

September 5, 2001

Mr. A. Alan Blind
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING STEAM
GENERATOR SURVEILLANCE REQUIREMENTS, INDIAN POINT NUCLEAR
GENERATING UNIT NO. 2 (TAC NO. MB0770)

Dear Mr. Blind:

In a letter dated December 11, 2000, Consolidated Edison Company of New York, Inc. (Con Edison) submitted a proposed amendment to revise the Technical Specifications (TSs) associated with the primary to secondary leakage limits and the steam generator tube inservice surveillance requirements at Indian Point Nuclear Generating Unit No. 2 (IP2). The TS changes support the inspection and operation of the steam generators that were replaced before restart in December 2000.

The NRC staff has reviewed the information the licensee provided and determined that additional information is required in order to complete its review. The specific information that is needed is in the enclosed request for additional information. In a telephone conversation with representatives of your staff on August 24, 2000, the Nuclear Regulatory Commission (NRC) obtained agreement that Con Edison would provide the additional information requested within 60 days from the date of this letter.

If you have any questions regarding this matter, please contact me at (301) 415-1457.

Sincerely,

/RA/

Patrick D. Milano, Senior Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosure: Request for Additional Information

cc w/encl: See next page

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E. Adensam
Acting SC

ESullivan
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OGC
ACRS

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*See previous concurrence

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| OFFICE | PM:PDI-1* | E | LA:PDI-1 | | SC:EMCB* | | (A)SC:PDI-1 | |
| NAME | PMilano | | TCClark for SLittle | | ESullivan | | PTam | |
| DATE | 09/05/01 | | 09/5 /01 | | 09/05/01 | | 09/ 5 /01 | |

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Indian Point Nuclear Generating Station
Units 1 & 2

Mayor, Village of Buchanan
236 Tate Avenue
Buchanan, NY 10511

Mr. William M. Flynn, President
New York State Energy, Research,
and Development Authority
Corporate Plaza West
286 Washington Ave. Extension
Albany, NY 12203-6399

Mr. John McCann
Manager of Nuclear Safety and
Licensing
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P.O. Box 38
Buchanan, NY 10511

Mr. Brent L. Brandenburg
Assistant General Counsel
Consolidated Edison Company
of New York, Inc.
4 Irving Place - 1822
New York, NY 10003

David Lochbaum
Nuclear Safety Engineer
Union of Concerned Scientists
1707 H Street, NW., Suite 600
Washington, DC 20006

Edward Smeloff
Pace University School of Law
The Energy Project
78 North Broadway
White Plains, NY 10603

Charles Donaldson, Esquire
Assistant Attorney General
New York Department of Law
120 Broadway
New York, NY 10271

Ms. Charlene D. Faison, Director
Nuclear Licensing
Power Authority of the State
of New York
123 Main Street
White Plains, NY 10601

Mr. Thomas Rose
Secretary - NFSC
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

Regional Administrator, Region I
U. S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. Paul Eddy
New York State Department of
Public Service
3 Empire State Plaza, 10th Floor
Albany, NY 12223

Michael Mariotte
Nuclear Information & Resources Service
1424 16th Street, NW, Suite 404
Washington, DC 20036

Deborah Katz
Executive Director
Citizens Awareness Network
P.O. Box 83
Shelburne Falls, MA 01370

Indian Point Nuclear Generating Station
Units 1 & 2

Marilyn Elie
Organizer
Citizens Awareness Network
2A Adrain Court
Cortlandt Manor, NY 10567

Tim Judson
Organizer
Citizens Awareness Network
140 Bassett Street
Syracuse, NY 13213

Kyle Rabin
Environmental Advocates
353 Hamilton Street
Albany, NY 12210

Mark Jacobs
Executive Director
Westchester Peoples Action Coalition
255 Dr. M.L. King Jr. Boulevard
White Plains, NY 10601

Paul Gunter
Nuclear Information & Resource Service
1424 16th Street, NW, #404
Washington, DC 20036

Mark J. Wetterhahn, Esq.
Winston & Strawn
1400 L Street, N.W.
Washington, D.C. 20005

Douglas E. Levanway, Esq.
Wise Carter Child & Caraway
P.O. Box 651
Jackson, Mississippi 39205

Jay E. Silberg, Esq.
Shaw Pittman
2300 N Street, N.W.
Washington, DC 20037

Request for Additional Information
Regarding Proposed Amendment to Technical Specifications
on Steam Generator Inspection Requirements
Consolidated Edison Company of New York, Inc.
Indian Point Nuclear Generating Unit No. 2
Docket No. 50-247

In a letter dated December 11, 2000, the licensee requested changes to the Technical Specifications (TSs) associated with the primary to secondary leakage limits and the steam generator tube inservice surveillance requirements. Specifically, the licensee has proposed changes that would lower the primary to secondary leakage limit and remove the provisions that do not apply to the replacement steam generators such as tube repair by sleeving, steam generator tube denting, and the F* alternate repair criteria. The Nuclear Regulatory Commission (NRC) staff has reviewed the information the licensee provided that supports the proposed TS changes. In order for the staff to complete its review, the staff requires the following information:

1. The Basis section of TS 4.13 states that steam generator tube burst and collapse tests demonstrated that tubes having wall thickness of not less than 0.025 inch have adequate margins of safety. In the December 11 letter, the licensee did not indicate what guidelines, if any, were used to obtain these results. Provide a summary of the analysis, including the loads considered, tube support plate conditions (locked or unlocked), and a list of any guidelines (e.g., NRC Regulatory Guide 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes"), used to support the proposed minimum wall thickness and plugging criteria.
2. The Basis section of TS 4.13 states that an essentially 100% tube examination was performed on each tube in each steam generator prior to service. Although the licensee indicated in the December 11 letter that no baseline imperfections were identified, it did not state what guidelines, if any, were used to conclude that the tubes were free of imperfections. In addition, the licensee did not indicate what guidelines would be used for the first inservice inspection (ISI). Provide a reference to the guidelines used for the preservice inspection and the first ISI. If the licensee intends to depart from any part of the guidelines, provide a summary of those differences.
3. Provide a summary of the assessment completed to confirm that the issues covered by NRC Bulletin 88-02, "Rapidly Propagating Fatigue Cracks in Steam Generator Tubes," are not applicable to the replacement steam generators.

Enclosure