January 14, 1997

Mr. C. K. McCoy Vice President - Nuc r Vogtle Project Georgia Power Company P. O. Box 1295 Birmingham, AL 35201

SUBJECT: CORRECTIONS TO AMENDMENT NOS. 96 AND 74 - VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 (TAC NOS. M92131 AND M92132)

Dear Mr. McCoy:

The Nuclear Regulatory Commission issued Amendment No. 96 to Facility Operating License NPF-68 and Amendment No. 74 to Facility Operating License NPF-81 for the Vogtle Electric Generating Plant, Units 1 and 2, on September 25, 1996. The amendments replace the current Technical Specifications (TS) and associated Bases with a set based on Revision 1 to NUREG-1431, "Standard Technical Specifications Westinghouse Plants."

The special provision for the next performance of Surveillance Requirement 3.8.1.9 that was included in your license amendment application was properly described in the license amendment page for Amendment 74 to NPF-81, but was not properly described in license page 3a that was included in the September 25, 1996, amendment issuance. Please replace NPF-81 page 3a that was issued on September 25, 1996, with the enclosed page 3a.

Also, an error has been discovered in Item 16.e. of TS Table 3.3.1-1 (Reactor Trip System Instrumentation, Power Range Neutron Flux). Specifically, the proposed TS attached to the application did not properly incorporate the provisions of the current licensing basis that had been specified elsewhere in the application. The staff safety evaluation was written to support the adoption of the current licensing basis for this item. Please replace TS page 3.3-18 issued on September 25, 1996, with the enclosed page 3.3-18.

Sincerely, Original signed by: Louis L. Wheeler, Senior Project Manager Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

Enclosures: As stated

cc w/encl: See next page

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

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Mr. C. K. McCoy Georgia Power Company

cc: Mr. J. A. Bailey Manager - Licensing Georgia Power Company P. O. Box 1295 Birmingham, Alabama 35201

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Generating Plant
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Waynesboro, Georgia 30830

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Resident Inspector U.S. Nuclear Regulatory Commission 8805 River Road Waynesboro, Georgia 30830

(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, and the Environmental Protection Plan contained in Appendix B, both of which are attached herein, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The Surveillance Requirements (SRs) contained in the Appendix A Technical Specifications and listed below are not required to be performed immediately upon implementation of Amendment No. 74. The SRs listed below shall be successfully demonstrated prior to the time and condition specified below for each:

- a) SRs 3.8.1.8, 3.8.1.11 and 3.8.1.13 shall be successfully demonstrated prior to the first entry into MODE 4 following the fifth refueling outage.
- b) SR 3.8.1.9 shall be successfully demonstrated prior to the first entry into MODE 4 following the sixth refueling outage.
- c) SR 3.8.1.20 shall be successfully demonstrated at the first regularly scheduled performance after implementation of this license amendment.

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		PUNCTION	APPLICABLE NODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SLEVEILLANCE REQUIREMENTS	ALLONABLE VALUE	TRIP SETPOINT
14.	Turi	bine Trip						
	۹.	Low Fluid Oil Pressure	1())	3	0	SR 3.3.1.10 SR 3.3.1.16	≥ 500 psig	2 580 paig
	b.	Turbine Stop Valve Closure	1(1)	4	P	SR 3.3.1.10 SR 3.3.1.14	2 96.7% open	2 96.7% open
15,	Saf Inj Inp Eng Fes Sys	ety ection (SI) ut from incered Safety ture Actuation tem (ESFAS)	1,2	2 trains	e	SR 3.3.1.13	MA	NA -
16.	Ree Sys	ctor Trip tem interlocks						
	8.	Intermediata Range Neutron Flux, P-6	2 ^(d)	2	R	SR 3.3.1.11 SR 3.3.1.12	≵ 6E-11 smp	≥ 1E-10 amp
	b.	Low Power Reactor Trips Block, P-7	1	i per trein	\$	st 3.3.1.5	NA	NA
	ç,	Power Range Neutron Flux, P-8	1	4	\$	st 3.3.1.11 St 3.3,1.12	≤ 50.3% RTP	≤ 48X RTP
	d.	Power Range Neutron Flux, P-9	۱	4	\$	SR 3.3.1.11 SR 3.3.1.12	\$ 52 .3% RTP	≤ 50% RTP
	€.	Power Range Neutron Flux, P-10 and input	1,2	. 6	R	SR 3.3.1.11 SR 3.3.1.12	(L,m)	(l,m)
	f.	to P-7 Turbine Impulse Pressure, P-13	۱	2	8	sa 3.3.1.10 sa 3.3.1.12	≤ 12.3% Impulse Pressure Equivalent turbine	≤ 10% Impulse Pressure Equivalent turbine

Table 3.3.1-1 (page 5 of 8) Reactor Trip System Instrumentation

(continued)

(d) Selow the P-6 (Intermediate Range Neutron Flux) interlocks.

(j) Above the P-9 (Power Range Neutron Flux) interlook.

- (1) For the P-10 input to P-7, the Allouable Value is \leq 12.3% RTP and the Trip Setpoint is \leq 10% RTP.
- (m) For the Power Range Neutron Flux, P-10, the Allowable Value is ≥ 7.7% RTP and the Trip Setpoint is ≥ 10% RTP.

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