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Docket Nos. 50-280
and 50-281

Virginia Electric & Power Company
ATTN: Mr. Stanley Ragone
Senior Vice President
Post Office Box 26666
Richmond, Virginia 23261

NOV 21 1975

Gentlemen:

The Commission has issued the enclosed Amendments No. 12 to Facility Operating Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station, Units 1 and 2. The amendments include Change No. 27 to the Technical Specifications for each license and are in response to your request dated October 22, 1975.

The amendments revise the provisions in the Technical Specifications relating to the heatup and cooldown limitations of the reactor coolant system.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

RS Rooney
for

Robert W. Reid, Chief
Operating Reactors Branch 4
Division of Reactor Licensing

Enclosures:

1. Amendment No. 12 to DPR-32
2. Amendment No. 12 to DPR-37
3. Safety Evaluation
4. Federal Register Notice

cc: See next page

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DATE →	11/21/75	11/21/75	11/21/75	11/21/75	11/21/75

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NO. 12 TO LICENSES NOS. DPR-32 AND DPR-37

CHANGE NO. 27 TO TECHNICAL SPECIFICATIONS

VIRGINIA ELECTRIC & POWER COMPANY

SURRY POWER STATION UNITS 1 & 2

DOCKETS NOS. 50-280 AND 50-281

Introduction

By letter dated October 22, 1975, Virginia Electric & Power Company (the licensee) requested changes to the Technical Specifications appended to Facility Operating Licenses Nos. DPR-32 and DPR-37 for the Surry Power Station Units 1 and 2. The purpose of the request is to revise the Surry 1 and 2 Technical Specifications relating to the heatup and cool-down limitations of the reactor coolant system.

Discussion

The proposed pressure-temperature limits for operation of Surry Units 1 and 2 are based on the requirements of 10 CFR 50-Appendix G, NRC Standard Review Plan 5.3.2, "Pressure and Temperature Limits" and data from tests on the material surveillance specimens withdrawn from their vessels. The limiting material for the proposed pressure-temperature limit curves is weld material from the Unit 1 reactor vessel. This material contains 0.25% copper. Surveillance specimens of this material, subjected to a fluence of 2.5×10^{18} nvt, showed an increase in RT_{NDT} of 250 F. The proposed limit curves are for 3.8 effective full power years (EFPY), at which time the fluence is calculated to be 3.58×10 nvt at 1/4 T location and the resulting shift in RT_{NDT} is calculated to be 280 F.

Evaluation

We have evaluated the pressure-temperature limit curve presented in Figure 3.1-1 of the licensee's proposed Technical Specification Change of October 22, 1975 and conclude that it is acceptable for heatup and cooldown pressure-temperature limits. We have augmented this curve to indicate pressure-temperature limits for inservice hydro testing and core criticality. The attached curves provide acceptable limits for these conditions.

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The following paragraph has been added to Specification 3.1.B.1:

"Core Operation:

During operation where the reactor core is in a critical condition (except for low level physics tests), vessel metal and fluid temperature shall be maintained above the reactor core criticality limits specified in Figure 3.1-1."

The following statement has been added to Specification 4.3.A:

"The pressure-temperature limits for these tests will be in accordance with Figure 3.1-1."

The pressure-temperature limits for inservice tests, heatup and cooldown and core operation contained in the licensee's Technical Specification Change of October 22, 1975 with the above revisions comply with the requirements of Appendix G, 10 CFR Part 50 and are acceptable. The use of Appendix G in establishing safe operating limitations will ensure adequate safety margins during operation, testing, maintenance and postulated accident conditions and constitute an acceptable basis for satisfying the requirements of NRC General Design Criterion 31, Appendix A, 10 CFR Part 50.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

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