Docket No. 50-280

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Mr. W. L. Stewart Senior Vice President - Nuclear Virginia Electric and Power Company 5000 Dominion Blvd. Glen Allen, Virginia 23060

Dear Mr. Stewart:

SUBJECT: SURRY UNIT 1 - ISSUANCE OF AMENDMENT RE: SURVEILLANCE REQUIREMENTS FOR THE WEIGHT-LOADED CHECK VALVE (TAC NO. 75763)

The Commission has issued the enclosed Amendment No. 140 to Facility Operating License No. DPR-32 for the Surry Power Station, Unit No. 1. The amendment consists of changes to the Technical Specifications in partial response to your application transmitted by letter dated January 8, 1990. One issue remains under review from your application.

The amendment postpones the requirement to cycle and verify open each weight-loaded check valve in the Containment and Recirculation Spray Systems until the next refueling outage.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By

Bart C. Buckley, Senior Project Manager Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 140 to DPR-32
- 2. Safety Evaluation

cc w/enclosures: See next page

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DATED: April 26,1990

AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE NO. DPR-32 - SURRY UNIT 1

Docket File NRC & Local PDRs PDII-2 Reading S. Varga, 14/E/4 G. Lainas, 14/H/3 H. Berkow D. Miller B. Buckley OGC-WF D. Hagan, 3302 MNBB E. Jordan, 3302 MNBB B. Grimes, 9/A/2 G. Hill (4), P1-137 Wanda Jones, P-130A J. Calvo, 11/F/23 ACRS (10) GPA/PA OC/LFMB PD Plant-specific file [Gray File]

cc: Plant Service list

M. Sinkule, R-II T. Sullivan



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

April 26, 1990

Docket No. 50-280

Mr. W. L. Stewart Senior Vice President - Nuclear Virginia Electric and Power Company 5000 Dominion Blvd. Glen Allen, Virginia 23060

Dear Mr. Stewart:

SURRY UNIT 1 - ISSUANCE OF AMENDMENT RE: SURVEILLANCE SUBJECT:

REQUIREMENTS FOR THE WEIGHT-LOADED CHECK VALVE (TAC NO. 75763)

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Sincerely,

Bart C. Buckley

Bart C. Buckley, Senior Project Manager

Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

Amendment No. 140to DPR-32

Safety Evaluation

cc w/enclosures: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 140 License No. DPR-32

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Virginia Electric and Power Company (the licensee) dated January 8, 1990 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 140, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Werbert N. Berkow, Director Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: April 26, 1990

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 140 FACILITY OPERATING LICENSE NO. DPR-32

DOCKET NO. 50-280

Revise Appendix A as follows:

Remove Page

Insert Page

TS 4.5-2

TS 4.5-2

- 2. By verifying that each motor-operated valve in the recirculation spray flow paths performs satisfactorily when tested in accordance with Specification 4.0.3.
- 3. At least once per 5 years, coincident with the closest refueling outage, by performing on air or smoke flow test and verifying each spray nozzle is unobstructed.
- C. Each weight-loaded check valve in the containment spray and outside containment recirculation spray subsystems shall be demonstrated operable at least once per 18 months,* during shutdown, by cycling the valve one complete cycle of full travel and verifying that each valve opens when the discharge line of the pump is pressurized with air and seats when a vacuum is applied.
- D. A visual inspection of the containment sump and the inside containment recirculation spray pump wells and the engineered safeguards suction inlets shall be performed at least once each refueling period and/or after major maintenance activities in the containment. The inspection should verify that the containment sump and pump wells are free of debris that could degrade system operation and that the sump components (i.e., trash racks, screens) are properly installed and show no sign of structural distress or excessive corrosion.

Basis

The flow testing of each containment spray pump is performed by opening the normally closed valve in the containment spray pump recirculation line returning water to the refueling water storage tank. The containment spray pump is operated and a quantity of water recirculated to the refueling water storage tank. The discharge to the tank is divided into two fractions; one for the major portion of the recirculation flow and the other to pass a small quantity of water through test nozzles which are identical with those used in the containment spray headers. The purpose of the recirculation through the test nozzles is to assure that there are no particulate material in the refueling water storage tank small enough to pass through pump suction strainers and large enough to clog spray nozzles.

* This Unit 1 test requirement can be extended to the Cycle 10 refueling outage.

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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 140 TO FACILITY OPERATING LICENSE NO. DPR-32

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NO. 1

DOCKET NO. 50-280

1.0 INTRODUCTION

By letter dated January 8, 1990, Virginia Electric and Power Company (the licensee) submitted an application to amend the Surry Unit 1 and 2 Technical Specifications. One of the requested changes would postpone the requirement of Technical Specification 4.5.C for Unit 1 which is due to be performed by May 1990. The licensee has requested that this surveillance be postponed until the next refueling outage, which is expected to be during the fourth quarter of 1990.

Technical Specification 4.5.C requires each weight-loaded check valve in the Containment Spray and Recirculation Spray Systems to be cycled one complete cycle of full travel and verified open on air pressure and seated with a vacuum at least once per 18 months.

2.0 DISCUSSION AND EVALUATION

In order to perform this surveillance, scaffolding must be erected in containment and test equipment installed in the containment. Setting up for and performing this test in a subatmospheric containment would expose personnel to risks due to the need to use a breathing apparatus and the time needed to allow for the differences in pressure upon entry and exit from containment. Because of these risks, performance of this surveillance as required by the existing Technical Specification (TS) 4.5.C would necessitate a plant shutdown. The spray systems are normally dry and the only service that they are exposed to is during the testing of these valves. The testing of the containment spray valves was last performed on May 28, 1989, and the testing of the recirculation spray valves was last performed on June 20, 1988.

In a meeting on February 16, 1990, between the licensee and NRC, a review of the results of past surveillances of these valves under TS 4.5.C and under the Appendix J Type C testing was conducted. A review of the maintenance history of these valves was also performed. As a result of the review, it was concluded that the surveillance and maintenance history of these valves indicates that they have performed reliably.

Based on the service history of these valves, the acceptability of these valves when they were last tested, and the benign service and environment to which the valves are normally exposed, the staff has concluded that the proposed change to TS 4.5.C, to extend the test requirement to the Cycle 10 refueling outage, should be approved.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and a change to a surveillance requirement. We have determined that this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 26, 1990

Principal Contributors:

T. Sullivan

B. Buckley