



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 14, 2001

Mr. Dale E. Young, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - ISSUANCE OF CORRECTED PAGE FOR
AMENDMENT REGARDING CONTAINMENT LEAKAGE RATE TESTING
PROGRAM (TAC NO. MB1349)

Dear Mr. Young:

Following the issuance of Amendment No. 197 to Facility Operating License No. DPR-72 for Crystal River Unit 3 (CR-3), CR-3 Licensing staff noted minor editorial errors on one Technical Specification (TS) page. The editorial changes to page 5.0-23A are corrections of typographical errors to the numbering of Improved Technical Specifications 5.6.2.19 (paragraphs "c" and "d"). This letter forwards the corrected page for insertion into the amendment. We regret any inconvenience this may have caused.

Sincerely,

A handwritten signature in cursive script, appearing to read "John M. Goshen".

John M. Goshen, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosures: Corrected TS page 5.0-23A

cc w/enclosures: See next page

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Florida Power Corporation

**CRYSTAL RIVER UNIT NO. 3
GENERATING PLANT**

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5.6 Procedures, Programs and Manuals

5.6.2.19 Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR) (continued)

- c. The reactor vessel pressure and temperature limits, including those for heatup and cooldown rates, shall be determined so that all applicable limits (e.g., heatup limits, cooldown limits, and inservice leak and hydrostatic testing limits) of the analysis are met.
- d. The PTLR, including revisions or supplements thereto, shall be provided upon issuance for each reactor vessel fluency period.

5.6.2.20 Containment Leakage Rate Testing Program

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak Test Program," dated September 1995, as modified by the following exception:

1. NEI 94-01-1995, Section 9.2.3: The first Type A test performed after the November 7, 1991 Type A test shall be performed no later than November 6, 2006.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P_a , is 54.2 psig. The containment design pressure is 55 psig.

The maximum allowable primary containment leakage rate, L_a , at P_a , shall be 0.25% of primary containment air weight per day.

Leakage Rate acceptance criteria are:

1. Containment leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $\leq 0.60 L_a$ for the Type B and Type C Tests and $\leq 0.75 L_a$ for Type A Tests.
2. Air lock testing acceptance criteria are:
 - a. Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 - b. For each door, leakage rate is $\leq 0.01 L_a$ when tested at ≥ 8.0 psig.

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.