

Docket Nos. 50-280  
and 50-281

3-28-78

Virginia Electric & Power Company  
ATTN: Mr. W. L. Proffitt  
Senior Vice President - Power  
P. O. Box 26666  
Richmond, Virginia 23261

Gentlemen:

The Commission has issued the enclosed Amendment Nos. 38 and 37 to Facility Operating License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated March 17, 1978 as supplemented on March 21, 1978.

The amendments revise Technical Specification 4.7, Main Steam Line Trip Valves, periodic testing frequency to be consistent with 10 CFR 50.55a(g) requirements and to be consistent with the standard Technical Specifications for Westinghouse Pressurized Water Reactors dated May 15, 1976.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

131

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:

1. Amendment No. 38 to DPR-32
2. Amendment No. 37 to DPR-37
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:  
See next page

DISTRIBUTION  
Dockets(2)  
NRC PDR(2)  
Local PDR  
ORB#1 Reading  
VStello  
AD/OR  
CParrish  
DNeighbors  
OELD  
OI&E(5)  
BJones(8)  
BScharf(10)  
BHarless  
DEisenhut  
ACRS(16)  
CMiles  
TBAbernathy  
JRBuchanan

OFFICE	ORB#1	OELD	ORB#1	EB	PSB
SURNAME	DNeighbors	KARL	ASchwencer	SHAO	BUTLER
DATE	3/27/78	3/28/78	3/28/78	12/78	3/27/78
NRC					



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

Docket Nos. 50-280  
and 50-281

March 28, 1978

Virginia Electric & Power Company  
ATTN: Mr. W. L. Profitt  
Senior Vice President - Power  
P. O. Box 26666  
Richmond, Virginia 23261

Gentlemen:

The Commission has issued the enclosed Amendment Nos. 38 and 37 to Facility Operating License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated March 17, 1978 as supplemented on March 21, 1978.

The amendments revise Technical Specification 4.7, Main Steam Line Trip Valves, periodic testing frequency to be consistent with 10 CFR 50.55a(g) requirements and to be consistent with the standard Technical Specifications for Westinghouse Pressurized Water Reactors dated May 15, 1976.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A handwritten signature in cursive script, appearing to read "A. Schwencer", is written over the typed name.

A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Enclosures:

1. Amendment No. 38 to DPR-32
2. Amendment No. 37 to DPR-37
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:  
See next page

March 28, 1978

cc: Mr. Michael W. Maupin  
Hunton & Williams  
Post Office Box 1535  
Richmond, Virginia 23213

Swem Library  
College of William & Mary  
Williamsburg, Virginia 23185

Mr. Sherlock Holmes, Chairman  
Board of Supervisors of Surry  
County  
Surry County Courthouse, Virginia 23683

Commonwealth of Virginia  
Council on the Environment  
903 Ninth Street Office Building  
Richmond, Virginia 23219

Mr. James R. Wittine  
Commonwealth of Virginia  
State Corporation Commission  
Post Office Box 1197  
Richmond, Virginia 23209

Chief, Energy Systems  
Analyses Branch (AW-459)  
Office of Radiation Programs  
U.S. Environmental Protection Agency  
Room 645, East Tower  
401 M Street, SW  
Washington, D.C. 20460

U.S. Environmental Protection Agency  
Region III Office  
ATTN: EIS COORDINATOR  
Curtis Building - 6th Floor  
6th and Walnut Streets  
Philadelphia, Pennsylvania 19106



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC & POWER COMPANY

DOCKET NO. 50-280

SURRY POWER STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 38  
License No. DPR-32

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric & Power Company (the licensee) dated March 17, 1978 as supplemented by letter dated March 21, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

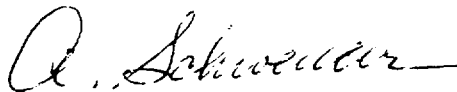
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-32 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 38, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 28, 1978

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. DPR-32

AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. DPR-37

DOCKET NOS. 50-280 AND 50-281

Revise Appendix A Technical Specifications as follows:

Remove the following pages and replace with identically  
numbered pages.

4.1-9  
4.7-1

TABLE 4.1-2A

## MINIMUM FREQUENCY FOR ELEMENT TESTS

<u>Description</u>	<u>Test</u>	<u>Frequency</u>	<u>FSAR Section Reference</u>
1. Control Rod Assemblies	Rod drop times of all full length rods at hot and cold conditions.	Each refueling shutdown or after disassembly or maintenance requiring the breach of the Reactor Coolant System integrity	7
2. Control Rod Assemblies	Partial movement of all rods	Every 2 weeks	7
3. Refueling Water Chemical Addition Tank	Functional	Each refueling shutdown	6
4. Pressurizer Safety Valves	Setpoint	Each refueling shutdown	4
5. Main Steam Safety Valves	Setpoint	Each refueling shutdown	10
6. Containment Isolation Trip	*Functional	Each refueling shutdown	5
7. Refueling System Interlocks	*Functional	Prior to refueling	9.12
8. Service Water System	*Functional	Each refueling shutdown	9.9
9. Fire Protection Pump and Power Supply	Functional	Monthly	9.10
10. Primary System Leakage	*Evaluate	Daily	4
11. Diesel Fuel Supply	*Fuel Inventory	5 days/week	8.5
12. Boric Acid Piping Heat Tracing Circuits	*Operational	Monthly	9.1
13. Main Steam Line Trip	Functional (1) Full closure (2) Partial closure	(1) Each cold shutdown (2) Before each startup	10
14. Service Water System Values in Line Supplying Recirculation Spray Heat Exchangers	Functional	Each refueling	9.9
15. Control Room Ventilation System	*Ability to maintain positive pressure for 1 hour using a volume of air equivalent to or less than that stored in the bottled air supply	Each refueling interval (approx. every 12-18 months)	9.13

\*See Specification 4.1.D

#### 4.7 MAIN STEAM LINE TRIP VALVES

##### Applicability

Applies to periodic testing of the main steam line trip valves.

##### Objective

To verify the ability of the main steam line trip valves to close upon signal.

##### Specification

###### A. Tests and Frequencies

1. Each main steam trip valve shall be tested for full closure each time the reactor is in the cold shutdown condition.
2. Each main steam trip valve shall be inservice tested for partial closure before each startup .

###### B. Acceptance Criteria

1. A full closure test of main steam trip valves shall be considered satisfactory if the valve closes fully in 5 sec or less.





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC & POWER COMPANY

DOCKET NO. 50-281

SURRY POWER STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37  
License No. DPR-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Virginia Electric & Power Company (the licensee) dated March 17, 1978 as supplemented by letter dated March 21, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 37, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief  
Operating Reactors Branch #1  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: March 28, 1978



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENTS NOS. 38 AND 37 TO

FACILITY OPERATING LICENSES NOS. DPR-32 AND DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY

SURRY POWER STATION UNIT NOS. 1 AND 2

DOCKET NOS. 50-280 AND 50-281

Introduction

By letters dated March 17 and 21, 1978, Virginia Electric and Power Company (the licensee) requested a change to the Technical Specifications appended to Facility Operating License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2. The requested change would require testing of the main steam trip valves (MSTV) in accordance with 10 CFR 50.55a.

Discussion

The main steam trip valves are swing disc type trip valves which would close on an excess flow signal resulting from a steam line rupture downstream of the valves. The existing Technical Specifications require that these valves be inservice tested for partial closure before each startup but at least quarterly.

It is possible to test these valves at power. However, past testing at power resulted, in some instances, in the valves slamming shut and causing the plant to be shutdown and causing damage to the valves. Thus when a valve slams shut, the plant must be brought to cold shutdown for valve internal inspection, which takes about a week. The licensee, as a result of the incidents at power, now tests the valves in the hot shutdown condition which takes about one day of outage time.

At the present time, Surry Unit No. 2 is shutdown for steam generator tube inspection and Surry Unit No. 1 is scheduled to be shutdown on April 24, 1978 for refueling. The next valve inspection is due on March 28, 1978 and the plant must be shutdown unless some type of relief is given.

The licensee has proposed to change the main steam trip valve testing requirements to be consistent with the testing practices of Section XI of the ASME Code.

#### Evaluation

The existing periodic testing requirements for the MSTV's are more conservative than the testing requirements of 10 CFR 50.55a. By letter dated November 17, 1976, the licensee was provided guidance on compliance with the provisions of 10 CFR 50.55a which use as a reference Section XI of the ASME Boiler and Pressure Vessel Code. The guidance in Section XI is that valves of this type should be tested quarterly unless it is not practical to test them while at power. Valves not tested each quarter would be tested before the next startup. Using this guidance, considering the satisfactory MSTV performance during periodic testing and having no technical basis to require additional periodic testing, the licensee requested a relaxation of the surveillance requirements of MSTV to that which is required by 10 CFR 50.55a. We have reviewed the technical basis for the current MSTV surveillance requirements and have determined that there is no technical justification at this time, to require additional testing beyond that required by 10 CFR 50.55a.

The change proposed to Section 4.7 of the Surry Technical Specifications is in compliance with 10 CFR 50.55a, is consistent with the provisions of the NRC Westinghouse Standard Technical Specifications and, therefore, is acceptable.

#### Environmental Consideration

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 28, 1978

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-280 AND 50-281

VIRGINIA ELECTRIC AND POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 38 and 37 to Facility Operating License Nos. DPR-32 and DPR-37, issued to Virginia Electric and Power Company (the licensee), which revised Technical Specifications for operation of the Surry Power Station, Unit Nos. 1 and 2 (the facilities) located in Surry County, Virginia. The amendments are effective as of the date of issuance.

The amendments revise Technical Specification 4.7, Main Steam Line Trip valves, periodic testing frequency to be consistent with 10 CFR 50.55a(g) requirements and to be consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors dated May 15, 1976.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 17, 1978, as supplemented by letter dated March 21, 1978, (2) Amendment Nos. 38 and 37 to Facility License Nos. DPR-32 and DPR-37, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW, Washington, DC and at the Swem Library, College of William and Mary, Williamsburg, Virginia. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 28th day of March 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read 'A. Schwencer'.

A. Schwencer, Chief  
Operating Reactors Branch#1  
Division of Operating Reactors