

November 21, 1986

Docket Nos. 50-280
and 50-281

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

DISTRIBUTION

<u>Docket File</u>	T. Barnhart (8)
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T. Novak	V. Benaroya
D. Miller	Tech Branch
C. Patel	ACRS (10)
L. Harmon	C. Miles, OPA
E. Jordan	L. Tremper, LFMB
B. Grimes	Gray File
J. Partlow	

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No. 110 to Facility Operating License No. DPR-32 and Amendment No. 110 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in partial response to your application transmitted by letter dated February 14, 1979, and supplemented September 21, 1982, and August 30, 1985.

These amendments revise the Technical Specifications by deleting the inservice inspection requirement for reactor vessel closure head cladding.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Chandu P. Patel, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 110 to DPR-32
- 2. Amendment No. 110 to DPR-37
- 3. Safety Evaluation

cc: w/enclosures
See next page

*PREVIOUS CONCURRENCE SEE DATE

LA: PAD#2*	PM: PAD#2*	OGC*	PD: PAD#2
DMiller	CPatel:hc		LRubenstein
11/10/86	11/10/86	11/13/86	11/20/86

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PDR ADOCK 05000280
P PDR

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and 50-281

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Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

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These amendments revise the Technical Specifications by deleting the inservice inspection requirement for reactor vessel closure head cladding.

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Sincerely,

Chandu P. Patel, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A
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LA:PAD#2
DMY:er
11/10/86

CP
PM:PAD#2
CPatel:hc
11/10/86

OGC
JOHNSON
11/13/86

PD:PAD#2
LRubenstein
11/ /86

John
of [unclear] to
SEN



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY
DOCKET NO. 50-280
SURRY POWER STATION, UNIT NO. 1
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 14, 1979, as supplemented September 21, 1982, and August 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

8612040141 861121
PDR ADOCK 05000281
P PDR

Mr. W. L. Stewart
Virginia Electric and Power Company

Surry Power Station

cc:

Mr. Michael W. Maupin
Hunton and Williams
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Richmond, Virginia 23213

Attorney General
Supreme Court Building
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Richmond, Virginia 23219

Mr. Robert F. Saunders, Manager
Surry Power Station
Post Office Box 315
Surry, Virginia 23883

Resident Inspector
Surry Power Station
U.S. Nuclear Regulatory Commission
Post Office Box 166, Route 1
Surry, Virginia 23883

Mr. Sherlock Holmes, Chairman
Board of Supervisors of Surry County
Surry County Courthouse
Surry, Virginia 23683

W. T. Lough
Virginia Corporation Commission
Division of Energy Regulation
Post Office Box 1197
Richmond, Virginia 23209

Mr. J. T. Rhodes
Senior Vice President - Power Ops.
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
101 Marietta Street, Suite 3100
Atlanta, Georgia 30323

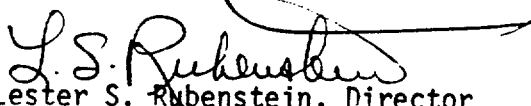
James B. Kenley, M.D., Commissioner
Department of Health
109 Governor Street
Richmond, Virginia 23219

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 21, 1986



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-37

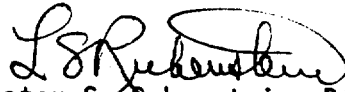
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 14, 1979, as supplemented September 21, 1982, and August 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 21, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 110 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 110 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

4.2-8

4.2-21

Insert Pages

4.2-8

4.2-21

Category N - Interior Surfaces and Internal Components of
Reactor Vessels

It is proposed that examinations in this category be made by remote television or borescopic examination. A critical examination would be made at the first refueling to detect if any changes have occurred due to initial operation. The amount of examination to be performed at subsequent refueling outages would depend upon the results of the first examination and those made on comparable pressurized-water systems.

B. Pressurizer

Category B - Pressure Containing Welds in Vessels

The primary heads on the pressurizer are of cast material and do not contain meridional welds. There are circumferential welds joining the heads to the barrel section and circumferential and longitudinal welds in the barrel section that require examination. The examinations scheduled for the 5-year period and tentatively scheduled for the 10-year period are given in TS Table 4.2-1.

Category D - Pressure Containing Nozzles in Vessels

The nozzles are integrally cast into the head, and therefore there are no welds in this category.

SECTION A. REACTOR VESSEL AND CLOSURE HEAD (Continued)

TABLE 4.2-1

<u>Item No.</u>	<u>Category</u>	<u>Required Examination Areas</u>	<u>Required Examination Methods</u>	<u>Extent of Examination Planned During First 5-Year Interval</u>	<u>Tentative Inspection During 10-Year Interval</u>	<u>Remarks</u>
1.13			(I T E M	D E L E T E D)		
1.14	I-1	Vessel Cladding	Visual	None	6 patches	During the 10-year period at least 6 patches (each 36 square inches) evenly distributed in the vessel head would be visually inspected.
1.15	N	Interior surfaces and internals and integrally welded internal supports	Visual	A critical examination will be made of the interior surfaces made available by normal refueling operations the 1st refueling cycle. This will be repeated at the 4th refueling cycle with the amount of the inspection being dependent upon results of the 1st inspection and that made on other pressurized-water systems.	The inspections made at the 4th refueling cycle will be repeated at the 7th and 10th refueling cycle	The examination will include internal support attachments welded to the vessel whose failure may adversely affect core integrity provided these are available for visual examination by components removed during normal refueling operations.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-280 AND 50-281

INTRODUCTION

By letter dated February 14, 1979, as supplemented September 21, 1982, and August 30, 1985, Virginia Electric and Power Company (the licensee), requested changes to the Technical Specifications (TS) for Surry Units 1 and 2 to meet the Inservice Inspection and Testing requirements for nuclear power plant components. The staff is currently reviewing the proposed changes. This evaluation covers only one item which is covered by Section 4.2 of Surry TS, Table 4.2-1, Section A, Item No. 1.13 which requires an examination of reactor vessel closure head cladding. The licensee has proposed to delete this requirement.

DISCUSSION AND EVALUATION

The closure head cladding examination requirement referred to above was taken, in conformance with the provisions of 10 CFR 50.55a(g), from the ASME Section XI (70W70) inservice inspection requirements in effect at the time the TSs were prepared. The examination has subsequently been determined unnecessary and omitted from later versions of Section XI which have been approved by the staff. Omission of this examination requirement from the Surry TSs is consistent with 10 CFR 50.55(a)(g)(4), which provides that inservice inspection requirements conform with the most recent version of the applicable Code provision. Therefore the staff considers the change to be acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of the facilities components located within the restricted areas as defined in 10 CFR 20 and in a surveillance requirement. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment.

on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: November 21, 1986

Principal Contributors:

E. H. Girard