Docket Nos. 50-280
and 50-281

December 11, 1985

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R. Diggs
R. Ballard
M. Schoppman

Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
Richmond, Virginia 23261

Dear Mr. Stewart:

The Commission has issued the enclosed Amendment No.104 to Facility Operating License No. DPR-32 and Amendment No. 104 to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letter dated March 16, 1982, as supplemented June 28, August 3, and August 9, 1982, June 30, and October 27, 1983, March 22, and November 2, 1984, and April 17, and August 30, 1985; and by letter dated November 30, 1984, as supplemented April 12 and August 30, 1985.

These amendments revise the Technical Specifications to reflect changes to the VEPCO offsite and the Surry Power Station organizations, and revise the immediate notification requirements and the Licensee Event Reporting System to be consistent with Sections 50.72 and 50.73 of 10 CFR Part 50.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

/s/TChan

Terence L. Chan, Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A

Enclosures:
1. Amendment No. 104 to DPR-32
2. Amendment No. 104 to DPR-37
3. Safety Evaluation

cc: w/enclosures
See next page

* SEE PREVIOUS CONCURRENCE

*ORB#1:DL	PWR#2-A	*BC-ORB#1:DL	*OELD	D/PWR#2-A
CParrish	TChan;ps	SVarga		LRubenstein
11/14/85	12/9/85	11/18/85	11/25/85	12/9/85

8512300531 851211
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Mr. W. L. Stewart
Vice President - Nuclear Operations
Virginia Electric and Power Company
Post Office Box 26666
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The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-32 and Amendment No. to Facility Operating License No. DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letter dated March 16, 1982, as supplemented June 28, August 3, and August 9, 1982, June 30, and October 27, 1983, March 22, and November 2, 1984, and April 17, and August 30, 1985; and by letter dated November 30, 1984, as supplemented April 12 and August 30, 1985.

These amendments revise the Technical Specifications to reflect changes to the VEPCO offsite and the Surry Power Station organizations, and revise the immediate notification requirements and the Licensee Event Reporting System to be consistent with Sections 50.72 and 50.73 of 10 CFR Part 50.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Terence L. Chan, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. to DPR-32
2. Amendment No. to DPR-37
3. Safety Evaluation

cc: w/enclosures
See next page

ORB#1:DL	ORB#1:DL	BC-ORB#1:DL	OELD	AD-OR:DL
CParrish	TChan;ps	SVarga		GLainas
11/ /85	11/ /85	11/ /85	11/25/85	11/ /85

See changes to Env. Cons. & Sec 9 SER

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Virginia Electric and Power Company

Surry Power Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 104
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 16, 1982, as supplemented June 28, August 3, and August 9, 1982, June 30, and October 27, 1983, March 22, and November 2, 1984, and April 17, and August 30, 1985; and application for amendment dated November 30, 1984, as supplemented April 12 and August 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-32 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

L S Rubenstein

Lester S. Rubenstein
PWR Project Directorate #2
Division of PWR Licensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 11, 1985

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.104
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Virginia Electric and Power Company (the licensee) dated March 16, 1982, as supplemented June 28, August 3, and August 9, 1982, June 30, and October 27, 1983, March 22, and November 2, 1984, and April 17, and August 30, 1985; and application for amendment dated November 30, 1984, as supplemented April 12 and August 30, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-37 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 104, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

L. S. Rubenstein

Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR LIcensing-A

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 11, 1985

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 104 FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 104 FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A as follows:

Remove Pages	Insert Pages
1.0-5	1.0-5
3.1-15a	3-1-15a
3.1-24	3.1-24
3.7-21	3.7-21
3.11-2	3.11-2
3.11-3	3.11-3
3.11-4	3.11-4
3.11-5	3.11-5
3.11-6	3.11-6
3.11-7	3.11-7
3.12-7	3.12-7
4.9-15	4.9-15
4.19-8	4.19-8
4.19-10	4.19-10
Table 4.19-2	Table 4.19-2
6.1-1	6.1-1
6.1-2	6.1-2
6.1-6	6.1-6
6.1-7	6.1-7
6.1-8	6.1-8
6.1-11	6.1-11
6.1-12	6.1-12
6.1-15	6.1-15
Figure 6.1-1	Figure 6.1-1
Figure 6.1-2	Figure 6.1-2
6.2-1	6.2-1
6.3-1	6.3-1

Remove Pages	Insert Pages
6.4-2	6.4-2
6.4-3	6.4-3
6.4-4	6.4-4
6.4-5	6.4-5
6.5-1	6.5-1
6.6-1	6.6-1
6.6-2	6.6-2
6.6-4	6.6-4
6.6-5	6.6-5
6.6-6	-----
6.6-7	-----
6.6-8	-----
6.6-9	-----
6.6-10	6.6-10
6.6-12	6.6-12
6.6-15	6.6-15
6.6-16	6.6-16
6.6-17	6.6-17

for operational activities provided that they are under administrative control and are capable of being closed immediately if required.

2. Blind flanges are installed where required.

3. The equipment access hatch is properly closed and sealed.

4. At least one door in the personnel air lock is properly closed and sealed.

5. All automatic containment isolation valves are operable or are locked closed under administrative control.

6. The uncontrolled containment leakage satisfied Specification 4.4.

I. <u>Reportable Event</u>

A Reportable Event shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

2. The specific activity of the reactor coolant shall be limited to ≤ 1.0 μCi/cc DOSE EQUIVALENT-131 whenever the reactor is critical or the average temperature is greater than 500°F.

3. The requirements of D-2 above may be modified to allow the specific activity of the reactor coolant >1.0 μCi/cc DOSE EQUIVALENT I-131 but less than 10.0 μCi/cc DOSE EQUIVALENT I-131. Following shutdown, the unit may be restarted and/or operation may continue for up to 48 hours provided that operation under these circumstances shall not exceed 10 percent of the unit's total yearly operating time. With the specific activity of the reactor coolant >1.0 μCi/cc DOSE EQUIVALENT I-131 for more than 48 hours during one continuous time interval or exceeding 10.0 μCi/cc DOSE EQUIVALENT I-131, the reactor shall be shut down and cooled to 500°F or less within 6 hours after detection. With the total cumulative operating time at a primary coolant specific activity > 1.0 μCi/cc DOSE EQUIVALENT I-131 exceeding 300 hours in any consecutive 6 month period, prepare and submit a Special Report to the NRC, Regional Administrator, Region II, within 30 days indicating the number of hours above this limit.

4. If the specific activity of the reactor coolant exceeds 1.0 μCi/cc DOSE EQUIVALENT I-131 or 100/E μCi/cc, a report shall be prepared and submitted to the Commission pursuant to Specification 6.2. This report shall contain the results of the specific activity analysis together with the following information:

   a. Reactor power history starting 48 hours prior to the first sample in which the limit was exceeded,

   b. Fuel burnup by core region,

   c. Clean-up flow history starting 48 hours prior to the first sample in which the limit was exceeded,

b. With both PORV's inoperable, depressurize the RCS within 8 hours unless Specification 3.1.G.1.b.(4) is in effect. When the RCS has been depressurized, open one PORV or establish the conditions listed below. Maintain the RCS depressurized until both PORV's have been restored to operable status.

(1) A maximum pressurizer narrow range level of 33%.

(2) The series RHR inlet valves open and their respective breakers locked open or an alternate letdown path operable.

(3) Limit charging flow to <150 gpm.

(4) Safety Injection accumulator discharge valves closed and their respective breakers locked open.

c. When the conditions noted in 3.1.G.2.b.(1) through 3.1.G.2.b.(4) above are required to be established, their implementation shall be verified at least once per 12 hours.

3. In the event that the Reactor Coolant System Overpressure Mitigating System is used to mitigate a RCS pressure transient, a Special Report shall be prepared and submitted to the Commission pursuant to Specification 6.6 within 30 days. The report shall describe the circumstances initiating the transient, the effect of the mitigating system or the administrative controls on the transient and any corrective actions necessary to prevent recurrence.

Basis

The operability of two PORV's or the RCS vented through an opened PORV ensures that the Reactor Vessel will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when the Reactor Coolant average temperature is ≤350°F and the Reactor Vessel Head is bolted. When the Reactor Coolant average temperature is >350°F, overpressure protection is provided by a bubble in the pressurizer and/or pressurizer safety valves. A single PORV has adequate relieving

TABLE 3.7-6
ACCIDENT MONITORING INSTRUMENTATION

	INSTRUMENT	TOTAL NO. OF CHANNELS	MINIMUM CHANNELS OPERABLE
1.	Auxiliary Feedwater Flow Rate	1 per S/G	1 per S/G
2.	Reactor Coolant System Subcooling Margin Monitor	2	1
3.	PORV Position Indicator (Primary Detector)	1/valve	1/valve
4.	PORV Position Indicator (Backup Detector)	1/valve	0
5.	PORV Block Valve Position Indicator	1/valve	1/valve
6.	Safety Valve Position Indicator (Primary Detector)	1/valve	1/valve
7.	Safety Valve Position Indicator (Backup Detector)	1/valve	0
8.	Reactor Vessel Coolant Level Monitor	2	1
9.	Containment Pressure	2	1
10.	Containment Water Level (Narrow Range)	2	1
11.	Containment Water Level (Wide Range)	2	1
12.	Contaiment High Range Radiation Monitor	2	1 (Note 1, b and c only)
13.	Process Vent High Range Effluent Monitor	2	2 (Note 1, a, b, and c)
14.	Ventilation Vent High Range Effluent Monitor	2	2 (Note 1, a, b, and c)
15.	Main Steam High Range Radiation Monitors (Units 1 and 2)	3	3 (Note 1, a, b, and c)
16.	Aux. Feed Pump Steam Turbine Exhaust Radiation Monitor	1	1 (Note 1, a, b, and c)

Note 1: With the number of operable channels less than required by the Minimum Channels Operable requirements
- a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), within 72 hours
- b. Either restore the inoperable channel to operable status within 7 days of the event, or
- c. Prepare and submit a Special Report to the commission pursuant to specification 6.2 within 30 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable.

c. The surveillance requirements for liquid effluents are given in Table 4.9-1.

d. The reporting requirements of section 6.2 are not applicable.

2. <u>Dose</u>

a. The dose or dose commitment to the maximum exposed member of the public from radioactive materials in liquid effluents released, from each reactor unit, to unrestricted areas shall be limited:

(i) During any calendar quarter to less than or equal to 1.5 mrems to the total body and to less than or equal to 5 mrems to the critical organ, and

(ii) During and calendar year to less than or equal to 3 mrems to the total body and to less than or equal to 10 mrems to the critical organ.

b. With the calculated dose from the release of radioactive materials in liquid effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days, pursuant to Specification 6.2, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

3. Liquid Radwaste Treatment

a. The Liquid Radwaste Treatment System shall be used to reduce the redioactive materials in liquid waste prior to their discharge when the projected dose due to liquid effluent releases to unrestricted areas (see figure 5.1-1) when averaged over 31 days would exceed 0.06 mrem to the total body or 0.2 mrem to the total body or 0.2 mrem to the critical organ.

b. With radioactive liquid waste being discharged without treatment and in excess of the above limits, prepare and submit to the Commission within 30 days pursuant to Specification 6.2 a Special Report that includes the following information:

(i) Explanation of why liquid radwaste was being discharged without treatment, identification of any inoperable equipment or sub-system, and the reason for the inoperability,

(ii) Action(s) taken to restore the inoperable equipment to operable status, and

(iii) Summary description of action(s) taken to prevent a recurrence.

B. Gaseous Effluents

1. Dose Rate

a. The dose rate due to radioactive materials released in gaseous effluents from the site to areas at and beyond the site boundary (see figure 5.1-1) shall be limited to the following:

(i) For noble gases: less than or equal to 500 mrems/yr. to the total body and less than or equal to 3000 mrems/yr. to the skin, and

(ii) For iodine-131, for tritium, and for all radionuclides in particulate form with half lives greater that 8 days: less than or equal to 1500 mrems/yr. to the critical organ.

b. With the dose rate(s) exceeding the above limits, without delay restore the release rate to within the above limit(s).

c. The reporting requirements of section 6.2 are not applicable.

2. Dose-Noble Gases

a. The air dose due to noble gases released in gaseous effluents, from each reactor unit, from the site to areas at and beyond the site boundary (see figure 5.1-1) shall be limited to the following:

(i) During any calendar quarter: less than or equal to 5 mrads for gamma radiation and less than or equal to 10 mrads for beta radiation and,

(ii) During any calendar year: less than or equal to 10 mrads for gamma radiation and less than or equal to 20 mrads for beta radiation.

b. With the calculated air dose from radioactive noble gases in gaseous effluents exceeding any of the above limits, prepare and submit to the Commission within 30 days, pursuant to Specification 6.2, a Special Report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

3. Dose-I-131, Tritium, and Radionuclides in Particulate Form

a. The dose to the maximum exposed member of the public from all I-131, from tritium, and from all radionuclides in particulate form with half-lives greater that 8 days in gaseous effluents released, from each reactor unit, from the site to areas at and beyond the site boundary (see figure 5.1-1) shall be limited to the following:

(i) During any calendar quarter: less than or equal to 7.5 mrems to the critical organ, and

(ii) During any calendar year: less than or equal to 15 mrems to the critical organ.

b. With the calculated dose from the release of I-131, tritium, and radionuclides in particulate form with half lives greater than 8 days, in gaseous effluents exceeding any of the above limits, prepare and submit to the commission within 30 days, pursuant to Specification 6.2, a Special Report that identifies the cause(s) for exceeding the limit and defines the corrective actions that have been taken to reduce the releases and the proposed corrective actions to be taken to assure that subsequent releases will be in compliance with the above limits.

4. Gaseous Radwaste Treatment

a. The appropriate portions of the Gaseous Radwaste Treatment System shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected gaseous effluent air doses due to gaseous effluent releases, from each reactor unit, from the site to areas at and beyond the site boundary (see Figure 5.1-1) would exceed 0.2 mrad for gamma radiation and 0.4 mrad for beta radiation when averaged over 31 days.

b. The Ventilation Exhaust Treatment System shall be used to reduce radioactive materials in gaseous waste prior to their discharge when the projected doses due to gaseous effluent releases, from each reactor unit, from the site to areas at and beyond the site boundary (see Figure 5.1-1) would exceed 0.3 mrem to the critical organ when averaged over 31 days.

c. With gaseous waste being discharged without treatment and in excess of the above limits,prepare and submit to the Commission within 30 days, pursuant to Specification 6.2, a Special Report that includes the following information:

(i) Explanation of why gaseous radwaste was being discharged without treatment, identification of any inoperable equipment or sub-systems, and the reason for the inoperability,

(ii) Action(s) taken to restore the inoperable equipment to operable status, and

(iii) Summary description of action(s) taken to prevent a recurrence.

5. Explosive Gas Mixture

a. The concentration of hydrogen or oxygen in the waste gas holdup system shall be limited to less than or equal to 4% by volume.

b. With the concentration of hydrogen or oxygen in the waste gas holdup system exceeding the limit, restore the concentration to within the limit within 48 hours.

6. Gas Storage Tanks

a. The quantity of radioactivity contained in each gas storage tank shall be limited to less than or equal to 24,600 curies of noble gases (considered as Xe-133).

a. The hot channel factors shall be determined within 2 hours and the power level adjusted to meet the requirement of Specification 3.12.B.1, or

b. If the hot channel factors are not determined within two hours, the power level and high neutron flux trip setpoint shall be reduced from rated power 2% for each percent of quadrant tilt.

c. If the quadrant to average power tilt exceeds ± 10%, the power level and high neutron flux trip setpoint will be reduced from rated power 2% for each percent of quadrant tilt.

7. If, except for physics and rod exercise testing, after a further period of 24 hours, the power tilt in Specification 3.12.B.5 above is not corrected to less than 2%:

a. If design hot channel factors for rated power are not exceeded, an evaluation as to the cause of the discrepancy shall be made and a special report issued to the Nuclear Regulatory Commission.

b. If the design hot channel factors for rated power are exceeded and the power is > 10%, the Nuclear Regulatory Commission shall be notified and the Nuclear Overpower, Nuclear Overpower ΔT, and Overtemperature ΔT trips shall be reduced 1% for each percent the hot channel factor exceeds the rated power design values.

c. If the hot channel factors are not determined, the Nuclear Regulatory Commission shall be notified and the Overpower

E is the counting efficiency (as counts per disintegration),

V is the sample size (in units of mass or volume),

2.22 is the number of disintegrations per minute per picocurie,

Y is the fractional radiochemical yield (when applicable),

is the radioactive decay constant for the particular radionuclide, and

t for environmental samples is the elapsed time between sample collection (or end of the sample collection period) and time of counting

Typical values of E, V, Y, and t should be used in the calculation.

It should be recognized that the LLD is defined as an <u>a priori</u> (before the fact) limit representing the capability of a measurement system and not as <u>a posteriori</u> (after the fact) limit for a particular measurement. Analyses shall be performed in such a manner that the stated LLDs will be achieved under routine conditions. Occasionally background fluctuations, unavoidable small sample sizes, the presence of interfering nuclides, or other uncontrollable circumstances may render these LLDs unachievable. In such cases, the contributing factors shall be identified and described in the Annual Radiological Environmental Operating Report pursuant to Specification 6.6.b.2.

F. Reports

a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.

b. The complete results of the steam generator tube inservice inspection shall be reported on an annual basis for the period in which the inspection was completed. This report shall include:

1. Number and extent of tubes inspected.

2. Location and percent of wall-thickness penetration for each indication of an imperfection.

3. Identification of tubes plugged.

c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported by special report prior to resumption of plant operation. The report shall provide a description of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.

withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 500 gallons per day per steam generator can readily be detected by radiation monitors of steam generator blowdown. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged.

Wastage-type defects are unlikely with the all volatile treatment (AVT) of secondary coolant. However, even if a defect of similar type should develop inservice, it will be found during scheduled inservice steam generator tube examination. Plugging will be required of all tubes with imperfections exceeding the plugging limit which, by the definition of Specification 4.19.E.a, if 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability of reliably detecting degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be reported to the Commission by special report prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations tests, additional eddy current inspection, and revision of the Technical Specification, if necessary.

Amendment Nos. 104 and 104

TAB[illegible].19-2

STEAM GENERATOR TUBE INSPECTION

1st SAMPLE INSPECTION			2nd SAMPLE INSPECTION		3rd SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S.G.	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 2S tubes in this S.G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect addit- 4S tubes in this S.G.	C-1	None
					C-2	Plug defective tubes
					C-3	Perform action for C-3 result of first sample
			C-3	Perform action for C-3 result of first sample	N/A	N/A
	C-3	Inspect all tubes in this S.G., plug defec- tive tubes & inspect 2S tubes in each other S.G. Special Report	All other S.G.s are C-1	None	N/A	N/A
			Some S.G.s C-2 but no additional S.G. are C-3	Perform action for C-2 result of second sample	N/A	N/A
			Additional S.G. is C-3	Inspect all tubes in each S.G. and plug defective tubes Special Report	N/A	N/A

$S=3\frac{N}{n}\%$ Where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection

6.0 ADMINISTRATIVE CONTROLS

6.1 Organization, Safety and Operation Review

Specification

A. The Station Manager shall be responsible for the overall operation of the facility. In his absence, the Assistant Station Manager (Operations and Maintenance) shall be responsible for the safe operation of the facility. During the absence of both, the Station Manager will delegate in writing the succession to this responsibility.

1. The off-site organization for facility management and technical support shall be as shown on TS Figure 6.1-1.

B. The Station organization shall conform to the chart as shown on TS Figure 6.1-2.

1. Each member of the facility staff shall meet or exceed the minimum qualifications of ANS 3.1 (12/79 Draft) * for comparable positions, and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, except for the Superintendent - Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

*Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operations Phase."

2. The Shift Technical Advisor shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

3. The Station Manager is responsible for ensuring that re-training and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5.5 of ANSI (12/79 Draft)* and Appendix "A" of 10 CFR Part 55 and the supplemental requirements specified in the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience identified by the SEC Staff.

4. Each on-duty shift shall be composed of at least the minimum shift crew composition for each unit as shown in Table 6.1-1.

5. A health physics technician shall be on site when fuel is in the reactor.

6. All core alterations shall be observed and directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

*Exceptions to this requirement are specified in VEPCO's QA Topical Report, VEP-1, "Quality Assurance Program, Operations Phase."

C. Organization units to provide a continuing review of the operational and safety aspects of the nuclear facility shall be constituted and have the authority and responsibilities outlined below:

1. Station Nuclear Safety and Operating Committee (SNSOC)

a. Function

The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

b. Composition

The SNSOC shall be composed of the:

Chairman	Assistant Station Manager, Nuclear Safety and Licensing
Vice Chairman	Assistant Station Manager, Operations and Maintenance
Member	Superintendent - Operations
Member	Superintendent - Maintenance
Member	Superintendent - Technical Services
Member	Superintendent - Health Physics

c. Alternates

All alternate members shall be appointed in writing however, no more than two alternates shall participate as voting members in SNSOC activities at any one time.

d. Meeting Frequency

The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

e. Quorum

A quorum of the SNSOC shall consist of the Chairman or Vice Chairman and two members including alternates.

f. Responsibilities

The SNSOC shall be responsible for:

1. Review of a) all proposed normal, abnormal, and emergency operating procedures and all proposed maintenance procedures and changes thereto, b) any other proposed procedures or changes thereto as determined by the Station Manager which affect nuclear safety.

2. Review of all proposed test and experiment procedures that affect nuclear safety.

3. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.

4. Review of proposed changes to Technical Specifications and shall submit recommended changes to the Station Manager.

5. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Vice President - Nuclear Operations and to the Director - Safety Evaluation and Control.

6. Review of all Reportable Events and special reports submitted to the NRC.

7. Review of facility operations to detect potential nuclear safety hazards.

8. Performance of special reviews, investigations or analyses and report thereon as requested by the Chairman of the SNSOC or Station Manager.

9. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Station Manager.

10. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Station Manager.

11. Review of every unplanned onsite release of radioactive material to the environs exceeding the limits of Specification 3.11, including the preparation or reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President-Nuclear Operations and to the Director-Safety Evaluation and Control.

12. Review of changes to the Process Control Program and the Offsite Dose Calculation Manual.

g. Authority

The SNSOC shall:

1. Provide written approval or disapproval of items considered under (1) through (3) above. SNSOC approval shall be certified in writing by an Assistant Station Manager.

2. Render determinations in writing with regard to whether or not each item considered under "(1)" through "(5)" above constitutes an unreviewed safety question.

3. Provide written notification within 24 hours to the Vice President - Nuclear Operations and the Director - Safety Evaluation and Control of disagreement between SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.A above.

3. Changes in the Technical Specifications or license amendments relating to nuclear safety prior to implementation except in those cases where the change is identical to a previously reviewed proposed change.

4. Violations and Reportable Events such as:

   (a) Violations of applicable codes, regulations, order, Technical Specifications, license requirements or internal procedures or instructions having safety significance;

   (b) Significant operating abnormalities or deviations from normal or expected performance of station safety-related structures, systems, or components; and

   (c) All Reportable Events.

   Review of events covered under this paragraph shall include the results of any investigations made and the recommendations resulting from such investigations to prevent or reduce the probability of recurrence of the event.

5. The Quality Assurance audit program at least once per 12 months and audit reports.

6. Any other matter involving safe operation of the nuclear power stations which is referred to the Director - Safety Evaluation and Control.

7. Reports and meeting minutes of the Station Nuclear Safety and Operating Committee.

f. Authority

The Director - Safety Evaluation and Control shall report to and advise the Manager - Nuclear Programs and Licensing, who shall advise the Vice President - Nuclear Operations on those areas of responsibility specified in Section 6.1.C.2.d. ?

g. Records

Records of SEC activities required by Specification 6.1.C2.e shall be prepared and maintained in the SEC files and a summary shall be disseminated each calendar month as follows:

1. Vice President - Nuclear Operations

2. Nuclear Power Station Managers

3. Manager - Nuclear Operations Support

4. Manager - Nuclear Programs and Licensing

5. Executive Manager - Quality Assurance

6. Others that the Director - Safety Evaluation and Control may designate

c. Records

Records of the Quality Assurance Department audits shall be prepared and maintained in the department files. Audit reports shall be disseminated as indicated below:

1. Vice President - Nuclear Operations

2. Nuclear Power Station Manager

3. Manager - Nuclear Operations Support

4. Executive Manager - Quality Assurance

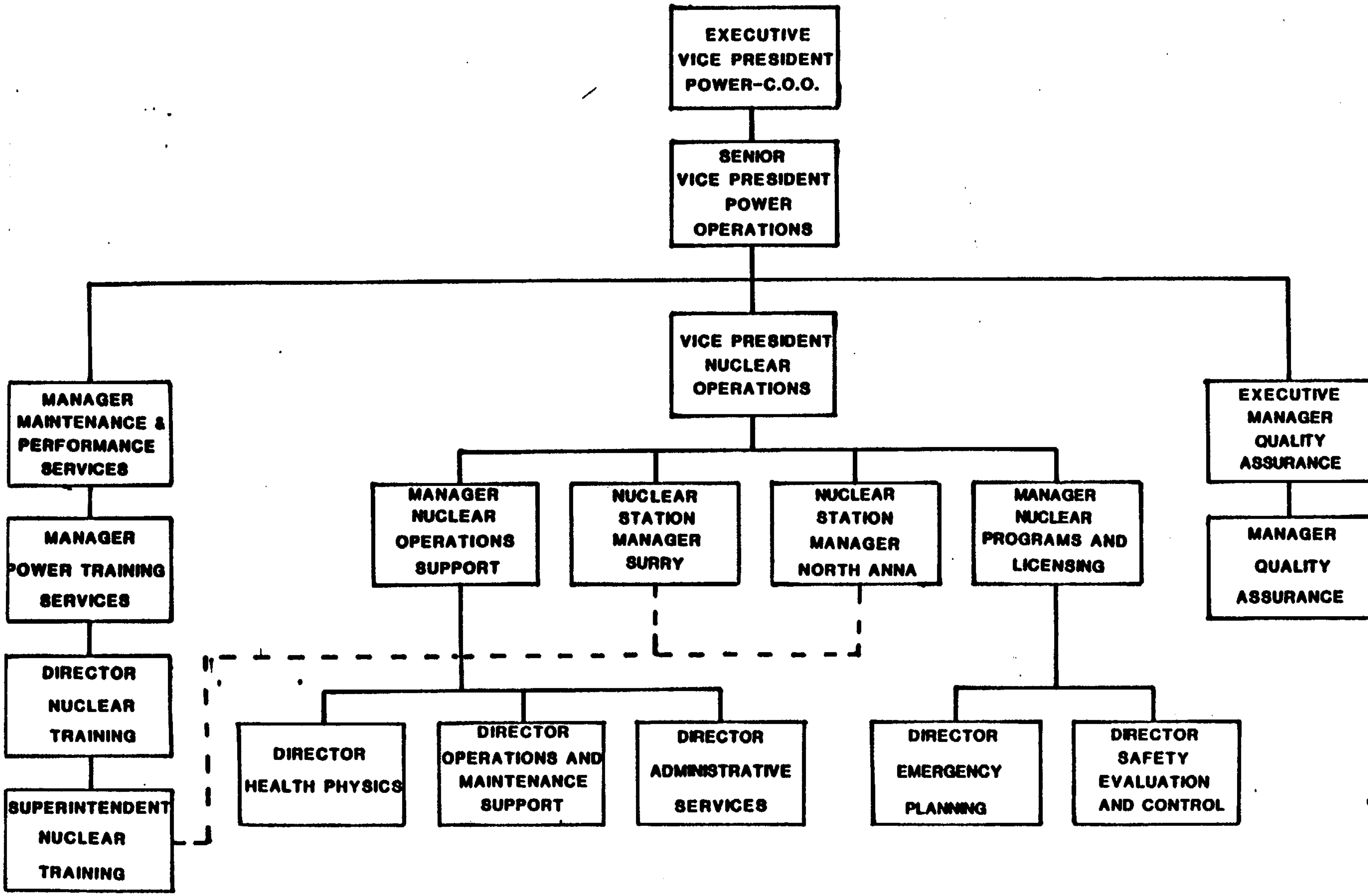
5. Manager - Nuclear Programs and Licensing

6. Director - Safety Evaluation and Control

7. Supervisor of area audited

8. Nuclear Power Station Manager-Quality Assurance

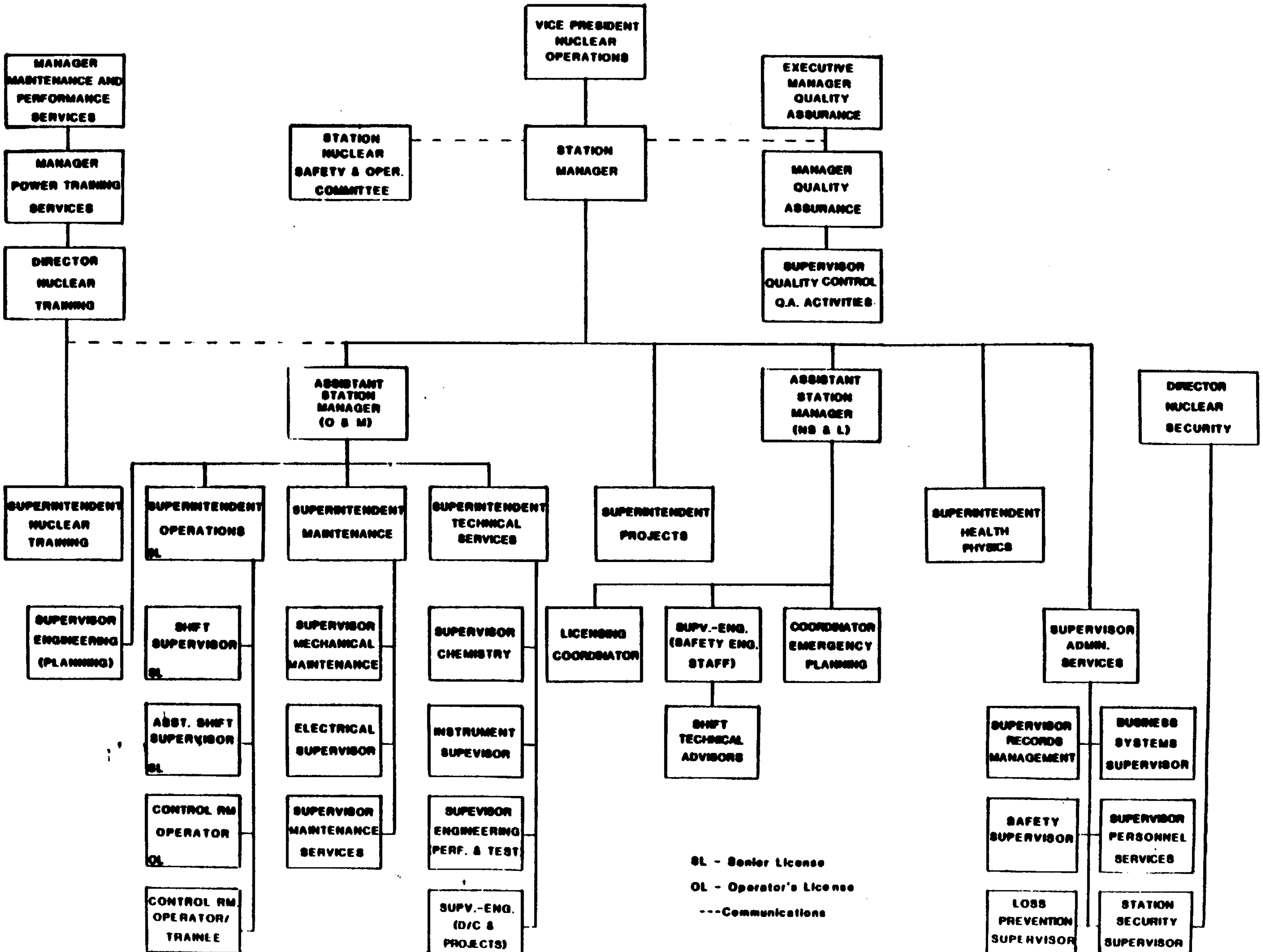
OFF-SITE ORGANIZATION FOR FACILITY MANAGEMENT AND TECHNICAL SUPPORT



T.S. Fig. 6.1-1

Amendment Nos. 104 and 104

SURRY POWER STATION ORGANIZATION CHART



Amendment Nos. 104 and 104

TS Figure 6.1-2

6.2 GENERAL NOTIFICATION AND REPORTING REQUIREMENTS

Specification

A. The following actions shall be taken for Reportable Events:

1. A report shall be submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and

2. Each Reportable Event shall be reviewed by the SNSOC. The Director-Safety Evaluation and Control and Vice-President Nuclear Operations shall be notified of the results of this review.

B. Immediate notifications shall be made in accordance with Section 50.72 to 10 CFR Part 50.

6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

Specification

A. The following actions shall be taken in the event a Safety Limit is violated:

1. The facility shall be placed in at least hot shutdown within 1 hour.

2. The Safety Limit violation shall be reported to the Commission, the Vice President - Nuclear Operations, and the Director - Safety Evaluation and Control within 24 hours.

3. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SNSOC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.

4. The Safety Limit Violation Report shall be submitted to the Commission, the Director - Safety Evaluation and Control, and the Vice President - Nuclear Operations within 14 days of the violation.

1. The intent of 10 CFR 20.203(c)(2)(iii) shall be implemented by satisfying the following conditions:

   a. The entrance to each radiation area in which the intensity of radiation is greater than 100 mrem/hr but less than 1000 mrem/hr shall be barricaded and conspicuously posted.

   b. The entrance to each radiation area in which the intensity of radiation is equal to or greater than 1000 mrem/hr shall be provided with locked barricades to prevent unauthorized entry into these areas. Keys to these locked barricades shall be maintained under the administrative control of the Shift Supervisor on duty and/or Superintendent Health Physics.

   c. All such accessible high radiation areas shall be surveyed by Health Physics personnel on a routine schedule, as determined by the Superintendent-Health Physics, to assure a safe and practical program.

   d. Any individual entering a high radiation area shall have completed the indoctrination course designed to explain the hazards and safety requirements involved, or shall be escorted at all times by a person who has completed the course.

   e. Any individual or group of individuals permitted to enter a high radiation area per 1.d above, shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

f. Entrance to areas with radiation levels in excess of 1 R/hr shall require the use of the "buddy system", whereby a minimum of two individuals maintain continuous visual and/or verbal communication with each other; or other mechanical and/or electrical means to provide constant communication with the individual in the area shall be provided.

g. A Radiation Work Permit system shall be used to authorize and control any work performed in high radiation areas.

h. All buildings or structures, in or around which a high radiation area exists, shall be surrounded by a chain-link fence. The entrance gate shall be locked under administrative control, or continuously guarded to preclude unauthorized entry.

i. Stringent administrative procedures shall be implemented to assure adherence to the restriction placed on the entrance to a high radiation area and the radiation protection program associated thereto.

2. Written procedures shall be established, implemented and maintained covering the activities referenced below:

   a. Process Control Program implementation.

   b. Offsite Dose Calculation Manual implementation.

C. All procedures described in 6.4.A and 6.4.B, and changes thereto, shall be reviewed and approved by the Station Nuclear Safety and Operating Committee prior to implementation.

D. All procedures described in Specifications 6.4.A and 6.4.B shall be followed.

E. Temporary changes to procedures described in Specifications 6.4.A and 6.4.B which do not change the intent of the original procedure may be made, provided such changes are approved prior to implementation by the persons designated below based on the type of procedure to be changed:

1.	Administrative	Cognizant Supervisor
2.	Abnormal	Shift Supervisor or Assistant Shift Supervisor
3.	Annunciator	Shift Supervisor or Assistant Shift Supervisor
4.	Health Physics	*Health Physicist
5.	Emergency	Shift Supervisor or Assistant Shift Supervisor
6.	Maintenance	*Cognizant Supervisor
7.	Operating	Shift Supervisor or Assistant Shift Supervisor
8.	Periodic Test	*Cognizant Supervisor
9.	Start-up Test	*Engineering Supervisor
10.	Special Test	*Engineering Supervisor
11.	Quality Assurance	Manager, Quality Assurance or Supervisor Quality Control
12.	Chemistry	*Chemist

*These procedures must have the approval of a licensed Senior Reactor Operator.

Such changes will be documented and subsequently reviewed and approved by the Station Nuclear Safety and Operating Committee within 14 days.

F. Temporary changes to procedures described in Specifications 6.4.A and 6.4.B which change the intent of the original procedures may be made, provided such changes are approved prior to implementation by the person designated below based on the type of the procedure to be changed.

1.	Administrative	Station Manager
2.	Abnormal	Superintendent - Operations
3.	Annunciator	Superintendent - Operations
4.	Health Physics	Superintendent - Health Physics
5.	Emergency	Superintendent - Operations
6.	Maintenance	Mechanical Supervisor Electrical Supervisor Instrument Supervisor
7.	Operating	Superintendent - Operations
8.	Periodic Test	Engineering Supervisor
9.	Start-up Test	Engineering Supervisor
10.	Special Test	Engineering Supervisor
11.	Quality Assurance	Manager, Quality Assurance or Supervisor
12.	Chemistry	Supervisor - Chemistry

Such changes will be documented and subsequently reviewed and approved by the Station Nuclear Safety and Operating Committee.

G. In cases of emergency, operations personnel shall be authorized to depart from approved procedures where necessary to prevent injury to personnel or damage to the facility. Such changes shall be documented, reviewed and approved by the Station Nuclear Safety and Operating Committee.

6.5 STATION OPERATING RECORDS

Specification

A. Records and logs relative to the following items shall be retained for 5 years, unless a longer period is required by applicable regulations.

1. Records of normal plant operation, including power levels and periods of operation at each power level.

2. Records of principle maintenance activities, including inspection repair, substitution, or replacement of principle items of equipment pertaining to nuclear safety.

3. Record of all Reportable Events.

4. Record of periodic checks, inspections, and calibrations performed to verify that surveillance requirements are being met.

5. Records of any special reactor test or experiments pursuant to 10 CFR 50.59.

6. Records of changes made in the Operating Procedures pursuant to 10 CFR 50.59.

7. Records of shipment of radioactive material.

8. Records of leakage testing of miscellaneous radioactive source test results, in units or microcuires, for leak tests performed pursuant to Technical Specification 4.16.

6.6 STATION REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Administrator of the appropriate NRC Regional Office unless otherwise noted.

A. Routine Reports

1. Startup Report

A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any addditional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commerical power operations), supplementary reports shall be submitted at least every 3 months until all three events have been completed.

2. Annual Operating Report[1/]

Deleted

(1) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, [2/] e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totaling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

3. Monthly Operating Report

Routine reports of operating statistics and shutdown experience, including documentation of all challenges to the Reactor Coolant System PORV's or safety valves, shall be submitted on a monthly basis to the Director, Office of Management and Program Analysis, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office of Inspection and Enforcement, no later than the 15th of each month following the calendar month covered by the report.

Pages 6.6-5 through 6.6-9 have been deleted.

B. Unique Reporting Requirements

1. Inservice Inspection Evaluation

Special summary technical report shall be submitted to the Director of Reactor Licensing, Office of Nuclear Reactor Regulation, NRC, Washington, D.C. 20555, after 5 years of operation. This report shall include an evaluation of the results of the inservice inspection program and will be reviewed in light of the technology available at that time.

2. Annual Radiological Environmental Operating Report.[1]

Routine Radiological Environmental Operating Reports covering the operation of the unit during the previous calendar year shall be submitted prior to May 1 of each year. The initial report shall be submitted prior to May 1 of the year following inital criticality.

The Annual Radiological Environmental Operating Reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, operational controls (as appropriate), and previous environmental surveillance reports, and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land use censuses required by Specification 3.11.D.2.a.

3. Semi-Annual Radioactive Effluent Release Report[1]

Routine Radioactive Effluent Release Reports covering the operation of the unit during the previous 6 months shall be submitted within 60 days after January 1 and July 1 of each year. The period of the first report shall begin with the date of initial criticality.

The Radioactive Effluent Release Reports shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water Cooled Nuclear Power Plants," Revision 1, June 1974, with data summarized on a quarterly basis following the format of Tables 1, 2, and 3 of Appendix B thereof.

The Radioactive Effluent Release Report to be submitted within 60 days after January 1 of each year shall include an assessment of the radiation doses to the maximum exposed members of the public due to the radioactive liquid and gaseous effluents released from the unit or station during the previous calendar year. Annual meteorological data shall be retained in a file on site and shall be made available to the NRC upon request. All assumptions used in making these assessments (i.e., specific activity, exposure time and location) shall be included in the

4. Containment Leak Rate Test

Each containment integrated leak rate test shall be the subject of a summary technical report. Upon completion of the initial containment leak rate test specified by proposed Appendix J to 10 CFR 50, a special report shall, if that Appendix is adopted as an effective rule, be submitted to the Director, Division of Reactor Licensing, USNRC, Washington, D.C. 20555, and other containment leak rate tests specified by Appendix J that fail to meet the acceptance criteria of the appendix, shall be the subject of special summary technical reports pursuant to Section V.B of Appendix J:

a. "Report of Test Results - The initial Type A tests shall be subject of a summary technical report submitted to the Commission approximately 3 months after the conduct of the test. This report shall include a schematic arrangement of the leakage rate measurement system, the instrumentation used, the supplemental test method, and the test program selected as applicable to the initial test, and all subsequent periodic tests. The report shall contain an analysis and interpretation of the leakage rate test data to the extent necessary to demonstrate the acceptability of the containment's leakage rate in meeting the acceptance criteria."

"For periodic tests, leakage rate results of Type A, B, and C tests that meet the acceptance criteria of Sections III.A.7, III.B.3, respectively, shall be reported in the licensee's periodic operating report. Leakage test results of Type A, B, and C tests that fail to meet the acceptance criteria of Sections III.A.7, III.B.3, and III.C.3, respectively, shall be reported in a separate summary report that includes an analysis and interpretation of the test data, the least squares fit analysis of the test data, the instrument error analysis, and the structural conditions of the containment or components, if any, which contributed to the failure in meeting the acceptance criteria. Results and analyses of the supplemental verification test employed to demonstrate the validity of the leakage rate test measurements shall also be included."

C. Special Reports

In the event that the Reactor Vessel Overpressure Mitigating System is used to mitigate a RCS pressure transient, submit a Special Report to the Commission within 30 days. The report shall describe the circumstances initiating the transient, the effect of the PORVs or the administrative controls on the transient and any corrective action necessary to prevent recurrence.

FOOTNOTES

1. A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

2. This tabulation supplements the requirements of K20.407 of 10 CFR Part 20.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NO. DPR-32

AND AMENDMENT NO. 104 TO FACILITY OPERATING LICENSE NO. DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

Introduction

By letters dated March 16, 1982, as supplemented June 28, August 3 and August 9, 1982; June 30 and October 27, 1983; March 22 and November 2, 1984; and April 17 and August 30, 1985, the Virginia Electric and Power Company (the licensee) requested an amendment to Operating Licenses DPR-32 and DPR-37 for Surry Power Station, Unit Nos. 1 and 2, respectively, relating to onsite and offsite organizational changes.

By letter dated November 30, 1984, as supplemented April 12 and August 30, 1985, the licensee, in response to Generic Letter (GL) 83-43, proposed changes to the Surry Power Station Technical Specifications (TS) with respect to notification requirements for operating nuclear power reactors and the licensee event report system. GL 83-43 dated December 19, 1983 informed utilities of changes to Title 10 of the Code of Federal Regulations (10 CFR) with respect to notification requirements for operating nuclear power reactors and requested utilities to revise Technical Specifications to incorporate these changes. These changes involved revision to 10 CFR 50.72 for immediate notification requirements and addition of 10 CFR 50.73 for a revised licensee event report system. Both of these changes became effective January 1, 1984. GL 83-43 provided a model Technical Specification showing the revisions which should be made in the "Administrative Control" and Definitions" sections to incorporate these regulation changes.

Discussion and Evaluation

A. Management Organization Changes

By letters dated March 16, June 28, August 3 and August 9, 1982; June 30 and October 27, 1983; March 22 and November 2, 1984; and April 17 and August 30, 1985, the licensee requested a series of Technical Specification changes associated primarily with the offsite and onsite organizations. The November 2, 1984, letter also proposes to (1) modify certain procedure review requirements and (2) change an ANSI standard reference to make the reference agree with the standard specified in the licensee's Quality Assurance Topical Report.

1. Proposed Reorganization of Nuclear Operations Department (1982)

A reorganization within the Nuclear Operations Department will result in the transfer of responsibilities from the Manager-Nuclear Operations and Maintenance (a position that was subsequently deleted by a reorganization in 1983) to the Vice President-Nuclear Operations. The corporate organizational structure specified in the Technical Specifications previously indicated that the Nuclear Station Manager reported to the Manager-Nuclear Operations and Maintenance, who in turn reported to the Vice President-Nuclear Operations. As a result of the reorganization, the Station Managers will report directly to the Vice President-Nuclear Operations. The Vice President-Nuclear Operations will be responsible for coordinating the technical services activities and the operations/maintenance activities of the nuclear units. The transfer of responsibilities to the Vice President-Nuclear Operations will provide continued assurance of adequate management involvement and enhances management attention to safety activities for the units.

2. Proposed Reorganization of the Quality Assurance Department (1982)

A reorganization within the Quality Assurance Department will result in the Manager, Quality Assurance at the station reporting directly to the Executive Manager, Quality Assurance. The position of Director-Emergency Planning will be created within the Nuclear Operations Department. The changes will be to enhance the quality assurance and emergency preparedness programs of the company.

3. Proposed Reorganization of the Nuclear Operations Department (1983)

A reorganization within the Nuclear Operations Department (NOD) will eliminate the position of Manager, Nuclear Operations and Maintenance. The Technical Analysis and Control group will be disbanded and its functions assumed within the Safety Evaluation and Control staff, NOD, and within the Engineering and Construction Division. The Manager, Nuclear Technical Services will be retitled Manager, Nuclear Operations Support and will assume the responsibilities of the Manager, Nuclear Operations and Maintenance. The new Fuel Operations group will also report to him.

Several other title and reporting changes will take place: The title of Executive Vice President will be changed to Executive Vice President and Chief Operating Officer. The Director, Administrative Services will be realigned to report directly to the Vice President-Nuclear Operations. (This was subsequently changed by a 1984 reorganization wherein the Director, Administrative Services was realigned to report to the Manager, Nuclear Operations Support.)

A reorganization within the Security Department will also take place. Although the Station Security Supervisor will continue to have communications with the Supervisor, Administrative Services at the station, he will report to the Corporate Director, Nuclear Security.

4. Proposed Reorganization of Training Activities (1983)

A reorganization within the Nuclear Operations Department and the Performance Services Department will aid the quality of training activities at the power stations. Although the Superintendent, Nuclear Training will continue to have communications with the Station Manager, he will report to the Director, Nuclear Training in the Maintenance and Performance Services Department. The Director, Nuclear Training will report to the Manager, Power Training Services, who in turn, will report to the Manager, Maintenance and Performance Services, who ultimately will report to the Senior Vice President-Power Operations. The consolidation of training activities will assure that effective and efficient technical training is provided to Nuclear Operations personnel.

5. Proposed Administrative Change in Onsite Organization (1984)

Trends within the utility "loss-prevention/fire-protection" field indicate the continued importance of property protection and the individuals charged with property protection. As a result, the title "Fire Marshall" will be changed to "Loss Prevention Supervisor." The new title implies a stronger business/professional role and more appropriately reflects the incumbent's rank and responsibility.

6. Proposed Reorganization of the Nuclear Operations Department (1984)

A reorganization within the Nuclear Operations Department will create two new management level positions: "Manager-Nuclear Programs and Licensing" and "Assistant Station Manager (Nuclear Safety and Licensing)." The Manager-Nuclear Programs and Licensing will assume certain authorities and responsibilities previously held by the Manager-Nuclear Operations Support in the areas of emergency preparedness, licensing, and independent safety reviews. The new Assistant Station Manager (Nuclear Safety and Licensing) will assume certain authorities and responsibilities previously held by both the Station Manager and the Assistant Station Manager (Operations and Maintenance) regarding the operation of the Station Nuclear Safety and Operating Committee (SNSOC), licensing, safety engineering, and emergency planning. The change will not create any new authorities or responsibilities within the Nuclear Operations Department; rather, by reducing the span of control of the affected managers (both new and existing), management control and effectiveness in the areas of concern will be enhanced. Thus, more management attention will be focused on significant issues.

The authority of the new Manager-Nuclear Programs and Licensing will be identified; and will be is designated to receive records of Safety Evaluation and Control activities and Quality Assurance audits.

The new Assistant Station Manager (Nuclear Safety and Licensing) will be designated as Chairman, Station Nuclear Safety and Operating Committee, replacing the Station Manager. The existing Assistant Station Manager will be redesignated as Assistant Station Manager (Operations and Maintenance).

The responsibilities and authorities of the SNSOC will be clarified. The proposed changes will allow the SNSOC to recommend Technical Specification changes to the Station Manager instead of just reviewing proposed changes. Also, the Chairman SNSOC reviews and approves recommended changes to the Plant Security Plan and implementing procedures and Emergency Plan and implementing procedures; under the new (proposed) arrangement, the SNSOC would review changes to the Plant Security Plan (and implementing procedures) and Emergency Plan (and implementing procedures) and submit recommended changes to the Station Manager.

7. Industry Standard on Qualifications and Training

The proposed change in the referenced standard on facility staff qualifications and training in Technical Specification Section 6.1 reflects the ANS standard specified in Vepco's QA Topical Report, "Quality Assurance Program Operations Phase," Amendment 4, and reflects Vepco's position on NRC Regulatory Guide 1.8-"Personnel Qualifications and Training." The QA Topical Report was approved by the NRC on October 6, 1982. Thus, the change would amend the Technical Specifications to make them consistent with the NRC-approved Topical Report. The specific change replaces ANSI N18.1-1971 with ANSI/ANS 3.1-(Draft 12/79). ANS 3.1-(Draft 12/79) meets or exceeds the requirements of the older ANSI N18.1-1971 and is acceptable to the staff.

8. Proposed Adminstrative Change in Onsite Organization (1985)

Trends within the utility "Health Physics" area indicate the continued importance of health and radiation protection and the individuals charged with these responsibilities. As a result, the title "Supervisor - Health Physics" will be changed to "Superintendent - Health Physics." The new title implies a stronger managerial and professional role and more appropriately reflects the incumbents rank and responsibility.

We have reviewed the proposed changes and find them acceptable as they meet the acceptance criteria of Sections 13.1.1, 13.1.2, and 13.4 of NUREG-0800, the Standard Review Plan. Technical Specification Figures 6.1-1 and 6.1-2, as revised, show the reorganized offsite and onsite management organizations.

B. Reporting Requirements to Conform to 10 CFR 50.72 and 50.73

Generic Letter No. 83-43 requested all licensees to revise their Technical Specifications to comply with 10 CFR 50.72 and 50.73. 10 CFR 50.72 has been revised to indicate the immediate notification requirements for operating nuclear power reactors. 10 CFR 50.73 is new and provides for a revised Licensee Event Report (LER) System. To comply with the new rules, the following changes to the Surry 1 and 2 Technical Specifications have been requested by letters dated November 30, 1984, and April 12 and August 30, 1985:

1. Throughout the Technical Specifications, revise the term "Reportable Occurrence" to "Reportable Events."

2. Change the definition of "Reportable Event" to read, "a Reportable Event shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50."

3. Delete the "Reportable Occurrences" section, Technical Specifications 6.6.2, 6.6.2.a, and 6.6.2.b, and revise the references to this section throughout the Technical Specifications to reflect the renumbering of Section 6 Specifications.

4. Insert where applicable, the reference to Section 50.73 to 10 CFR Part 50.

5. Renumber Technical Specification 6.6.a to follow the outline format of the other sections of the Surry Technical Specifications.

6. Throughout the Technical Specifications revise the references to Technical Specification 6.6.

7. Revise the NRC recipient of certain reports to reflect the current NRC organization, i.e., Regional Administrator of an NRC Regional Office.

8. Delete Technical Specification Section 6.6.3.f regarding the submittal of the initial containment structural test special summary report.

9. Correction of minor editorial, format and typographical errors.

We have reviewed the proposed changes and find them acceptable as they are consistent with the guidance and intent of GL 83-43.

Environmental Consideration

These amendments involve changes in reporting and administrative procedures and requirements. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statementor environmental assessment need be prepared in connection with the issuance of these amendments.

The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: December 11, 1985

Principal Contributors:

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