

Appendix A to
Provisional Operating License DPR-5
for
Entergy Nuclear Indian Point 2, LLC
and Entergy Nuclear Operations, Inc.

Indian Point Station

Unit No. 1

Docket No. 50-3

Amendment No.

Appendix A to
Provisional Operating License DPR-5

For the

Entergy Nuclear Indian Point 2, LLC
and Entergy Nuclear Operations, Inc.

1.0 GENERAL INFORMATION

The facility, known as the Indian Point Station Unit No. 1, is located on the 235 acre site in the Village of Buchanan, Westchester County, New York. The Indian Point Station Unit No. 2 and the Indian Point Station Unit No. 3 share this site.

Indian Point Unit No. 1 includes a pressurized water reactor which operated with an authorized maximum steady state power level of 615 thermal megawatts until October 31, 1974. Pursuant to a June 19, 1980 Commission Order Revoking Authority to Operate Facility and a Decommissioning Plan for Indian Point Unit No. 1 submitted by Con Edison to NRC on October 17, 1980 in accordance with that Order, the reactor remains in a defueled status and the unit continues to operate as a support facility for overall Indian Point Units 1 and 2 site operations. Unit No. 1 and Unit No. 2 are physically contiguous and share a number of systems and facilities as well as a common operating organization. The technical specifications contained herein recognize this commonality as well as the intended use of the Unit No. 1 facilities to support Unit No. 2 until retirement of that unit, and contain specific references to Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26. Unit No. 1 contains radioactive waste processing facilities which provide waste processing services for both Unit No. 1 and Unit No. 2. Radiological effluent limits are met on an overall site basis and specific operating limits and surveillance requirements for effluent monitoring instrumentation, including stack noble gas monitoring, are discussed in Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

1.1 Definitions

1.1.1 Operable-Operability

A system, subsystem, train, component or device shall be operable or have operability when it is capable of performing its intended safety function(s). Implicit in this definition shall be the assumption that necessary instrumentation, controls, electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its safety function(s) are also capable of performing their related support functions.

1.1.2 Member(s) of the Public

Member(s) of the Public includes all persons who are not occupationally associated with the site. This category does not include employees of either Entergy Nuclear Indian Point 2, LLC (ENIP2), Entergy Nuclear Operations, Inc. (ENO), or other site licensee, their contractors or vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries.

1.1.3 Offsite Dose Calculation Manual (ODCM)

The Offsite Dose Calculation Manual contains the current methodology and parameters used in the calculation of offsite doses due to radioactive gaseous and liquid effluents, in the calculation of gaseous and liquid effluent monitoring alarm/trip setpoints, and in the conduct of the environmental radiological monitoring program.

1.1.4 Process Control Program (PCP)

The Process Control Program is a manual containing and/or referencing selected operational information concerning the solidification of radioactive wastes from liquid systems.

1.1.5 Site Boundary

The Site Boundary is that line beyond which the land is neither owned, leased, nor otherwise controlled by either ENIP2, ENO, or other site licensee.

1.1.6 Solidification

Solidification is the conversion of wet wastes into a form that meets shipping and burial ground requirements.

1.1.7 Unrestricted Area

An Unrestricted Area is any area at or beyond the Site Boundary, access to which is not controlled by either ENIP2, ENO, or other site licensee for purposes of protection of individuals from exposure to radiation and radioactive materials.

1.2 Exclusion Distance and Restricted Area

1.2.1 The minimum distance from the reactor facility to the nearest land boundary of the exclusion area, as defined in Part 100 of the Commission's regulations, shall be 1400 feet.

1.2.2 The minimum distance from the reactor center line to the boundary of the site exclusion area and the outer boundary of the low population zone as defined in 10 CFR 100.3 is 460 meters and 1100 meters, respectively. For the purpose of satisfying 10 CFR Part 20, the Restricted Area is the same as the Exclusion Area defined in Figure 2.2-2 of Section 2.2 of the IP#2 FSAR.

1.3 Principal Activities

1.3.1 The principal activities carried on within the Exclusion Area shall be the generation, transmission and distribution of steam and electrical energy (except by gas-fired power plant); associated service activities; activities relating to the controlled conversion of the atomic energy of fuel to heat energy by the process of nuclear fission; and the storage, utilization and production of special nuclear, source and byproduct materials. Transmission and distribution of natural gas shall be through the use of facilities located as described in the application as amended.

which are appropriate in view of the nature of the repair, replacement, or modification, and the condition of the system.

5.2 Testing

5.2.5 Functional radiation monitoring systems (only for the following: nuclear services building sewage, sphere foundation sump, and secondary purification blowdown cooling water) and area radiation monitoring systems shall be:

- (a) qualitatively checked daily to verify acceptable operability of instrument channel behavior during operation, and
- (b) tested quarterly by injection of a simulated signal into the instrument channel to verify that it is operable, including alarm and/or trip initiating action. The quarterly interval is defined as quarterly plus or minus 25% of the quarter.

5.2.6 Unit 1 radioactive effluent monitoring instrumentation shall satisfy the surveillance requirements as specified in Specification 4.10 of Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

5.3 Spent Fuel Storage Pool Sampling

Any spent fuel storage pool containing spent fuel stored in water shall be sampled monthly for chloride level, pH and Cesium 137 activity. If Cesium 137 activity is found to be elevated above normal levels, an effort shall be promptly initiated to investigate the cause of the elevated level and take subsequent corrective action, as appropriate.

5.4 Sealed Sources

All sealed sources located on the Indian Point Units 1 and 2 Site are maintained under the Indian Point Unit No. 2 Facility Operating License No. DPR-26 and surveillance and use of such sources are addressed in Appendix A to the Indian Point Unit No. 2 Facility Operating License No. DPR-26.

The reports shall also include the following: a summary description of the radiological environmental monitoring program; at least two legible maps³ covering all sampling locations keyed to a table giving distances and directions from the centerline of one reactor; the results of ENO participation in the Interlaboratory Comparison Program; discussion of all deviations from the sampling schedule; and discussion of all analyses in which the LLD required was not achievable.

6.1.3 Radioactive Effluent Release Report¹

6.1.3.1 Routine Radioactive Effluent Release Reports covering the previous 12 months of operation shall be submitted by May 1 of each year.

6.1.3.2 The Radioactive Effluent Release Report shall include a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the unit as outlined in the Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants", Revision 1, June 1974, with data summarized on a quarterly basis following the format of Appendix B thereof.

The Radioactive Effluent Release Report to be submitted by May 1 of each year shall include an annual summary of hourly meteorological data collected over the previous year. This annual summary may be either in the form of an hour-by-hour listing of magnetic tape of wind speed, wind direction, atmospheric stability, and precipitation (if measured), or in the form of joint frequency distribution of wind speed, wind direction, and atmospheric stability.⁴ This same report

¹ A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

³ One map shall cover stations near the site boundary; a second shall include more distant stations.

⁴ In lieu of submission with the first half year Radioactive Effluent Release Report, ENO has the option of retaining this summary of required meteorological data on site in a file that shall be provided to the NRC upon request.

APPENDIX A

TO

FACILITY OPERATING LICENSE DPR-26

FOR

ENTERGY NUCLEAR INDIAN POINT 2, LLC
AND ENTERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING PLANT UNIT NO. 2

DOCKET NO. 50-247

TECHNICAL SPECIFICATIONS AND BASES

Amendment No.

1.20 SITE BOUNDARY

The site boundary is that line beyond which the land is neither owned, leased, nor otherwise controlled by either Entergy Nuclear Indian Point 2, LLC (ENIP2), Entergy Nuclear Operations, Inc. (ENO), or other site licensee.

1.21 SOLIDIFICATION

Solidification is the conversion of wet wastes into a form that meets shipping and burial ground requirements.

1.22 UNRESTRICTED AREA

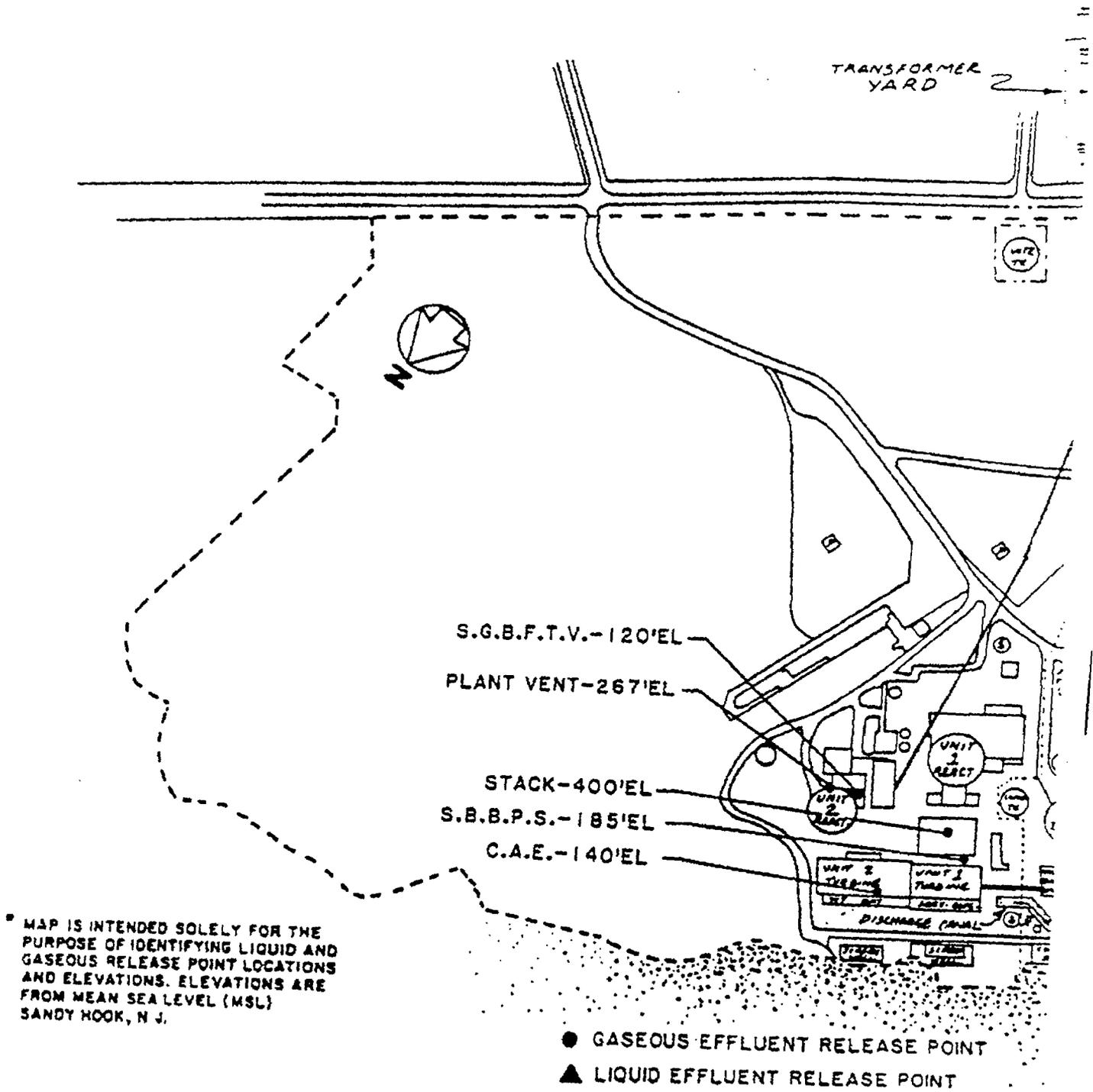
An unrestricted area is any area at or beyond the site boundary access to which is not controlled by either ENIP2, ENO, or other site licensee for purposes of protection of individuals from exposure to radiation and radioactive materials.

1.23 VENTILATION EXHAUST TREATMENT SYSTEM

A Ventilation Exhaust Treatment System is any system designed and installed to reduce gaseous radioiodine or radioactive material in particulate form in effluents by passing ventilation or vent exhaust gases through charcoal adsorbers and/or HEPA filters for the purpose of removing iodines or particulates from the gaseous exhaust stream prior to the release to the environment. Such a system is not considered to have any effect on noble gas effluents. Engineered Safety Feature (ESF) atmosphere cleanup systems are not considered to be Ventilation Exhaust Treatment System components.

1.24 VENTING

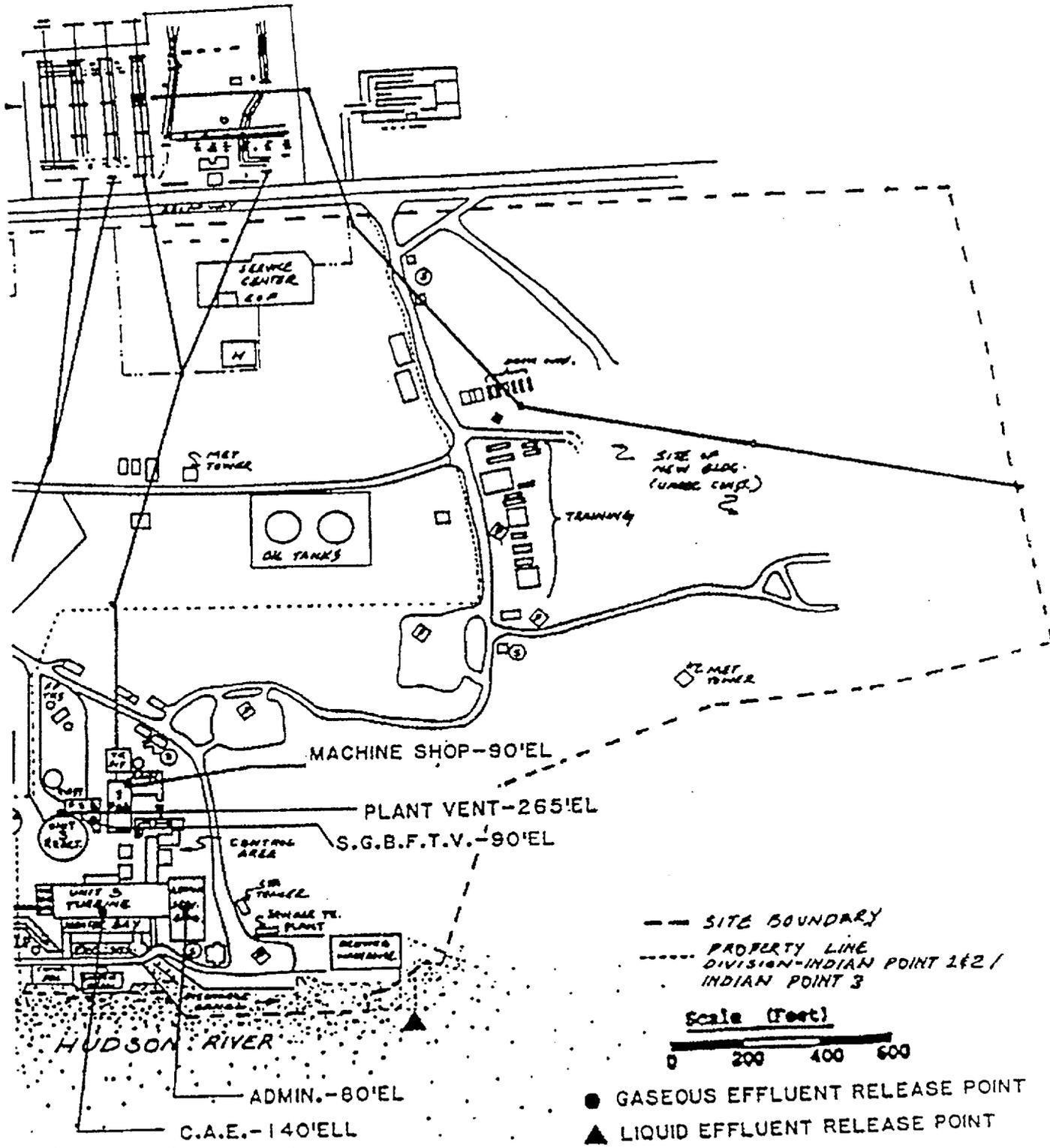
Venting is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is not provided or required.



MAP IS INTENDED SOLELY FOR THE PURPOSE OF IDENTIFYING LIQUID AND GASEOUS RELEASE POINT LOCATIONS AND ELEVATIONS. ELEVATIONS ARE FROM MEAN SEA LEVEL (MSL) SANDY HOOK, N.J.

MAP DEFINING UNRESTRICTED AREAS FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS

FIGURE 5.1-1
A



MAP IS INTENDED SOLELY FOR THE PURPOSE OF IDENTIFYING LIQUID AND GASEOUS RELEASE POINT LOCATIONS AND ELEVATIONS. ELEVATIONS ARE FROM MEAN SEA LEVEL (MSL) SANDY HOOK, N J.

MAP DEFINING UNRESTRICTED AREAS FOR RADIOACTIVE GASEOUS AND LIQUID EFFLUENTS

FIGURE 5.1-1
B

APPENDIX B
TO
FACILITY OPERATING LICENSE
FOR
ENTERGY NUCLEAR INDIAN POINT 2, LLC
AND ENTERGY NUCLEAR OPERATIONS, INC.

INDIAN POINT NUCLEAR GENERATING
UNITS NUMBER 1 AND 2

ENVIRONMENTAL TECHNICAL SPECIFICATION
REQUIREMENTS

NON-RADIOLOGICAL ENVIRONMENTAL PROTECTION PLAN

FACILITY LICENSES NO. DPR-5 AND DPR-26

DOCKET NUMBERS 50-3 AND 50-247

Unit 1 Amendment No.
Unit 2 Amendment No.