Total:	1	20

ES-401					ı	PWR F	RO Exa	minatio - Tier 2	n Outlin /Group	ie 3			Form ES-401	-4
System # / Name	K1	K2	КЗ	K4	K5	K6	A1	A2	А3	A4	G	K/A Topic(s)	Imp.	Points
*005 Residual Heat Removal		22						82	7.0	/	J	K2.03/A2.02 Power supply/ Pressure protection in CSD	2.7/3.5	2
007 Pressurizer Relief/Quench Tank	_							23		24		A2.05/A4.10 PRT high pressure limits/Recognize leaking PORV	3.2/3.6	2
008 Component Cooling Water	<b>_</b>		83									K3.03 Loss of CCWS on RCP	4.1	1
027 Containment Iodine Removal														
028 Hydrogen Recombiner and Purge Control		L												
034 Fuel Handling Equipment														
041 Steam Dump/Turbine Bypass Control										25		A4.08 Monitor SDS control of Tave	3.0	1
045 Main Turbine Generator	84											K1.20 Protective system	3.4	1
*076 Service Water									85			A3.02 Emergency heat loads	3.7	1
*078 Instrument Air														
103 Containment								ļ						
K/A Category Point Totals:	1	1	1	0	0	0	0	2	1	2	0	Group Point Total:	12800-1	8
Plant-Specific Priorities														
System / Topic						Reco	mmen	ided Re	eplacer	nent fo	r	Reason		Points
				_					<del></del>					
Diant Daniel Di dia Tala (II. II. II.											· · · · · ·			
Plant-Specific Priority Total: (limit 10)														

Facility: FEN	OC BVPS UI	NIT 1 Date of Exam: 8/06/01 Exam Le	vel: RO	
Category	K/A#	Торіс	lmp.	Points
	2.1.1 86	Knowledge of conduct of operations requirements	3.7	1
Conduct of	2.1.22 <b>87</b>	Ability to determine mode of operation	2.8	1
Operations	2.1.23 88	Ability to perform plant procedures, all modes	3.9	1
	2.1.27 <b>89</b>	Knowledge of system purpose/function	2.8	1
	Total			4
	2.2.12 <b>90</b>	Knowledge of surveillance procedures	3.0	1
	2.2.13 <b>91</b>	Knowledge of tagging and clearance procedures	3.6	1
Equipment Control	2.2.26 <b>92</b>	Knowledge of refueling administrative requirements	2.5	1
	2.2.33 <b>93</b>	Knowledge of control rod programming	2.5	1
	Total			4
	2.3.1 <b>94</b>	Knowledge of 10 CFR20 related radiation controls	2.6	1
	2.3.2 <b>95</b>	Knowledge of ALARA	2.5	1
Radiation Control				
	Total			2
	2.4.6 <b>97</b>	Knowledge of symptom based EOP strategy	3.1	1
Emergency	2.4.16 <b>96</b>	Knowledge of EOP hierarchy	3.0	1
Procedures/ Plan	2.4.10 <b>100</b>	Knowledge of alarm response procedures	3.0	1
	Total			3
Tier 3 Point Tot	al (RO)			13

Facility: FENOC I	Facility: FENOC BVPS Unit 1 Date of Exam: 8/06/01 Exam Level:												RO
Tier	Croup				K/A	\ Cat	egory	Poin	its				Point
Her	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G *	Total
1.	1	3	3	6				5	6			1	24
Emergency & Abnormal Plant	2	5	1	5				3	1			1	16
Evolutions	3	1	1	0				0	1			0	3
	Tier Totals	9	5	11				8	8			2	43
	1	4	0	1	4	1	1	2	4	0	1	1	19
2. Plant	2	1	2	1	2	1	1	0	4	2	2	1	17
Systems	3	1	0	1	0	0	0	0	1	1	0	0	4
	Tier Totals	6	2	3	6	2	2	2	9	3	3	2	40
3. Generic K	nowledge a	ınd Ab	ilities		Ca	ıt 1	Ca	ıt 2	Ca	at 3	C	at 4	
	-					6	į	5		3		3	17

Note: 1.

- Ensure that at least two topics from every K/A category are sampled within each tier (i.e., the "Tier Totals" in each K/A category shall not be less than two).
- 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate ±1 from that specified in the table based on NRC revisions. The final exam must total 100 points.
- 3. Select topics from many systems; avoid selecting more than two or three K/A topics from a given system unless they relate to plant-specific priorities.
- 4. Systems/evolutions within each group are identified on the associated outline.
- 5. The shaded areas are not applicable to the category/tier.
- 6.\* The generic K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.
- 7. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings for the RO license level, and the point totals for each system and category. K/As below 2.5 should be justified on the basis of plant-specific priorities. Enter the tier totals for each category in the table above.

ES-401					PW	R SRO	Examination Outline	Form ES-4	101-3
			Emer	gency a	ınd Abn	ormal F	Plant Evolutions - Tier 1/Group 1		
E/APE # / Name / Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	lmp.	Points
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1	1						AK1.10 Definition of QPTR	2.9	1
000005 Inoperable/Stuck Control Rod / 1			2				AK3.04 Tech. Spec. limits	4.1	1
000011 Large Break LOCA / 3		41		3			EK2.02/EA1.03 Interrelation between LOCA and Pumps/Securing RCPs	2.7/4.0	2
W/E04 LOCA Outside Containment / 3				42	4		EA1.1/EA2.1 Components and functions/Select Procedure	4.0/4.3	2
W/EO1 & E02 Rediagnosis & SI Termination / 3		71	<u> </u>				EK2.2 SI Termination sequence	3.9	1
000015/17 RCP Malfunctions / 4					5		AA2.08 Bearing High Temperature	3.5	1
BW/E09; CE/A13; W/E09&E10 Natural Circ. / 4					6		EA2.2 Adherence to appropriate procedures	3.8	1
000024 Emergency Boration / 1	7						AK1.03 Boron calculation	2.9	1
000026 Loss of Component Cooling Water / 8	-				26		AA2.02 Determine cause of possible CCW loss	3.6	1
000029 Anticipated Transient w/o Scram / 1									
000040 (BW/E05; CE/E05; W/E12) Steam Line Rupture - Excessive Heat Transfer / 4				28			AA1.04 Isolation of main steam lines	4.3	1
CE/A11; W/E08 RCS Overcooling - PTS / 4									
*000051 Loss of Condenser Vacuum / 4			29				AK3.01 Steam dump vacuum	3.1	1
000055 Station Blackout / 6			30		31		EK3.02/EA2.03 Actions for loss of power / Actions to restore power	4.6/4.7	2
*000057 Loss of Vital AC Elec. Inst. Bus / 6			32				AK3.01 Actions for loss of instrument bus	4.4	1
000059 Accidental Liquid RadWaste Rel. / 9		33					AK2.01 Radioactive gas monitors	2.8	1
*000062 Loss of Nuclear Service Water / 4			34				AK3.03 Actions for loss of service water	4.2	1
000067 Plant Fire On-site / 9	35						AK1.02 Fire fighting	3.9	1
000068 (BW/A06) Control Room Evac. / 8			36				AK3.18 Actions for control room evacuation	4.5	1
000069 (W/E14) Loss of CTMT Integrity / 5				37			AA1.01 Monitor isolation valves	3.7	1
000074 (W/E06&E07) Inad. Core Cooling / 4				38	8		EA1.06/EA2.02 Operate RCPs / AFW system	3.9/4.6	2
BW/E03 Inadequate Subcooling Margin / 4									
*000076 High Reactor Coolant Activity / 9		<u> </u>				9	2.1.12 Ability to apply Tech. Specs.	4.0	1
BW/A02&A03 Loss of NNI-X/Y / 7									
K/A Category Totals:	3	3	6	5	6	1	Group Point Total:		24

ES-401					PWR S	SRO Ex	amination Outline	Form ES-40	01-3
			Emerge	ncy an	d Abnor	mal Pla	ant Evolutions - Tier 1/Group 2		
E/APE # / Name / Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	lmp.	Points
000007 (BW/E02&E10 CE/E02) Reactor Trip - Stabilization - Recovery / 1									
BW/A01 Plant Runback / 1									
BW/A04 Turbine Trip / 4									
000008 Pressurizer Vapor Space Accident / 3			39				AK3.02 PORV/Code Safety Exit Temp	4.1	1
000009 Small Break LOCA / 3				40			EA1.01 Monitor RCS press. / temp.	4.3	1
BW/E08; W/E03 LOCA Cooldown - Depress. / 4									
W/E11 Loss of Emergency Coolant Recirc. / 4					43		EA2.2 Adherence to procedures	4.2	1
000022 Loss of Reactor Coolant Makeup / 2				44			AA1.08 VCT level	3.3	1
000025 Loss of RHR System / 4	45						AK1.01 Loss of RHR during all modes	4.3	1
000027 Pressurizer Pressure Control System Malfunction / 3	27						AK1.02 Expansion of liquid	3.1	1
000032 Loss of Source Range NI / 7		46					AK2.01 Switch positions	3.1	1
000033 Loss of Intermediate Range NI / 7	47					1	AK1.01 Effects of voltage changes	3,0	1
000037 Steam Generator Tube Leak / 3						48	2.2.25 Tech, Spec. reason	3.7	1
000038 Steam Generator Tube Rupture / 3			49				EK3.06 Bases for actions in EOP's	4.5	1
000054 (CE/E06) Loss of Main Feedwater / 4	10						AK1.01 Main feed line break	4.3	1
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	50			11			EK1.2/EA1.2 Loss of Heat Sink EOP's / Operating characteristics	4.5/4.0	2
000058 Loss of DC Power / 6									
000060 Accidental Gaseous Radwaste Rel. / 9									
000061 ARM System Alarms / 7			51				AK3.02 Guidance in alarm response	3.6	1
W/E16 High Containment Radiation / 9			52				EK3.1 Reasons for operating characteristics	3.1	1
000065 Loss of Instrument Air / 8			12				AK3.03 Effects of isolating equipment	3.4	1
CE/E09 Functional Recovery									
K/A Category Point Totals:	5	1	5	3	1	1	Group Point Total:		16

ES-401					PWR	SRO E	xamination Outline	Form ES-40	1-3
			Emer	gency a	nd Abn	ormal F	Plant Evolutions - Tier 1/Group 3		
E/APE # / Name / Safety Function	K1	K2	КЗ	A1	A2	G	K/A Topic(s)	lmp.	Points
000028 Pressurizer Level Malfunction / 2		13					AK2.03 Controllers and positioners	2.9	1
000036 (BW/A08) Fuel Handling Accident / 8									
000056 Loss of Off-site Power / 6					14		AA2.18 Operate and monitor PZR pressure	4.0	1
BW/E13&E14 EOP Rules and Enclosures									
BW/A05 Emergency Diesel Actuation / 6									
BW/A07 Flooding / 8									
CE/A16 Excess RCS Leakage / 2									
W/E13 Steam Generator Over-pressure / 4	53						EK1.2 Normal Abnormal & EOP's	3.3	1
W/E15 Containment Flooding / 5									<u> </u>
									1
									1
K/A Category Point Totals:	1	1	0	0	1	0	Group Point Total:		3

ES-401								minatio				F	orm ES-401	-3
System # / Name	K1	K2	КЗ	K4	K5	К6	A1	A2	А3	A4	G	K/A Topic(s)	Imp.	Points
001 Control Rod Drive	16						98					K1.03/A1.01 Physical connection to CRDM/Predict Tave response	3.6/4.2	2
003 Reactor Coolant Pump			54					55				K3.01/A2.02 RCS / Abnormal shutdown of RCP	4.0/3.9	2
004 Chemical and Volume Control						56		57				K6.07/A2.18 HX & Cond. / High VCT level	2.8/3.1	2
013 Engineered Safety Features Actuation				58								K4.13 MFW Isolation/reset	3.9	1
014 Rod Position Indication														
015 Nuclear Instrumentation	59											K1.01 RPS	4.2	1
017 In-core Temperature Monitor					60							K5.02 Saturation & subcooling	4.0	1
022 Containment Cooling							61				15	A1.01/2.2.22 Monitor CNMT Temp/Knowledge of LCO's	3.7/4.1	2
025 Ice Condenser														
026 Containment Spray														
056 Condensate								62				A2.04 Loss of condensate pumps	2.8	1
059 Main Feedwater				63						64		K4.08/A4.12 Mismatch S/F signal / Initiation of FWI	2.7/3.5	2
061 Auxiliary/Emergency Feedwater	65							66				K1.01/A2.04 S/G system / Pump failure	4.1/3.8	2
063 DC Electrical Distribution														
068 Liquid Radwaste	67											K1.07 Sources of input	2.9	1
*071 Waste Gas Disposal				68								K4.01 Decay tank pressure	3.0	1
*072 Area Radiation Monitoring				69								K4.01 Containment isolation	3.6	1
K/A Category Point Totals:	4	0	1	4	1	1	2	4	0	1	1	Group Point Total:		19

ES-401					F	WR SF	RO Exa	minatio	Outlin	e			Form ES-40	1-3
					Pl	ant Sys	stems -	Tier 2/	Group 2					
System # / Name	K1	K2	кз	K4	K5	К6	A1	A2	А3	A4	G	K/A Topic(s)	lmp.	Points
*002 Reactor Coolant				70								K4.02 Monitor vessel level	3.8	1
006 Emergency Core Cooling					74			17				K5.02/A2.02 Accum. Press, and Level/Loss of flowpath	2.9/4.3	2
010 Pressurizer Pressure Control		99								72		K2.01/A4.01 PZR Heater power supply/Monitor spray valves	3.4/3.5	2
011 Pressurizer Level Control	73											K1.03 PZR/PCS	4.0	1
012 Reactor Protection		100							18			K2.01/A3.06 RPS power supply/Trip Logic	<b>3.7</b> /3.7	2
016 Non-nuclear Instrumentation														
*027 Containment lodine Removal														
028 Hydrogen Recombiner and Purge Control						19						K6.01 Loss of H₂ recombiner	3.1	1
029 Containment Purge														
*033 Spent Fuel Pool Cooling								75				A2.02 Loss of SFPCS	3.0	1
034 Fuel Handling Equipment														
035 Steam Generator			76									K3,03 Effect on secondary systems	3.1	1
039 Main and Reheat Steam				77								K4.02 Use of Tave	3.2	1
055 Condenser Air Removal									78			A3.03 Automatic diversion to containment	2.7	1
062 AC Electrical Distribution								79				A2.05 Re-energize dead bus	3.3	1
064 Emergency Diesel Generator								20				A2.05 Malfunction loading EDG	3.2	1
073 Process Radiation Monitoring										80		A4.02 Rad. monitor control panel	3.7	1
075 Circulating Water											81	2.1.32 Explain system P&L's	3.8	1
079 Station Air														
086 Fire Protection														
103 Containment														
					-								_	-
K/A Category Point Totals:	1 1	2	1	2	1	1	0	4	2	2	1	Group Point Total:		17

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Contain # / No.	124	1 1/2	T 1/0	Ι			T	Г	Group 3	I	I		<del> </del>	T
System # / Name	K1	K2	К3	K4	K5	K6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Points
*005 Residual Heat Removal					ļ			82				A2.02 Pressure protection in CSD	3.7	1
007 Pressurizer Relief/Quench Tank					ļ		ļ	<u> </u>						
008 Component Cooling Water			83					<u> </u>				K3.03 Loss of CCWS on RCP	4.2	1
041 Steam Dump/Turbine Bypass Control														
045 Main Turbine Generator	84											K1.20 Protective system	3.6	1
*076 Service Water									85			A3.02 Emergency heat loads	3.7	1
078 Instrument Air														
K/A Category Point Totals:	1	0	1	0	0	0	0	1	1	0	0	Group Point Total:		4
Plant-Specific Priorities							•	<u> </u>			<del>'</del>		77-77-10-5-	!
System / Topic						Reco	mmeno	led Rep	laceme	nt for		Reason		Points
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Plant-Specific Priority Total: (limit 10)											T	I		

Facility: FENC		NIT 1 Date of Exam: 8/06/01	Exam Level:	SRO ———
Category	K/A #	Торіс	lmp.	Points
	2.1.1 <b>86</b>	Knowledge of conduct of operations requirements	3.8	1
	2.1.7 <b>21</b>	Ability to make operational judgment based on plant behavior and instruments	4.4	1
	2.1.12 <b>22</b>	Apply Technical Specifications to a system	4.0	1
Conduct of Operations	2.1.22 <b>87</b>	Ability to determine mode of operation	3.3	1
	2.1.23 88	Ability to perform plant procedures, all modes	4.0	1
	2.1.27 <b>89</b>	Knowledge of system purpose/function	2.9	1
	Total			6
	2.2.12 <b>90</b>	Knowledge of surveillance procedures	3.4	1
	2.2.13 <b>91</b>	Knowledge of tagging and clearance procedures	3.8	1
	2.2.22 <b>23</b>	Knowledge of LCO's and safety limits	4.1	1
Equipment Control	2.2.26 <b>92</b>	Knowledge of refueling administrative requirements	3.7	1
	2.2.33 <b>93</b>	Knowledge of control rod programming	2.9	1
	Total			5
	2.3.1 <b>94</b>	Knowledge of 10CFR20 related radiation control	3.0	1
Radiation Control	2.3.2 <b>95</b>	Knowledge of ALARA	2.9	1
	2.3.4 <b>24</b>	Knowledge of limits, cont. control and extensions	3.1	1
	Total			3
	2.4.6 <b>97</b>	Knowledge of symptom based EOP	4.0	1
Emergency Procedures/	2.4.7 <b>25</b>	EOP mitigation strategy	3.8	1
Plan	2.4.16 <b>96</b>	Knowledge of EOP hierarchy	4.0	1
	Total			3