

April 21, 1995

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Mr. J. P. O'Hanlon
Senior Vice President - Nuclear
Virginia Electric and Power Company
5000 Dominion Blvd.
Glen Allen, Virginia 23060

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: STATION
NUCLEAR SAFETY AND OPERATING COMMITTEE AND THE MANAGEMENT
SAFETY REVIEW COMMITTEE (TAC NOS. M88551 AND M88552)

Dear Mr. O'Hanlon:

The Commission has issued the enclosed Amendment No. 197 to Facility
Operating License No. DPR-32 and Amendment No. 197 to Facility Operating
License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2,
respectively. The amendments consist of changes to the Technical
Specifications (TS) in response to your application transmitted by letter
dated September 6, 1994, as supplemented March 7, 1995.

These amendments modify the TS to revise the review responsibilities of the
Station Nuclear Safety and Operating Committee and the Management Safety
Review Committee.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will
be included in the Commission's biweekly Federal Register notice.

Sincerely,
/s/

Bart C. Buckley, Senior Project Manager
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 197 to DPR-32
2. Amendment No. 197 to DPR-37
3. Safety Evaluation

cc w/enclosures:
See next page

DOCUMENT NAME: C:\AUTOS\WPDOCS\SURRY\SU88551.AMD

OFFICE	LA:PDII-1	PM:PDII-1	D:PDII-1	OGC	TQMB
NAME	EDunnington <i>ED</i>	BBuckley <i>BB</i>	DMatthews	<i>[Signature]</i>	S. BLACK <i>RG for</i>
DATE	04/13/95	04/17/95	04/20/95	04/16/95	
COPY	Yes/No	Yes/No	Yes/No	Yes/No	Yes/No

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[Handwritten marks]

Mr. J. P. O'Hanlon
Virginia Electric and Power Company

Surry Power Station
Units 1 and 2

cc:

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Robert B. Strobe, M.D., M.P.H.
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DATED: April 21, 1995

AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1
AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

Distribution

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

J. Zwolinski

D. Hagan, TWFN, 4/A/43

G. Hill (4), TWFN 5/C/3

C. Grimes, 11/F/23

ACRS (4)

OPA

OC/LFDCB

D. Verrelli, RII



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY
DOCKET NO. 50-280
SURRY POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 6, 1994, as supplemented March 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 197, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 21, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 197
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated September 6, 1994, as supplemented March 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 197, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



David B. Matthews, Director
Project Directorate II-1
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 21, 1995

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 6.1-7
TS 6.1-10
TS 6.1-11
TS 6.4-3
TS 6.4-4
TS 6.4-5

Insert Pages

TS 6.1-7
TS 6.1-10
TS 6.1-11
TS 6.4-3
TS 6.4-4
TS 6.4-5

f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all new normal, abnormal, and emergency operating procedures and all new maintenance procedures, b) all procedure changes that require a safety evaluation, and c) any other procedures or changes thereto as determined by the Station Manager which affect nuclear safety.
2. Review of all new test and experiment procedures that affect nuclear safety.
3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Station Manager.
5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and to the Management Safety Review Committee.
6. Review of all Reportable Events and special reports submitted to the NRC.
7. Review of facility operations to detect potential nuclear safety hazards.
8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Station Manager.

e. Meeting Frequency

The MSRC shall meet at least once per calendar quarter.

f. Quorum

The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

g. Review

The MSRC shall be responsible for the review of:

1. Safety evaluations as programmatically discussed in the Updated Final Safety Analysis Report for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to assess the effectiveness of the safety evaluation program and to verify that the reviewed actions did not constitute an unreviewed safety question.
2. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
3. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
4. Proposed changes to Technical Specifications or the Operating Licenses.

5. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
6. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
7. Events requiring written notification to the Commission.
8. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
9. A representative sample of reports and meeting minutes of the SNSOC.

h. Audits

Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

1. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions.
2. The performance, training and qualifications of the entire facility staff.
3. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety.

2. The requirements of 6.4.B.1 above, shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr, but less than 500 rads/hr at one meter from a radiation source or any surface through which radiation penetrates. In addition, locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under the administrative control of the Shift Supervisor on duty and/or the senior station individual assigned the responsibility for health physics and radiation protection.
 3. Written procedures shall be established, implemented, and maintained covering the activities referenced below:
 - a. Process Control Program implementation.
 - b. Offsite Dose Calculation Manual implementation.
- C. All procedures described in 6.4.A and 6.4.B shall be reviewed and approved by the Station Nuclear Safety and Operating Committee (SNSOC) prior to implementation. Subsequent procedure changes that require a safety evaluation shall also be reviewed and approved by SNSOC prior to implementation. All other changes shall be independently reviewed and approved as discussed in the Updated Final Safety Analysis Report.

- D. All procedures described in Specifications 6.4.A and 6.4.B shall be followed.
- E. Deleted |
- F. Deleted |
- G. In cases of emergency, operations personnel shall be authorized to depart from approved procedures where necessary to prevent injury to personnel or damage to the facility. Such changes shall be documented, reviewed and approved by the Station Nuclear Safety and Operating Committee.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 197 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

1.0 INTRODUCTION

By letter dated September 6, 1994, as supplemented March 7, 1995, Virginia Electric and Power Company requested changes to the Technical Specifications (TS) for Surry Units 1 & 2 with respect to the review responsibilities of their safety committees and the review of procedures. The licensee's submittal of March 7, 1995 provided clarifying information within the scope of the September 6, 1994 amendment application and did not change the staff's no significant hazard consideration determination.

Following is an evaluation of the proposed change request.

2.0 EVALUATION

a) Virginia Electric and Power Company proposes to revise the current requirement (Sections 6.1.C.1.f.1 and 6.4.C) that the Station Nuclear Safety and Operating Committee (SNSOC) review all changes to normal, abnormal, and emergency operating procedures, and all maintenance procedures and changes thereto. The proposed change would require the SNSOC to review only new normal, abnormal, and emergency operating procedures and all new maintenance procedures, and any procedure changes that require a safety evaluation. Likewise, only new tests and experiment procedures (Section 6.1.C.1.f.2) would be reviewed by the SNSOC rather than all proposed tests and experiments.

The licensee states that procedures undergo a screening process to determine whether a safety evaluation is required which determines whether a procedure goes to the SNSOC. We have reviewed the screening provisions described in VEPCO Station Administrative Procedure VPAP-3001 and find them acceptable. VEPCO also stated in their March 7, 1995 letter that the screening process would be specified in their Operational Quality Assurance Program Topical Report (OQAPTR) and that procedure changes that do not require a safety evaluation must be approved by cognizant management and a senior reactor operator.

Based on the screening process and procedure change approval by cognizant management and a senior reactor operator we find the proposed change to Section 6.1.C.2.f.1 with respect to the review of procedures acceptable.

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b) Virginia Electric and Power Company proposes to revise the current requirement (Sections 6.1.C.2.g.1 and 6.1.C.2.g.9) that the Management Safety Review Committee (MSRC) shall be responsible for the review of safety evaluations for changes to procedures, equipment or systems and tests or experiments completed under the provisions of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question. The proposed change would limit the MSRC review to samples of safety evaluations after 100% review to select the most safety significant as programmatically discussed in the Updated Final Safety Analysis Report (UFSAR). The licensee will also continue to assess the effectiveness of the safety evaluation program.

The licensee, in their letter dated March 7, 1995, stated that they will add to their OQAPTR a statement that the MSRC will screen 100% of all safety evaluations for safety significance; and additionally do in-depth reviews of a sample of safety evaluations based on safety significance and a sampling plan criteria.

Based on the commitment for the MSRC to perform a 100% screening of safety evaluations and the performance of in-depth review of a sample of safety evaluations based on safety significance, the staff finds the requested change with respect to the MSRC review of safety evaluations acceptable.

c) The licensee proposes to delete the special requirements (Sections 6.4.E and 6.4.F) for the review and approval of temporary changes to procedures. These procedures would then be processed as all other procedures and procedure changes.

The staff finds these changes acceptable as it does not diminish the level of review and approval of temporary procedures.

3.0 SUMMARY

The staff finds that the licensee's proposed changes meet the appropriate design criteria of Sections 13.4 and 13.5.1 of NUREG 0800, the Standard Review Plan, and are, therefore, acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Virginia State official was notified of the proposed issuance of the amendments. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Allenspach

Date: April 21, 1995