

As a reference, these conditions are further described in Supplement No. 4 to the SER (NUREG-0422), Appendix D, items 3b, 4a, 4f and 9b, respectively.

The licensee shall complete each of the following conditions to the satisfaction of the NRC by the times indicated. Each of the following conditions references the appropriate item in Section 22.3, "Full-Power Requirements", in SER Supplement 5, NUREG-0422:

a. NSSS Vendor Review Procedures (I.C.7)

Prior to exceeding 5% power, the licensee shall document that the Westinghouse review of the power ascension test procedures is complete.

b. Training During Low-Power Testing (I.G.1)

Prior to exceeding 5% power the licensee shall complete the required Special Tests and the low-power test training program. The results of the test program shall be provided to the NRC within 30 days.

c. Deleted

d. Training for Mitigating Core Damage (II.B.4)

Prior to exceeding 5% power the licensee shall complete training for mitigating core damage.

e. Auxiliary Feedwater System Evaluation (II.E.1.1)

Prior to exceeding 5% power the licensee shall complete performance testing of the auxiliary feedwater system pumps and shall submit a report within 30 days after all tests are completed.

f. Inadequate Core Cooling Instruments (II.F.2)

- (1) The licensee shall install a reactor vessel water level instrumentation system prior to startup after the first refueling.
- (2) Prior to exceeding 5% power the licensee shall install a full range in-core thermocouple temperature (2300° F) backup display; and
- (3) The licensee shall upgrade the in-containment portion of the incore thermocouple system prior to startup following the first refueling outage, and shall provide a schedule for upgrade of the remainder of the system in the Regulatory Guide 1.97 Accident Monitoring Review Report submittal pursuant to NUREG 0737, Supplement 1.

5.5 Programs and Manuals

5.5.2 Containment Leakage Rate Testing Program (continued)

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 14.8 psig. The maximum allowable containment leakage rate, L_a , at P_a , shall be 0.3% of containment air weight per day.

Leakage Rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first plant startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.75 L_a$ for Type A tests.

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

5.5.3 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Containment Spray, Safety Injection, Chemical and Volume Control, Nuclear Sampling, RHR, Boron Recycle, Refueling Water, Liquid Waste, and Waste Gas. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements; and
- b. Integrated leak test requirements for each system at refueling cycle intervals or less.

5.5.4 Deleted

b. Deleted

c. Inadequate Core Cooling Instruments (II.F.2)

- 1) Prior to startup following the first refueling outage, the licensee shall install a reactor vessel water level instrumentation system, and
- 2) Prior to startup following the first refueling outage, the licensee shall upgrade the in-containment portion of the incore thermocouple system and provide a schedule for update of the remainder of the system.

d. Anticipatory Reactor Trip (II.K.3.10)

Prior to exceeding 50% power the licensee shall complete the described turbine trip tests to verify that PORVs will not be challenged when the anticipatory trip bypass is in effect.

e. Hydrogen Control Measures (II.B.7)

- 1) Prior to startup following the first refueling outage, the licensee shall:
 - a. Install two additional igniter units in the containment lower compartment and four additional igniter units in the containment upper compartment in locations acceptable to the NRC staff.
 - b. Provide a means acceptable to the NRC staff of verifying the operational status of the hydrogen control system in the main control room.
 - c. Provide the capability to actuate the Hydrogen Mitigation System from the control room.
- 2) Operation of the hydrogen mitigation igniter system shall be activated upon a safety injection signal with accompanying indications of a loss of coolant accident.

f. Emergency Response Capability (I.C.1, I.D.1, I.D.2, III.A.1.2, III.A.2.2)

- 1) By April 15, 1983, the licensee shall submit a response to NRC generic letter 82-33, dated December 17, 1982, related to emergency response capabilities.
- 2) The licensee shall maintain interim emergency support facilities (Technical Support Center, Operations Support Center and the Emergency Operations Facility) until the upgraded facilities are completed.

5.5 Programs and Manuals

5.5.2 Containment Leakage Rate Testing Program (continued)

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 14.8 psig. The maximum allowable containment leakage rate, L_a , at P_a , shall be 0.3% of containment air weight per day.

Leakage Rate acceptance criteria are:

- a. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first plant startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.75 L_a$ for Type A tests.

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

5.5.3 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Containment Spray, Safety Injection, Chemical and Volume Control, Nuclear Sampling, RHR, Boron Recycle, Refueling Water, Liquid Waste, and Waste Gas. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements; and
- b. Integrated leak test requirements for each system at refueling cycle intervals or less.

5.5.4 Deleted