

October 18, 1990

Docket Nos. 50-280  
and 50-281

DISTRIBUTION  
See attached sheet

Mr. W. L. Stewart  
Senior Vice President - Nuclear  
Virginia Electric and Power Company  
5000 Dominion Blvd.  
Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY UNITS 1 AND 2 - ISSUANCE OF AMENDMENTS RE: MANAGEMENT  
SAFETY REVIEW COMMITTEE (TAC NOS. 76012 AND 76013)

The Commission has issued the enclosed Amendment No. 146 to Facility Operating License No. DPR-32 and Amendment No. 142 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated February 1, 1990, as supplemented July 30, 1990.

These amendments replace the Independent/Operational Event Review Group with the Management Safety Review Committee (MSRC) as the organization responsible for the offsite review function.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Bart C. Buckley, Senior Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 146 to DPR-32
2. Amendment No. 142 to DPR-37
3. Safety Evaluation

cc w/enclosures:  
See next page

OFC	: LA: PD22	: PM: PD22	: D: PD22	: OGC	:	:	:
NAME	: DJM	: BBuckley	: kdj: HBerk	: [Signature]	:	:	:
DATE	: 9/19/90	: 10/1/90	: 10/1/90	: 10/2/90	:	:	:

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Document Name: SURRY 1&2 76012 AND 76013  
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PDR ADDCK 05000280  
PDC

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Surry Power Station

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DATED: October 18, 1990

AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1  
AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-37 - SURRY UNIT 2

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

G. Lainas, 14/H/3

H. Berkow

D. Miller

B. Buckley

OGC-WF

D. Hagan, 3302 MNBB

E. Jordan, 3302 MNBB

B. Grimes, 9/A/2

G. Hill (8), P-137

Wanda Jones, P-130A

J. Calvo, 11/F/23

M. Case

ACRS (10)

GPA/PA

OC/LFMB

PD Plant-specific file [Gray File]

M. Sinkule, R-II

Others as required

cc: Plant Service list

10/18/90

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11



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 146  
License No. DPR-32

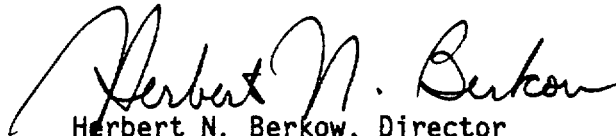
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 1, 1990, as supplemented July 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 146, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 18, 1990



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142  
License No. DPR-37

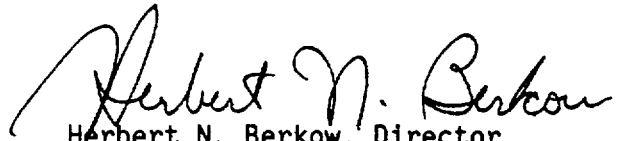
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated February 1, 1990, as supplemented July 30, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 18, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 146 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 142 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages

TS 6.1-1  
TS 6.1-2  
TS 6.1-6  
TS 6.1-7  
TS 6.1-8  
TS 6.1-8a  
TS 6.1-9  
    thru TS 6.1-15  
TS 6.2-1  
TS 6.3-1

Insert Pages

TS 6.1-1  
TS 6.1-2  
TS 6.1-6  
TS 6.1-7  
TS 6.1-8  
----  
TS 6.1-9  
    thru TS 6.1-13  
TS 6.2-1  
TS 6.3-1



6.0 ADMINISTRATIVE CONTROLS6.1 Organization, Safety and Operation Review Specification

- A. The Station Manager shall be responsible for the overall operation of the facility. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for the safe operation of the facility. During the absence of both, the Station Manager will delegate in writing the succession to this responsibility.

An onsite and an offsite organization shall be established for facility operation and corporate management. The onsite and offsite organization shall include the positions for activities affecting the safety of the nuclear power plant.

1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of departmental responsibilities and relationships, and job descriptions for key personnel positions, or in equivalent forms of documentation. These requirements shall be documented in the UFSAR.
  2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
  3. The Vice President-Nuclear Operations shall have corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
  4. The management position responsible for training of the operating staff and the position responsible for the quality assurance functions shall have sufficient organizational freedom including sufficient independence from cost and schedule when opposed to safety considerations.
  5. The management position responsible for health physics shall have direct access to that onsite individual having responsibility for overall facility management. Health Physics personnel shall have the authority to cease any work activity when worker safety is jeopardized or in the event of unnecessary personnel radiation exposures.
- B. The facility organization shall conform to the following requirements:
1. Each member of the facility staff shall meet or exceed the minimum qualifications of ANS 3.1 (12/79 Draft)\* for comparable positions, and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for the Superintendent-Radiological Protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

\* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operations Phase."

2. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.
3. The Manager - Nuclear Training is responsible for ensuring that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5.5 of ANS 3.1 (12/79 Draft)\* and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the operating experience program.
4. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.
5. A health physics technician shall be on site when fuel is in the reactor.
6. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

\* Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operations Phase."

- C. Organization units to provide a continuing review of the operational and safety aspects of the nuclear facility shall be constituted and have the authority and responsibilities outlined below:

1. Station Nuclear Safety and Operating Committee (SNSOC)

a. Function

The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

b. Composition

The SNSOC shall be composed of the:

Chairman: Assistant Station Manager, Nuclear Safety  
and Licensing

Vice Chairman and Member: Assistant Station Manager, Operations and  
Maintenance

Member: Superintendent - Operations

Member: Superintendent - Maintenance

Member: Superintendent - Radiological Protection

Member: Superintendent - Engineering

c. Alternates

All alternate members shall be appointed in writing however, no more than two alternates shall participate as voting members in SNSOC activities at any one time.

d. Meeting Frequency

The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

e. Quorum

A quorum of the SNSOC shall consist of the Chairman or Vice Chairman and two members including alternates.

f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all proposed normal, abnormal, and emergency operating procedures and all proposed maintenance procedures and changes thereto, b) any other proposed procedures or changes thereto as determined by the Station Manager which affect nuclear safety.
2. Review of all proposed test and experiment procedures that affect nuclear safety.
3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Station Manager.
5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and to the Management Safety Review Committee.
6. Review of all Reportable Events and special reports submitted to the NRC.
7. Review of facility operations to detect potential nuclear safety hazards.
8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Station Manager.

9. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.
10. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.
11. Review of every unplanned onsite release of radioactive material to the environs exceeding the limits of Specification 3.11, including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President - Nuclear Operations and to the Management Safety Review Committee.
12. Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

g. Authority

The SNSOC shall:

1. Provide written approval or disapproval of items considered under (1) through (3) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.
2. Render determinations in writing with regard to whether or not each item considered under (1) through (5) above constitutes an unreviewed safety question.
3. Provide written notification within 24 hours to the Vice President - Nuclear Operations and to the Management Safety Review Committee of disagreement between SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.A above.

h. Records

The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Nuclear Operations and to the Management Safety Review Committee.

## 2. MANAGEMENT SAFETY REVIEW COMMITTEE (MSRC)

### a. Function

The Management Safety Review Committee shall function to provide independent review and audit of designated activities in the areas of:

1. Station Operations
2. Maintenance
3. Reactivity Management
4. Engineering
5. Chemistry and Radiochemistry
6. Radiological Safety
7. Quality Assurance Practices
8. Emergency Preparedness

### b. Composition

The MSRC shall be composed of the MSRC Chairman and a minimum of four MSRC members. The Chairman and all members of the MSRC shall have qualifications that meet the requirements of Section 4.7 of ANS 3.1- (12/1979 Draft)

### c. Alternates

All alternate members shall be appointed in writing by the MSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in MSRC activities at any one time.

### d. Consultants

Consultants should be utilized as determined by the MSRC Chairman to provide expert advice to the MSRC.

e. Meeting Frequency

The MSRC shall meet at least once per calendar quarter.

f. Quorum

The minimum quorum of the MSRC necessary for the performance of the MSRC review and audit functions of these Technical Specifications shall consist of the Chairman or his designated alternate and at least 50% of the MSRC members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the unit.

g. Review

The MSRC shall be responsible for the review of:

1. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
2. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
3. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
4. Proposed changes to Technical Specifications or the Operating Licenses.

5. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
6. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety.
7. Events requiring written notification to the Commission.
8. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety.
9. Reports and meeting minutes of the SNSOC.

h. Audits

Audits of facility activities shall be performed under the cognizance of the MSRC. These audits shall encompass:

1. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
2. The performance, training and qualifications of the entire facility staff at least once per 12 months.
3. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.



4. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
5. The Emergency Plan and implementing procedures at least once per 12 months.
6. The Security Plan and implementing procedures at least once per 12 months.
7. Any other area of facility operation considered appropriate by the MSRC or the Vice President - Nuclear Operations.
8. The Fire Protection Program and implementing procedures at least once per 24 months.
9. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
10. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.
11. The radiological environmental monitoring program at least once per 12 months.
12. The Offsite Dose Calculation Manual and implementing procedures at least once per 12 months.
13. The Process Control Program and implementing procedures for processing and packaging of radioactive waste at least once per 12 months.

i. Authority

The MSRC shall report to and advise the Senior Vice President - Nuclear on those areas of responsibility specified in Sections 6.1.c.2.g and 6.1.c.2.h.

j. Records

Records of MSRC activities shall be prepared, approved and distributed as indicated below:

1. Minutes of each MSRC meeting shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days of each meeting.
2. Reports of reviews with safety significant findings encompassed by Section 6.1.c.2.g above, shall be prepared, approved and forwarded to the Senior Vice President - Nuclear within 14 days following completion of the review.
3. Audit reports encompassed by Section 6.1.c.2.h above, shall be forwarded to the Senior Vice President - Nuclear and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

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## 6.2 GENERAL NOTIFICATION AND REPORTING REQUIREMENTS

### Specification

- A. The following actions shall be taken for Reportable Events:
1. A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR, and
  2. Each Reportable Event shall be reviewed by the SNSOC. The Vice President - Nuclear Operations and the MSRC shall be notified of the results of this review.
- B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR.

### 6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

#### Specification

- A. The following actions shall be taken in the event a Safety Limit is violated:
1. The facility shall be placed in at least hot shutdown within 1 hour.
  2. The Safety Limit violation shall be reported to the Commission, the Vice President - Nuclear Operations, and the MSRC within 24 hours.
  3. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
  4. The Safety Limit Violation Report shall be submitted to the Commission, the Vice President - Nuclear Operations, and the MSRC within 14 days of the violation.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NO. DPR-32  
AND AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY  
SURRY POWER STATION, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-280 AND 50-281

INTRODUCTION

By letter dated February 1, 1990, as supplemented July 30, 1990, the Virginia Electric and Power Company (the licensee) proposed changes to the Administrative Controls Section of the Technical Specifications (TS) for the Surry Power Station, Units 1 and 2. The major purpose of the changes was to reassign the responsibility of the offsite review function from the Independent/Operational Event Review (IOER) Group to the Management Safety Review Committee (MSRC). Other minor changes were proposed to accommodate organizational changes. Below is a brief discussion and our evaluation of the proposed changes.

The July 30, 1990 letter provided additional information requested by the staff regarding the constitution of a quorum and the timeliness of the MSRC meeting minutes. The additional information did not alter in any way the staff's initial determination of no significant hazards consideration as noticed in the Federal Register on March 21, 1990 (55 FR 10547).

DISCUSSION

The essential element of the proposed TS changes is to utilize a standing committee (MSRC) to perform the responsibilities of the independent review body that were previously performed by a licensee organizational unit (IOER). The acceptability of these changes was evaluated using the acceptance criteria of the Standard Review Plan for independent review. The proposed changes are acceptable because the qualifications of the MSRC membership and the provisions described for its operations meet the criteria for a standing committee that functions as an independent review body. Additionally, the licensee organization that was previously responsible for the offsite review function (IOER) is not being dissolved but will be used by the MSRC to perform reviews and assessments of plant activities. This will further ensure that the independent review program will collectively have the experience and competence required for the review of nuclear plant activities.

The licensee utilized the Westinghouse Standard Technical Specifications section on the offsite safety review and audit function to describe the responsibilities

and activities of the MSRC. This section will replace TS Section 6.5.2 - Independent Operational Event Review Group and 6.5.3 - Quality Assurance Audits in the existing TS. Significant differences between the proposed TS and the Westinghouse Standard Technical Specifications and their acceptability are discussed below.

#### Section 6.1.C.2.a Function

The licensee modified the list of areas and activities to be reviewed by the MSRC to make it more site-specific. This change neither limits nor excludes the MSRC from any area related to nuclear power plant activities and is therefore acceptable.

#### Section 6.1.c.2.b/f Composition/Quorum

The licensee did not list the composition of the MSRC by specific position title but instead described the minimum membership, their qualification requirements and appropriate quorum. This change is acceptable because the qualifications requirements of the MSRC will meet the acceptance criteria of the Standard Review Plan (SRP). Also, the general requirements contained in these sections capture the essential elements of membership and qualification of the MSRC without reliance on specific position title.

#### Section 6.1.c.2.g Reviews

The licensee proposed to remove an existing review of the Quality Assurance program and audit results by the IOER staff. This requirement was incorporated into the existing TS because the QA Department was responsible for the TS-required audits and provided the necessary interface for the two groups. This change is acceptable because it is not one of the subjects requiring independent review under the SRP criteria and all TS audits will be performed directly under the cognizance of the MSRC and will provide the required interface with the group performing the audit.

#### Section 6.1.c.2.h Audits

The licensee proposed a change in this section that would allow the TS-required audits that are now the responsibility of the Quality Assurance Department to be performed under the cognizance of the MSRC. This change is acceptable since the Quality Assurance Department will still normally perform most of the audit activities and their activities should be enhanced by the direction, expertise, and independent outlook provided by MSRC oversight. Additionally, this change will bring the Surry TS into conformance with the Westinghouse Standard Technical Specifications.

In addition, the licensee proposed other minor changes to the TS in order to reflect organizational changes. These changes are discussed below.



### Section 6.1.c.1. Station Nuclear Safety and Operating Committee (SNSOC)

The licensee proposed two changes that would affect the composition of the SNSOC. The first was a clarification that the SNSOC Vice Chairman can also function as a member (when the SNSOC Chairman is presiding) and the second was the elimination of the position of Superintendent-Technical Services. These changes are acceptable because the committee will still meet the criteria to provide interdisciplinary review of subject matter as required by the SRP.

### Sections 6.1, 6.2, and 6.3 Title Changes

The licensee proposes to make the following title changes in the above listed TS sections:

- Senior Vice President-Power to Senior Vice President-Nuclear
- Vice President-Nuclear to Vice President-Nuclear Operations
- Superintendent-Health Physics to Superintendent-Radiological Protection

These changes are acceptable because they do not alter the qualifications, responsibilities, or relationships established by the Technical Specifications.

### Section 6.1.b.3 Training

Due to organizational changes, the licensee modified this section to reflect that the Manager-Nuclear Training (instead of the Station Manager) is functionally responsible for the training program and that relevant industry events are identified by the operating experience program (the Safety Evaluation and Control Staff was eliminated). These changes do not alter the training requirements contained in this TS section but merely adapt it to utility-specific structure and are thereby acceptable.

### EVALUATION

In summary, the requested changes are acceptable because they do not diminish the licensee's technical qualification to engage in licensed activities as required by 10 CFR Part 50, Section 50.40b, and will increase the licensee's senior management's responsibility and involvement in the review and assessment of Surry Units 1 and 2 activities.

### ENVIRONMENTAL CONSIDERATION

These amendments relate to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 18, 1990

Principal Contributor: M. Case