

PP&L-SUSQUEHANNA TRAINING CENTER

SIMULATOR SCENARIO

Scenario Title:

Small RPV Break with Loss of High Pressure Injection

Scenario Duration: 90 Minutes

Scenario Number:

2001 01 NRC Exam Scenario, 01NRC

Revision/Date:

Rev. 0, 06/20/01

Course:

Operator License Training

Operational Activities:

HPCI Isolation Valve Failure Feedwater Heater Tube Leak

Power Reduction

FWLC Failure

Generator Lockout

Primary System Break in the Drywell

RCIC Failure

Rapid Depressurization

Prepared By:	Rich Chin Instructor	6/20/01 Date
Reviewed By:	Nuclear Operations Training Supervisor	Date
Approved By:	Supervising Manager/Shift Supervisor	<u>€-28-01</u> Date

Page 2 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO SUMMARY

The crew assumes the shift with the plant operating at 100% power. During a routine surveillance HPCI Valve Testing Surveillance SO-152-004 the outboard HPCI steam isolation valve HV-152-F003 will bind in the almost closed position causing the valve breaker to trip on thermal overload. The Crew will respond IAW Alarm Response procedures and Technical Specifications. A Maintenance investigation will reveal that the valve stem is bent. The Crew will declare HPCI inoperable and enter a 14-day LCO (T.S. 3.5.1.D and 3.6.1.3 actions A.1 and A.2).

A tube leak in the 3B Feedwater Heater will cause the 3B heater Drain Valve and Dump Valve to open, and Heater High Level alarms. The crew will enter ON-147-002, Loss of Feedwater Heater String and ON-100-104, Power greater than 100%. Entry into the procedure will require a power reduction and eventual isolation of the heater string.

After the power reduction, the "C" Reactor Feedwater Pump (RFPT) Controller will fail as-is. This will cause a signal failure alarm and RFPT lockup. The operator must manually control the "X" RFPT.

After the RFPT Controller failure has been addressed there will be a spurious trip of the Main Generator Primary Lockout. The lockout will cause a turbine trip and scram. Operators will enter EO-100-102, RPV Control. The RPV pressure transient from the turbine/generator trip will break a RWCU line in the Drywell. The resulting High Drywell pressure will require entry into EO-100-103, Primary Containment Control and cause a Plant Aux Loadshed. The loadshed will result in a loss of Condensate, Feedwater, Service Water and Circulating Water pumps. This loss of high pressure injection will cause RPV level to lower.

RCIC will fail to automatically start, likewise the Manual Initiation Pushbutton will NOT initiate RCIC. RCIC may be started using a component by component start as outlined in OP-150-001. RCIC will subsequently trip on overspeed and require resetting, but will continue to trip on overspeed. RPV level will lower to –161" (TAF) requiring an RPV Rapid Depressurization to allow the use of low pressure injection systems. ADS auto logic is failed, requiring manual initiation to open the valves.

Page 4 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO OBJECTIVES

The SRO will:

- 1. Direct the PCO to Perform SO-152-004, "Quarterly HPCI Valve Exercising." (52.OP.009).
- 2. Ensure Plant Operates in Accordance With the Operating License and Technical Specifications. (00.TS.001).
- 3. Apply Technical Specification Requirements. (00.TS.003).
- 4. Implement Operations Standards for System and Equipment Operation, OP-AD-001. (00.AD.131).
- 5. Implement Alarm Response Procedures. (00.AR.005).
- 6. Respond to a loss of Feedwater heating in accordance with ON-147-002. (47.ON.005)
- 7. Respond to RPV Vessel Level Control Malfunctions IAW ON-145-001, REACTOR VESSEL LEVEL CONTROL SYSTEM MALFUNCTION (Section 3.10) (45.ON.006).
- 8. Respond to a leak inside the Drywell from Mode 1 in accordance EO-100-103, Primary Containment Control. (00.EO.010).
- 9. Implement the Emergency Plan, and assume the duties of the Emergency Director (00.EP.005).
- 10. Classify the emergency as conditions indicate (00.EP.001).
- 11. Ensure initiation of ESF equipment if automatic operation was not properly initiated.
- 12. Implement Scram (00.ON.018).
- 13. Implement RPV control (00.EO.026).
- 14. Implement Rapid Depressurization (00.EO.030).
- 15. Ensure that required actions per Technical Specifications/Technical Requirements are met when a LCO/TRO is entered (00.TS.003).

The ROs will:

- 1. Perform SO-152-004, "Quarterly HPCI Valve Exercising." (52.OP.009).
- 2. Comply with Technical Specification Requirements. (00.TS.003).
- 3. Implement Alarm Response Procedures. (00.AR.005).
- 4. Respond to a loss of feedwater heating in accordance with ON-147-002. (47.ON.005)
- 5. Respond to RPV Vessel Level Control Malfunctions IAW ON-145-001, REACTOR VESSEL LEVEL CONTROL SYSTEM MALFUNCTION (Section 3.10) (45.ON.006).
- 6. Respond to a leak inside the Drywell from Mode 1 in accordance EO-100-103, Primary Containment Control. (00.EO.010).
- 7. Perform Scram (00.ON.018).
- 8. Perform RPV Control (00.EO.026).
- 9. Perform Automatic/Manual Startup of RCIC System IAW OP-150-001. (50.OP.010)
- 10. Perform Rapid Depressurization (00.EO-030).
- 11. Perform Manual Operation of ADS IAW OP-183-001. (83.OP.001).

Page 6 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

Form 01NRC Rev. 0, (03/01) Page 6 of 23

CRITICAL TASKS

- ★ Performs Rapid Depressurization when RPV level cannot be maintained above -161 inches (TAF).
- ★ Restores and maintains RPV Level greater than -161 inches (TAF).

Page 8 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

Form 01NRC Rev. 0, (03/01) Page 8 of 23

SCENARIO REFERENCES

1. HPCI Surveillance and Valve Failure:

1.1	SO-152-004	QUARTERLY HPCI VALVE EXERCISING
1.2	Tech. Spec.	3.5.1, 3.6.1.3

2. Tube Leak in 3B Heater:

2.1	OP-147-001	FEEDWATER HEATERS
2.2	ON-147-002	LOSS OF FEEDWATER HEATER STRING
2.3	ON-100-104	REACTOR POWER GREATER THAN 100 PERCENT

3. Power Reduction:

```
3.1 GO-100-012 POWER MANEUVERS
```

4. "C" Feedwater Controller Failure:

4.1	ON-145-001	RPV LEVEL CONTROL SYSTEM MALFUNCTION
4.2	AR-101-B16	RFPT CONTROL SIGNAL FAILURE

5. Generator Lockout / Turbine Trip / Scram / RPV Break:

5.1	ON-100-101	SCRAM
5.2	EO-100-102	RPV CONTROL
5.3	EO-100-103	PRIMARY CONTAINMENT CONTROL
5.4	OP-149-004	RHR IN CONTAINMENT SPRAY MODE

6. RCIC failure to start:

6.1 OP-150-001 RCIC OPERATION

7. Rapid Depressurization:

7.1 EO-100-112 RAPID DEPRESSURIZATION

Page 10 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO SPECIAL INSTRUCTIONS

- 1. Initialize the Simulator to IC-182. Both Units at 100 percent power.
- 2. Load the following malfunctions and overrides OR use: restorepref YPP.NRC2001

	IMF RL01:E511K3 IMF RC150002 IMF AD183001 IMF MV09:HV155F003 5 0 100 IOR ZDIHSB211S30AA AsIs IOR ZDIHSB211S31AA AsIs IOR ZDIHSB211S30BA AsIs IOR ZDIHSB211S31BA Asis	
PB 1	MRF DC188113 OPEN	OPEN FEEDER BREAKER TO HPCI F003
PB 2	MRF DC188113 CLOSE	CLOSE FEEDER BREAKER TO HPCI F003
PB 3	MRF DB106236 OPEN	OPEN FEEDER BREAKER TO HPCI F002
PB 4	IMF HX02:1E103B 8 0 0	8% Tube Leaks on 3B Heater
PB 5	IMF FW145004C	RFPT Control signal Failure
PB 6	DMF FW145004C	DELETE RFPT Control signal Failure
PB 7	IMF EG198004	Spurious Trip of the Main Generator Primary Lockout 86GA
PB 8	IMF RR164010 10 0 0	Rx Vessel Bottom Head Drain Line Leak / Rupture Inside
PB 9	MMF RR164010 20 2:00	Containment.
PB 10	MMF RR164010 30 3:00	

- PB 11 IMF PM03:1P102A Condensate Pump 1A Overcurrent trip
 PB 12 IMF PM03:1P102B Condensate Pump 1B Overcurrent trip
 PB 13 IMF PM03:1P102C Condensate Pump 1C Overcurrent trip
- PB 14 IMF PM03:1P102D Condensate Pump 1D Overcurrent trip
- 4. Place the Simulator in RUN.
- 5. Prepare a copy of SO-152-004 with a Surveillance Authorization coversheet
- 6. Prepare a turnover package with turnover sheets.
 - Both Units at 100 percent power.
 - Perform SO-152-004, Quarterly HPCI Valve Exercising at beginning of shift.
 - All systems OPERABLE.

Form 01NRC Rev. 0, (03/01) Page 11 of 23

Page 12 Scenario 01NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO EVENT DESCRIPTION FORM

Initial Conditions: Both Units are at 100 Percent.

EVENT	TIME	DESCRIPTION		
1	5	Perform SO-152-004, "Quarterly HPCI Valve Exercising".		
2	10	Respond to the outboard HPCI steam isolation valve HV-152-F003 binding		
3	20	Respond to a tube leak in the 3B Feedwater Heater		
4	30	Reactor Power Reduction		
5	35 .	"C" Reactor Feedwater Pump Controller Failure		
6	45	Generator Lockout / Turbine Trip / Scram / RPV Break		
7	50	RCIC Failure to Auto Start and Subsequent Overspeed Trip		
8	60	Rapid Depressurization with RPV Level below –161 inches and recover RPV water level with Low Pressure systems		
	·			

SCENARIO EVENT FORM

Event No:

1 and 2

Brief Description: HPCI Valve Exercising Surv. and failure of HPCI-F003

POSITION	TIME	STUDENT ACTIVITIES
ВОР	5	Perform SO-152-004, QUARTERLY HPCI VALVE EXERCISING
·		Recognizes/reports AR-159-B03, HPCI DIV 2 MOV OL OR PWR LOSS BIS annunciator.
		Recognizes/reports HPCI F003 Valve failed to fully close.
		Dispatches NPO to check Breaker.
SRO		Dispatches Electrical/Mechanical Maintenance to investigate HPCI F003 Motor Operator and Breaker.
		Declares HPCI inoperable; consults Technical Specifications, and declares 14-Day LCO IAW TS 3.5.1 ECCS action D and 3.6.1.3 PCIV action A.1 and A.2
		Enters TRO 3.8.2 MOV Thermal Overload Protection when HPCI O/L are bypassed.
		Directs closing/deactivating HPCI F002 valve.

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

1 and 2

Brief Description: HPCI Valve Exercising Surv. and failure of HPCI-F003

INSTRUCTOR ACTIVITY:

Malfunction to bind HPCI F003 is pre-inserted with the preference file.

If/when directed, open breakers for the HPCI Inboard and Outboard Steam Supply valves as necessary by depressing:

PB 1

MRF DC188113 OPEN

OPEN FEEDER BREAKER TO HPCI F003

PB 2

MRF DC188113 CLOSE

CLOSE FEEDER BREAKER TO HPCI F003

PB 3

MRF DB106236 OPEN

OPEN FEEDER BREAKER TO HPCI F002

ROLE PLAY:

- 1. As NPO dispatched to check the breaker for HPCI F003, wait approximately two minutes and report that the Breaker is closed and the thermals were tripped. If asked to visually inspect HPCI F003, report that the valve stem appears to be distorted.
- 2. As Electrical/Mechanical Maintenance dispatched to investigate the HPCI F003 failure, wait approximately three minutes and report that the valve motor operator is okay; looks like stem is binding. We will give you more information as soon as it becomes available.

SCENARIO EVENT FORM

Event No:	3
Brief Description:	Respond to a tube leak in the 3B Feedwater Heater

POSITION	TIME	STUDENT ACTIVITIES
ВОР	20	Responds to Feedwater Heater Local Panel 1C102 alarms and indication on 1C668.
		Diagnoses tube failure and enters ON-147-002, LOSS OF FEEDWATER HEATER STRING
		Enters OP-147-001, FEEDWATER HEATERS Takes actions to remove Heater 3B from service.
		Monitor Main Steam and Offgas radiation levels.
RO		Determines Reactor power is rising above 100%
		Enters ON-100-104, REACTOR POWER GREATER THAN 100 PERCENT
		Restores Reactor power to ≤ 100%
SRO		Directs Reactor Power monitored, enters ON-100-104, REACTOR POWER GREATER THAN 100 PERCENT
		Directs implementation of ON-147-002.
		Directs a power reduction to ≤ 75% IAW GO-100-012 and CRC Instructions
		Notifies RX Engineering, Chemistry, Health Physics, Operations Management.
		Directs periodic checks of Power/Flow Map
		Direct monitoring of Main Steam and Offgas radiation levels.
		Initiate request for I&C/Maintenance investigation of FW Heater problem.
		Notify Plant Management of ongoing events.

NOTES:			

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

Brief Description: Respond to a tube leak in the 3B Feedwater Heater

INSTRUCTOR ACTIVITY:

When HPCI event has been completed, insert tube leak in the 3B FW HEATER by depressing:

PB 4

IMF HX02:1E103B 8 0 0 Feedwater Heater Tube Leaks on Selected Heater.

NOTE: This loss of FW heating will not result in fuel damage.

ROLE PLAY:

- 1. As the NPO dispatched to investigate the local feedwater panel alarms using local panel graphics for Panel 1C102.
- 2. As Reactor Engineering when called, inform Shift you will run a new Core Performance Log and will be in the Control Room shortly.
- As necessary, when contacted as Plant Management, GCC, Chemistry and Health Physics. 3.
- 4. As necessary, when contacted as I&C/Electrical Maintenance.
- 5. When directed to isolate Instrument Air to HV10244B by closing valve 1251467, wait a couple of minutes and report that it has been closed; likewise for PCV10244B.

SCENARIO EVENT FORM

POSITION	TIME	STUDENT ACTIVITIES
RO	30	Immediately reduces Reactor Power to ≤ 75 percent in accordance with ON-147-002
		Plots POWER/FLOW conditions on power-to-flow map.
		Monitors Main Steam and Offgas activity levels.
		Monitors Reactor power and feedwater temperatures.
		Notifies GCC of plant status and load capability.
ВОР		Maintains EHC Load Set within 100Mwe of actual load.
		Maintains Auto and Manual Voltage regulators balanced.
SRO		Directs power lowered to ≤75% IAW GO-100-012 POWER MANEUVERS and CRC Instructions.
		Monitors COLR limits and looks for indication of core instability.
		Makes appropriate management notifications
	· ·	
NOTES:		

Form 01NRC Rev. 0, (03/01) Page 18 of 23

INSTRUCTOR ACTIVITIES, ROLE P	LAY,
AND INSTRUCTOR'S PERSONAL N	OTES

Event No:

Brief Description: Power Reduction

INSTRUCTOR ACTIVITY:

None

ROLE PLAY:

As necessary

INSTRUCTOR ACTIVITIES, ROLE PLAY,

SCENARIO EVENT FORM

Event No:	5
Brief Description:	"C" Reactor Feedwater Pump Controller Failure

POSITION	TIME	STUDENT ACTIVITIES
RO	40	Recognize and respond to AR-101-B16 RFPT CONTROL SIGNAL FAILURE
		Report failure to SRO
		Implement ON-145-001, Section 3.8.
		 Place LIC-C32-1R601C to MANUAL. Adjust level to 35". Reset Control Signal Failure. Null FW Level Control/Demand Signal.
		Place LIC-C32-1R601C to AUTO.
ВОР		Maintain plant stability
SRO		Directs implementation of ON-145-001, Section 3.8
		Notifies I&C to support troubleshooting.

NOTES:			

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

5

Brief Description:

"C" Reactor Feedwater Pump Controller Failure

INSTRUCTOR ACTIVITY:

When Feedwater Heater evolution has been adequately addressed, insert "C" Reactor Feedwater Pump Controller Failure by depressing:

PB 5

IMF FW145004C

RFPT Control signal Failure

Following fuse replacement by I&C, remove RFPT Control Signal failure by depressing:

PB 6

DMF FW145004C

Delete malfunction RFPT Control signal Failure

ROLE PLAY:

As I&C:

When notified of problem take 2 minutes then report that a fuse blew in the local controller. Wait until the operator actions have been addressed, then report that it has been replaced. Notify the SRO that the controller may be returned to service.

SCENARIO EVENT FORM

Event	

6

Brief Description: Generator Lockout / Turbine Trip / Scram / RPV Break

POSITION	TIME	STUDENT ACTIVITIES
RO		Verify reactor scrammed – All Rods IN, power lowering
		Place Feedwater in Startup Level Control following the Scram.
		Verify proper operation of Turbine Bypass Valves.
ВОР		Perform EO-100-102, RPV Control actions as directed
		Report status of Isolations, ECCS Initiations, DG starts
		Recognize/Report failure of RCIC to automatically initiate.
		When directed, Perform Suppression Chamber Spray IAW OP-149-005
		When Directed, initiate Drywell Sprays, limiting flow to between 1000 and 2800 for the first 30 seconds.
		Stop Drywell/Suppression Chamber Sprays before Drywell/Suppression Chamber pressure drops to 0 psig.
		Place RHRSW in Service.
SRO		Implements EO-100-102, RPV Control
		Implements EO-100-103, Primary Containment Control.
		Plot points on the RPV Saturation Curve to ensure RPV Vessel level instruments are operable.

NOTES:			

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

6

Brief Description: Generator Lockout / Turbine Trip / Scram / RPV Break

INSTRUCTOR ACTIVITY:

When RFPT "C" is returned to Auto control, initiate Generator Lockout / Turbine Trip / Scram / RPV Break by depressing:

PB 7

IMF EG198004

Spurious Trip of Main Generator Primary Lockout 86GA

PB8

IMF RR164010 10 4:00

RPV Bottom Head Drain Line Leak/Rupture (10%) Inside

Containment ramped over 4 minutes

Increase RWCU leak as necessary to ensure RPV level reaches TAF and Drywell Sprays will be required by depressing:

PB 9

MMF RR164010 20 2:00

RPV Bottom Head Drain Line Leak/Rupture (20%) Inside

Containment ramped over 2 minutes

PB 10

MMF RR164010 30 3:00

RPV Bottom Head Drain Line Leak/Rupture (30%) Inside

Containment ramped over 3 minutes

ROLE PLAY:

As necessary

SCENARIO EVENT FORM

Event No:	7
Brief Description:	Failure of RCIC to Initiate

POSITION	TIME	STUDENT ACTIVITIES
ВОР		Recognize/report failure of RCIC to automatically initiate.
		Attempt Manual Pushbutton Initiation – Determine component by component startup required. Enter Section OP-150-001, sect. 3.4.
		Place RCIC Flow Controller in MANUAL.
		Start Barometric Condenser Vacuum Pump.
		Open Cooling Water Valve, HV-150-F046.
		Open Steam to RCIC, HV-150-F045.
		Observe RCIC speed rises.
		Diagnose RCIC trips on overspeed.
		Report Failure of RCIC to SRO
· , · · · · · · · · · · · · · · · · · ·		
	_	
		·
	······································	

NOTES:		

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

Brief Description: Failure of RCIC to Initiate

INSTRUCTOR ACTIVITY:

Ensure the following malfunctions are active:

IMF RL01:E511K3

RCIC Failure to Auto Start

IMF RC150007

RCIC Overspeed

ROLE PLAY:

Role play as necessary

SCENARIO EVENT FORM

Eve	nt	No:	

Event No: 6 (Continued)

Brief Description: RPV Water Level requires Rapid Depressurization

POSITION	TIME	STUDENT ACTIVITIES
BOP		Monitor RPV Level decrease to TAF using Fuel Zone Level Instrumentation
		Recognize/report RPV Water Level cannot be maintained above –161 inches.
SRO		Ensure all available injection sources are started
		Implements EO-100-112 RAPID DEPRESSURIZATION when RPV Level reaches –161 inches,
		Enter EDP to flood RPV due to loss of RPV level indicati
ВОР	See Note #1	Manually initiate ADS with handswitches.
	7,010 #1	Direct NPO to Upper or Lower Relay Room to keep ADS Valves open
RO/BOP		Control injection to maintain RPV level +13 to +54 inches
· · · · · · · · · · · · · · · · · · ·		

NOTES:	Note #1: With ADS logic failed, the crew will have to manually initiate ADS or keylock switches from the relay room.

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES

Event No:

6 Continued

Brief Description: RPV Water Level requires Rapid Depressurization

INSTRUCTOR ACTIVITY:

To Ensure Operators are unable to use Condensate Pumps following Plant Auxiliary Loadshed, trip them on overcurrent if started by depressing:

PB 11 IMF PM03:1P102A

1A Condensate Pump overcurrent trip

PB 12 IMF PM03:1P102B

1B Condensate Pump overcurrent trip

PB 13 IMF PM03:1P102C

1C Condensate Pump overcurrent trip

PB 14 IMF PM03:1P102D

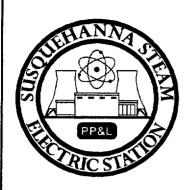
1D Condensate Pump overcurrent trip

ROLE PLAY:

As Necessary

TERMINATION CRITERIA:

Reactor depressurized and RPV level restoration in progress to maintain between +13" and +54".



PP&L-SUSQUEHANNA TRAINING CENTER

SIMULATOR SCENARIO

Scenario Title:

4 KV Bus Failure and ATWS - Level Power Control

Scenario Duration: 90 Minutes

Scenario Number:

2001 02 NRC Exam Scenario, 02NRC

Revision/Date:

Rev. 0, 06/20/01

Course:

Operator License Training

Operational Activities:

Remove T-10 From service Loss of 4 KV Bus 1D **Spurious HPCI Initiation**

"A" APRM Flow Unit Failure Loss of Instrument Air Hydraulic ATWS - Level Power control

Power Reduction

Prepared By:	Rich Chin Instructor	6/20/01 Date
Reviewed By:	Nuclear Operations Training Supervisor	4/Z4/0(Date
Approved By:	Supervising Manager/Shift Supervisor	€ 28-01 Date

Page 2 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

Form STCP-QA-612A Rev. 0, (03/01) Page 2 of 19

SCENARIO SUMMARY

The Plant is in MODE 1 at 100% power. The crew must transfer loads from Transformer T-10 to SU Bus 20 per OP-003-001 to remove Startup Transformer T-10 from service for a maintenance inspection that's expected to last less than 8 hours.

After the transfer is completed the normal supply breaker to 4KV bus 1A204 (ESS Bus 1D) will trip and the bus will transfer to its alternate source. This momentary loss of power will cause a loss of equipment including RBCCW to the Drywell and B CRD Pump. Entry into ON-155-007, Loss of CRD Flow, will be required to start a CRD Pump.

When the lineups have been restored HPCI will spuriously initiate and CANNOT be controlled using the flow controller. HPCI must be tripped and isolated. Reactor power must be lowered.

After power has been stabilized the A APRM Flow unit will fail downscale causing a half scram requiring bypassing the flow unit and resetting the half scram. A loss of instrument air will require tripping the plant. When the mode switch is placed in Shutdown the rods do not fully insert due to a hydraulic ATWS. As actions progress, the loss of Instrument Air will cause the MSIVs to close and power level will require entry into Level/Power Control.

After the crew controls RPV water level and Reactor power the crew must start inserting control rods using the RMCS and/or venting CRD over-piston volume

After the Crew has demonstrated their ability to insert the control rods and established RPV water level control the Simulator activity is over.

Page 4 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO OBJECTIVES

The SRO candidate will:

- 1. Ensure Plant Operates IAW the Operating License and Technical Specifications (00.TS.001).
- 2. Ensure that Required Actions per Technical Specifications/Technical Requirements are met when a LCO/TRO is entered (00.TS.003).
- 3. Inform other shift members and plant management of changes in plant status, potential plant problems or limitations. (00.AD.131).
- 4. Implement ON-104-204 LOSS OF 4KV ESS BUS 1D (1A204)
- 5. Direct entry into ON-155-007, Loss of CRD Flow, to restore CRD Flow
- 6. Implement ON-100-004 REACTOR POWER GREATER THAN 100%
- 7. Implement ON-164-001 RECIRC DRIVE FLOW INSTRUMENTATION FAILURE (64.ON.009).
- 8. implement ON-100-101 SCRAM (00.ON.018).
- 9. implement EO-100-102 RPV CONTROL (00.EO.026).
- 10. Implement EO-100-113 LEVEL / POWER CONTROL (00.EO.031).
- 11. Implement EO-100-103 PRIMARY CONTAINMENT CONTROL (00.EO.027).
- 12. Implement GO-100-012 POWER MANEUVERS (00.GO.010).
- 13. Classify the Emergency IAW EP-PS-100 EMERGENCY DIRECTOR (00.EP.001).

The RO candidate will:

- 1. Perform ON-155-007, LOSS OF CRD FLOW, to restart a CRD Pump.
- 2. Implement ON-164-001 RECIRC DRIVE FLOW INSTRUMENTATION FAILURE (64.ON.009
- 3. Perform operation of RHR in Suppression Pool Cooling with a LPCI signal present (49,OP,012).
- 4. Perform maximizing CRD flow (55.OP.001).
- 5. Perform ON-100-004 REACTOR POWER GREATER THAN 100%
- 6. Perform initiation of Standby Liquid Control System (53.OP.003).
- 7. Perform inserting manual scram with CRD in service (55.OP.006).
- 8. Perform inhibiting ADS (83,OP,005).
- 9. Implement Scram (00.ON.018).
- 10. Implement RPV Control (00.EO.026).
- 11. Implement Level / Power Control (00.EO.031).
- 12. Implement Primary Containment Control (00.EO.027).
- 13. Implement GO-100-012 Power Maneuvers (00.GO.010).
- 14. Perform overriding HPCI System (52.OP.009).
- 15. Perform a power change with Recirc or Rods (00.GO.012).
- 16. Perform overriding Core Spray System (51.0P.004).
- 17. Perform overriding RHR System (49.OP.011).
- 18. Perform manual bypass of RWM (31.OP.001).
- 18. Perform RHRSW System startup Unit 1 RHRSW Pump to Unit 1 Heat Exchanger (16.OP.002).

Page 6 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

CRITICAL TASKS

- * Insert Control Rods IAW EO-100-113, Sheet 2, CONTROL ROD INSERTION.
- Initiate the Standby Liquid Control (SLC) system IAW EO-100-113, Step LQ/Q-3, LEVEL/POWER CONTROL.
- ★ Throttle and prevent injection to maintain RPV water level between −60 to −161 inches.

Page 8 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

SCENARIO REFERENCES

1.	Remove Startup Transformer T-10 from Service			
	1.1	OP-003-001	13.8KV Common Electrical Equipment	
2.	Supply Breaker trip to Bus 1D			
	2.1 2.2	ON-104-204 ON-155-007	Loss of 4KV ESS Bus 1D (1A204) Loss of CRD Flow	
3.	Inadvertent HPCI Initiation			
	3.1 3.2	OP-152-001 T.S. 3.5.1	HPCI System operation Emergency Core Cooling Systems	
4.	Power Reduction:			
	4.1	GO-100-012	Power Maneuvers	
5.	Recirc Flow Unit Failure			
	5.1 5.2 5.3	ON-164-001 TS 3.3.1.1 TR 3.1.3	Recirc Drive Flow Instrument Failure RPS Instrumentation Control Rod Block Instrumentation	
6.	Loss of Instrument Air			
	6.1 6.2 6.3	ON-118-001 ON-100-101 EO-100-102	Loss of Instrument air Reactor Scram RPV Control	
7.	Failure to Scram - ATWS			
	7.1 7.2 7.3 7.4 7.5 7.6 7.7	EO-100-113 EO-100-103 OP-150-001 OP-155-001 OP-149-001 OP-151-001 OP-145-001	Level Power Control Primary Containment Control RCIC System Operation CRD System Operation RHR System Operation Core Spray System Operation Feedwater System Operation	

RHRSW System Operation

RHR SPC Operation

7.8

7.9

OP-116-001

OP-149-005

Page 10 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

Form STCP-QA-612A Rev. 0, (03/01) Page 10 of 19

SCENARIO SPECIAL INSTRUCTIONS

1. Initialize the Simulator to IC-182. Both Units at 100 percent power.

Ensure the "B" CRD Pump is operating

2. Load the following malfunctions and overrides:

	IMF RD155017 bat RPB.HYDATWS_1 bat RPB.STKRDS IMF CN02:FCE411R600 89 0 100 IOR ZDIFCE411R600A OPEN	HYD ATS BROWN FERRY 58 RODS STUCK IN CORE 16 RODS STUCK IN CORE HPCI FLOW CONTROLLER FAILED AS IS HPCI FLOW CONTROLLER FAILED OPEN
PB 1	IMF BR03:1a20409	4KV ESS Bus 1D (1A204) Normal Supply Breaker Trips
PB 2	IMF HP152004	Inadvertent HPCI Initiation
PB 3	IMF NM178012	"A" Flow Unit Failure Downscale
PB 4	MRF NM178006 ZERO	Place 'A' Flow Unit mode switch to ZERO
PB 5	IMF IA118002 2% to 20%	Instrument Air Break on the Common Header
PB 6	bat RPB.ES158002	(over 20 minutes) Bypasses RPS Scrams
PB 7	bat RPB.DISABLARI	Disables ARI
PB 8	DMF RD155017	DELETES BROWN FERRY ATWS
PB 9	bat RPB.HYATWSCLR	DELETES 58 RODS STUCK
PB 10	bat EOP.ACT17	DELETES 16 RODS STUCK

- 3. Place the Simulator in RUN.
- 4. Prepare a turnover package with turnover sheets
 - Both Units at 100 percent power.
 - Perform OP-003-001 to remove Startup Transformer T-10 from service for a maintenance inspection that's expected to last less than 8 hours.
 - All systems OPERABLE.

Page 12 Scenario 02NRC Rev. 0, 06/20/01

THIS PAGE IS INTENTIONALLY LEFT BLANK

Form STCP-QA-612A Rev. 0, (03/01) Page 12 of 19

SCENARIO EVENT DESCRIPTION FORM

Initial Conditions: Unit 1 is at 100% Power. No LCOs are in Effect.

EVENT	TIME	DESCRIPTION
1	3 Min	Remove Startup Transformer T-10 from service.
2	10 M in	Breaker supplying 4KV ESS Bus Trips OPEN
3	25 Min	Inadvertent HPCI Initiation
4	35 Min	Reactor Power Reduction
5	55 Min	"C" APRM Flow Unit Fails Downscale
6	75 min	Loss of Instrument Air
7	80 min	ATWS

	ř	T · · · · · · · · · · · · · · · · · · ·
POSITION	TIME	STUDENT ACTIVITIES
BOP	3 min	Enter OP-003-001, Section 3.7
		Notify Power Control Center
		Verify loads on bus will not exceed 2500 Amps
		Insert key in TIE BKR SYNC SEL, HS-00018 keyswitch to position 1.
		Close TIE BKR BKR OA10502
		Verify SU XFMR 10 TO BUS 10 BKR OA10301 automatically opens.
		Return TIE BKR SYNC SEL, HS-00018 keyswitch to OFF and remove the key.
		Align all control switch flags to the actual breaker position.
		Open SU XFMR 10 MOAB 1R105 by placing switch to OPEN. Notify Revers Control Control
		Notify Power Control Center
		·
	· · · · · · · · · · · · · · · · · · ·	
	··	1
NOTES:		
	····	

INSTRUCTOR ACTIVITIES, ROLE PLAY, AND INSTRUCTOR'S PERSONAL NOTES				
Event No:	1			
Brief Description:	Remove Startup Transformer T-10 from service.			

INSTRUCTOR ACTIVITY:

None Required

ROLE PLAY:

1. As Power Control Center Operator acknowledge the removal of the T-10 Transformer for maintenance expected to last less than 8 hours.

Event No:	2
Brief Description:	Breaker supplying 4KV ESS Bus Trips OPEN

POSITION	TIME	STUDENT ACTIVITIES
BOP	10 min	Diagnose trip of the normal supply breaker to ESS Bus 1D
		 Enter ON-104-204, section 3.1, Attachment A Verify ESS Bus 1D energized Verify Instrument Gas Compressor operating normally. Start CRD Pump in with ON-155-007, Loss of CRD Flow. Restore RBCW to the Recirc Pump motor coolers (within 7 min) Dispatch NPO to locally determine breaker condition.
SRO		Refers to T.S. (No LCO) Refers to ON-104-204 LOSS OF 4KV ESS BUS 1D (1A204) Contacts Electrical Maintenance/EWAC to investigate breaker.

NOTES:	Corrective maintenance may lead to LCO not being met, therefore, Required Action A.1 would be entered.

Event No:

Brief Description: Breaker supplying 4KV ESS Bus Trips OPEN

INSTRUCTOR ACTIVITY:

When directed by the chief examiner initiate the malfunction by depressing:

PB₁

IMF BR03:1A20409

4KV ESS Bus 1D (1A204) Normal Supply Breaker Trips

ROLE PLAY:

- 1. If dispatched as NPO to locally check position of the breaker, wait ≈2 minutes and report the breaker is open and there are no tripped flags or indications of a problem.
- 2. If dispatched as maintenance to investigate the breaker, wait ≈ 5 minutes and report that the breaker must eventually be removed from the Bus. State you need to get a work plan approved to perform the necessary repairs. No time estimate is available at this time.
- 3. If/When dispatched to the instrument Air Compressor panel 1C140, reset AR-B02, "Dryer Control Power Failure" was in, but is all clear at this time.
- 4. If/When dispatched to the D D/G, report everything looks normal, and the air receivers have just shut down.

Event No:

3

Brief Description: Inadvertent HPCI Initiation and Injection

POSITION	TIME	STUDENT ACTIVITIES
BOP/RO	25 min	Recognizes and reports HPCI has initiated
SRO		Determines HPCI mis-operation in Auto
		Directs BOP to override HPCI injection
		Directs RO to monitor reactor power
		Directs isolation of HPCI when HPCI cannot be overridden
		Directs RO to monitor MSL and Off-gas rad levels
···		Call I&C/EWAC to investigate HPCl problem
		Refers to T.S. 3.5.1 and Declares HPCI inoperable
		Enter Required Action D.1, verify RCIC is operable immediately and D.2, restore HPCI Operable in 14 days
BOP		Determines HPCI initiation invalid by observing RPV Level and Drywell pressure indication
		Refers to OP-152-001 Section 3.9 to override HPC!
		Notifies SRO HPCI is not responding
		Takes action to override HPCI injection
		Depresses isolation pushbutton or closes Isolation valves and verifies HPCI F003 shuts, turbine trips and injection stops
RO		Monitors APRM and thermal power change
		Monitors MSL and Off-gas Rad Monitors
		Directs NPO to reset 1C605 Rad Monitors

NOTES:			

Eve	nt	N	۸.	
		IW	u.	

Brief Description: Inadvertent HPCI Initiation and Injection

INSTRUCTOR ACTIVITY:

1. When actions are complete for ESS Bus 1D, insert the following to cause HPCI initiation:

PB₂

IMF HP152004

Inadvertent HPCI Initiation

2. If requested to reset ARMs at panel 1C605, insert the following:

MRF RM179004 RESET

RESET ARMs AT 1C605

ROLE PLAY:

As I & C dispatched to investigate HPCI system, wait ≈ 5 minutes and report an intermittent ground exists in the logic. Additional investigation is required, and no time estimate for restoration is possible at this time. The failure of the controller is a separate issue and is not related to the initiation logic as far as we can tell at this point.

Event No:	4
Brief Description:	Power Reduction

POSITION	TIME	STUDENT ACTIVITIES
RO	35 min	Immediately reduces reactor power to within the licensed limit IAW ON-100-104, or GO-100-012 and CRC Instructions if power reduction > 5%.
		Plot POWER/FLOW change on power-to-flow map in accordance with NDAP-QA-0338-10.
		Monitors main steam and Offgas activity levels.
		Monitors reactor power and Feedwater temperatures.
		Notifies GCC of plant status and load capability.
SRO		Directs performance of ON-100-004 REACTOR POWER GREATER THAN 100%
		Monitors COLR limits and looks for indication of core instability.
4.00		Notifies Reactor Engineering, Operations Management of events
ВОР		Maintains Load Set within 100Mwe of actual generator load
		Maintains Auto and Manual Voltage regulators balanced
		Dispatch NPO to Feedwater Heater Panel for alarms resulting from the power change
		RE calls to request power reduction of 5%.
····		

NOTES:	

Event No:

Brief Description: Power Reduction

INSTRUCTOR ACTIVITY:

None

ROLE PLAY:

As necessary

Event	No:	5

Brief Description: Recirc Flow Unit Failure.

POSITION	TIME	STUDENT ACTIVITIES
RO	55 min	Respond to AR-103-C04, C05, and AR-104-H05
		Recognize/report Rod Block and half scram.
		Recognize/report indication of a failed Recirc Flow Unit.
		Determines Flow Unit 'C' has failed.
SRO		Dispatches NPO to Relay Room.
		Implements ON-164-001 for loss of Recirc Drive Flow instrument failure.
		Directs bypass of failed flow unit and resetting half scram
		Reviews Technical Specifications 3.3.1.1, determines LCO is not met. Enters Action A.1, place channel in trip or place the trip system in trip within 12 hour
		Declares 1 of 2 required Flow Biased Simulated Thermal Power - High trip functions inoperable.
		Refers to Technical Requirement 3.1.3, determines TRO is met.
		Contact I & C to investigate Recirc Flow Unit 'C' Failure.
RO		Places joystick for "A" Flow Unit to BYPASS.
		Directs NPO to place mode switch for "C" Flow Unit to ZERO position.
		Resets half scram

NOTES:			

INSTRUCTOR ACTIV	'ITIES, ROLE PLAY,
AND INSTRUCTOR'S	PERSONAL NOTES

Event No: 5

Brief Description: Recirc Flow Unit Failure.

INSTRUCTOR ACTIVITY:

1. When the Chief/Lead Examiner indicates the exam team has seen sufficient power change, insert a Recirc Flow Unit failure by depressing:

PB3

IMF NM178012A

"A" Flow Unit Failure Downscale

2. When directed to place 'C' Flow Unit mode switch to ZERO insert:

PB4

MRF NM178006 ZERO

"A" Flow Unit mode switch to ZERO

ROLE PLAY:

- 1. As NPO dispatched to the relay room: if asked for Flow Unit meter indications, report C indicates downscale.
- 2. As NPO dispatched to the Relay Room to place Mode Switch to ZERO: wait ~2 minutes and report the 'C' Flow Unit Mode Switch is in ZERO position.
- 3. As I&C dispatched to investigate flow unit failure: wait ~5 minutes and report a suspected circuit card failure, but no additional information is available; trouble shooting is continuing.

Event No:	6
Brief Description:	Loss of Instrument Air

POSITION	TIME	STUDENT ACTIVITIES
BOP/RO	75 min	Recognize/report Instrument air Pressure lowering.
		Dispatches NPO to Instrument Air compressors and receiver area.
	************	T
ВОР		Performs scram eminent actions: Shift Unit Auxiliary Busses to the Startup Transformer Start: Main Lube Oil Suction Pump
· · · · · · · · · · · · · · · · · · ·		Turning Gear Oil Pump
RO		Lowers Reactor power with Recirc flow IAW CRC Instructions 65 Mlbm/hr
		Initiates a Manual Scram before/when Instrument Air pressure reaches 65 psig
SRO		Implements ON-118-001 LOSS OF INSTRUMENT AIR.
		Recognizes inability to recover and directs scram imminent actions
· · · · · · · · · · · · · · · · · · ·		

NOTES:			

INSTRUCTOR	ACTIV	ITIES,	ROLE	PLAY,
AND INSTRUCT	TOR'S	PERS	ONAL	NOTES

Event No: 6
Brief Description: Loss of Instrument Air

INSTRUCTOR ACTIVITY:

1. When "A" Flow Unit has been bypassed and half scram reset, insert Instrument air break on the Common Header by depressing:

PB5

IMF IA118002 20 20:00

20% Instrument Air Break on the Common Header (over 20 minutes)

Note: Leak is slow enough to ensure a power reduction and Scram Imminent Actions are completed prior to reaching 65# on the header.

ROLE PLAY:

1. As NPO dispatched to the Instrument Air area report hearing an air leak but not being able to specifically see the exact location, report system responding normally but pressure slowly lowering.

Event No:	7	
Brief Description:	Hydraulic ATWS:	

POSITION	TIME	STUDENT ACTIVITIES
RO	80 min	Place Reactor MODE SWITCH to S/D, arms and depresses the manual scram
		pushbuttons; reports partial rod motion.
		Lowers RPV level <-60" controls RPV level in target band -60" to 110".
		Inserts control rods with RMCS
CREW		Recognizes that the RFPT will trip, and MSIVs will close and that with no Instrument ai
		the scrams CANNOT be reset (ES-158-002 should NOT be entered).
SRO		Enters EO-100-102; then exits EO-100-102 and enters EO-100-113.
		Directs inhibiting ADS.
		Directs lowering RPV level <-60" using Feedwater, HPCl and RCIC; directs RPV level to be controlled in target band -60" to -110".
		Directs manually initiating ARI
		Directs Injection of SLC
		Directs Maximizing CRD
ВОР		Manually initiates ARI.
		Maximize CRD. Initiate SLC.
		Inhibit ADS.
	 .	Stabilizes RPV pressure <1,087 psig using SRVs.
		When directed, maximizes Suppression Pool Cooling.
		When directed sends operator to vent Scram Air Header
	 	

NOTES:	

Event No:

Brief Description: Hydraulic ATWS:

IF directed by ES-158-002, reset RPS and scram rods by depressing

PB 6

Bat RPB.ES158002

BYPASS RPS SCRAMS

ROLE PLAY:

If directed report you are ready to disable ARI:

PB 7

Bat RPB.DISABLARI

Disables ARI

When scram has been reset and prior to manual Scram, allow rods to insert by depressing:

PB 8

DMF RD155017

DELETES BROWN FERRY ATWS

PB9

bat RPB.HYATWSCLR

DELETES 58 RODS STUCK

PB 10

bat EOP.ACT17

DELETES 16 RODS STUCK

ROLE PLAY:

If directed, report you are ready to BYPASS RPS.

After the Crew has demonstrated inserting control rods, has lowered power and is controlling RPV water level, AND WHEN DIRECTED BY THE CHIEF EXAMINER place the Simulator to FREEZE.

UNIT SUPERVISOR TURNOVER SHEET

	UNIT:	1	Date: August 15, 2001	
ز	SHIFT 1900 to 0700 Start End		SHIFT <u>0700</u> to <u>1900</u> Start End	
_	MODE1		MODE	
	POWER LEVEL 100	%	POWER LEVEL	 _%
	GENERATOR OUTPUT 1125	MWe	GENERATOR OUTPUT	_MWe
	CASK STORAGE GATE INSTALLED:	YES/NO	CASK STORAGE GATE INSTALLED:	YES/NO
REN	IARKS: UNIT 1			
1)	Both Units at 100% power			
2)	Remove Startup Transformer T-10 from	service for Main	tenance IAW OP-003-001.	
3)	All systems are operable.			
4)				
5)				
6)				
7)			W	
8)			1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	
9)				
10))			
11)				
(2ئر				
13))			
14))			
15)				
16))			
17)				
COM	<u>IMON</u>			
1)	Common Recombiner in S/B			
,				
		<u></u> i		
,				· · · · · · · · · · · · · · · · · · ·

UNIT SUPERVISOR TURNOVER SHEET

	UNIT:	1	Date: August 15, 2001	
·-	SHIFT 1900 to 0700 Start End		SHIFT <u>0700</u> to <u>1900</u> Start End	
	MODE1		MODE	
	POWER LEVEL 100	 %	POWER LEVEL	~~~
	GENERATOR OUTPUT 1125	^0 MWe	GENERATOR OUTPUT	⁷⁸ MWe
	CASK STORAGE GATE INSTALLED:	YES(NO)	CASK STORAGE GATE INSTALLED:	YES/NO
	CASK STORAGE GATE INSTALLED.	TESINO	CASK STORAGE GATE INSTALLED.	TES/NO
	ARKS: UNIT 1			<u> </u>
	Both Units at 100% power			
	Remove Startup Transformer T-10 from	service for Main	tenance IAW OP-003-001.	
3)	All systems are operable.			
4)				
5)				
6)				
7)				
8)				
9)				
10)				
11)				
_± <u>2)</u>				
13)				
14)				
15)				
16)				
17)				
COM	IMON	, 1,1,1,1,1,1,1,1,1,1,1,1,1,1,1,1,1,1,1		-
	<u>IMON</u>			
1)	Common Recombiner in S/B			
_				