

December 9, 1988

Docket No. 50-424

Mr. W. G. Hairston, III  
Senior Vice President -  
Nuclear Operations  
Georgia Power Company  
P.O. Box 1295  
Birmingham, Alabama 35201

Dear Mr. Hairston:

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT (TAC 71404)

Enclosed for your information is a copy of a "Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration Determination and Opportunity for Hearing" related to your December 6, 1988, request for an amendment to Facility Operating License NPF-68 for the Vogtle Electric Generating Plant, Unit 1. The proposed amendment would revise Technical Specification requirements for the Control Room Emergency Filtration System and its associated actuation instrumentation to reflect the control room configuration for two-unit operation.

The notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

*JSI*

Jon B. Hopkins, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
Federal Register Notice

cc w/enclosure:  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

December 9, 1988

Docket No. 50-424

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Senior Vice President -  
Nuclear Operations  
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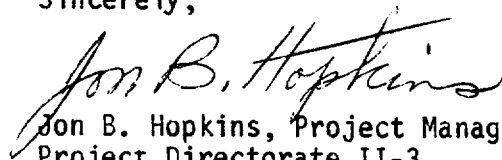
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Jon B. Hopkins, Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosure:  
Federal Register Notice

cc w/enclosure:  
See next page

Mr. W. G. Hairston, III  
Georgia Power Company

Vogtle Electric Generating Plant

cc:

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Atlanta, Georgia 30323

Mr. J. Leonard Ledbetter, Commissioner  
Department of Natural Resources  
270 Washington Street, NW  
Atlanta, Georgia 30334

Office of the County Commissioner  
Burke County Commission  
Waynesboro, Georgia 30830

Attorney General  
Law Department  
132 Judicial Building  
Atlanta, Georgia 30334

Office of Planning and Budget  
Room 615B  
270 Washington Street, S.W.  
Atlanta, Georgia 30334

UNITED STATES NUCLEAR REGULATORY COMMISSIONGEORGIA POWER COMPANYOGLETHORPE POWER CORPORATIONMUNICIPAL ELECTRIC AUTHORITY OF GEORGIACITY OF DALTON, GEORGIADOCKET NO. 50-424NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE AND PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION AND OPPORTUNITY FOR HEARING

The United States Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF-68, issued to Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensee), for operation of the Vogtle Electric Generating Plant, Unit 1, located in Burke County, Georgia.

The licensee proposes to delete the Vogtle, Unit 1 Technical Specifications (TS) Section 3/4.3.3.7, Chlorine Detection Systems, and amend Section 3/4.3.2, Control Room Emergency Filtration System, and the relevant portions of Section 3/4.3.2, Engineered Safety Features Actuation System Instrumentation, and Bases 3/4.3, Instrumentation, to reflect the control room configuration for two-unit operation. This amendment request is necessitated by differences between the current Unit 1 limiting conditions for operation, action statements, and surveillance requirements, and those proposed for the combined Unit 1 and Unit 2.

The combined control room envelope will be served by four Control Room Emergency Filtration System (CREFS) trains. Two trains will be powered from

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Unit 1 and two powered from Unit 2. Each train has sufficient capacity to remove the combined control room heat load and pressurize the combined control room to 1/8 inch water gauge relative to adjacent areas. The bases of the combined Unit 1 and Unit 2 TS meet the same criteria as the current Unit 1 TS. That is, the system has been designed with sufficient heat removal capability so that equipment qualification temperatures will not be exceeded and the control room will remain habitable during and following all credible accident conditions, as well as meet single failure criteria.

During normal plant operation the combined control room is to be served by a non-safety related HVAC system which draws in outside air makeup via either of two outside air (OSA) intakes. There is one intake associated with each unit. Each air intake is provided with two redundant radiation monitors that are powered by the associated unit. Upon detection of radiation in one OSA intake at or above the radiation monitor setpoint, each monitor will automatically initiate both of the associated unit's Control Room Isolation signals (CRI-A and CRI-B). Each CRI signal functions to start its associated CREFS and isolate the normal HVAC system. The lead/lag logic employed will permit only one CREFS to start. If high radiation is detected in both OSA intakes, all four CRI signals will be generated and two CREFSs will start. Upon receipt of a Safety Injection (SI) signal in a unit, the associated train's CRI signal will be generated. Just as for high radiation initiated CRI signals, if an SI-A and an SI-B initiate CRI signals, the lead/lag logic will permit only one CREFS to start.

The radiological consequences of the limiting LOCA have been analyzed by the licensee for the two-unit control room configuration. These analyses

showed slight increases in control room doses; however, the total doses remained within the limits of 10 CFR 50, Appendix A, General Design Criterion 19. The NRC staff approved the revised control room doses in the safety evaluation for Amendment 11 to the Vogtle Unit 1 TS. The bases for the operability of the CREFS actuation instrumentation also remain applicable. These instruments provide redundant and diverse means for initiation of a CRI in response to credible accidents in either unit, as well as meet single failure criteria.

The licensee has elected to limit on-site chlorine gas storage to 20 lbs or less on a permanent basis, eliminating the need to consider an on-site chlorine release as a credible accident and permitting the deletion of TS Section 3/4.3.3.7 and the disabling of the circuitry for Control Room Isolation on a chlorine signal.

The limiting condition for operation and mode applicability for TS 3/4.3.2 have been revised to reflect that four CREFSs are required and that the specification is applicable when either unit is in Mode 1, 2, 3 or 4 or in Mode 5 or 6 during movement of irradiated fuel or movement of loads over irradiated fuel. That is, the specification is applicable at all times unless both units are shut down and no movement of irradiated fuel or movement of loads over irradiated fuel is occurring. This change was requested in the licensee's letter dated December 6, 1988.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

The licensee has evaluated the proposed changes in the plant TS in accordance with the standards of 10 CFR 50.92(c) and has determined that operation of Vogtle, Unit 1 in accordance with these changes would not:

- (1) involve a significant increase in the probability or consequences of an accident previously evaluated. The changes would revise the limiting conditions for operation, action statements, and surveillance requirements for the Control Room Emergency Filtration System and associated instrumentation to reflect the two-unit control room configuration. The changes would include administrative modifications which delete obsolete footnotes and maintain consistent nomenclature. These changes would not affect equipment involved in the initiation of previously analyzed accidents, hence, the probability of such accidents would not be increased. The changes would, however, affect equipment involved in accident mitigation. The heat removal capability of the two-unit control room configuration is such that

equipment qualification temperatures would not be exceeded and the control room would be habitable during and following a credible accident.

Since chlorine gas will no longer be stored on-site in quantities exceeding 20 lbs, the consequences of an on-site chlorine release need no longer be considered, permitting the deletion of the corresponding TS.

For the above reasons, the consequences of previously evaluated accidents would not be significantly increased.

- (2) create the possibility of a new or different kind of accident from any accident previously evaluated. The changes would not introduce any new equipment into the plant or require existing equipment to operate in a different manner from which it was designed to operate. The operating configurations which would be allowed by the proposed changes have been reviewed against postulated accident conditions, including single failures. This review demonstrated that control room habitability would be maintained consistent with the current TS requirements and that an unanalyzed failure mode would not be made possible by the changes. A new or different kind of accident would, therefore, not result.
- (3) involve a significant reduction in a margin of safety. The changes would not affect safety limits or limiting safety system settings. The bases of the current Unit 1 TS would be maintained for the two-unit control room configuration. The current Unit 1 criteria for control room heat removal, pressurization, radiation protection, and safety injection



actuation would be met, including single failure capability. The proposed surveillance requirements and allowed outage times would also be consistent with current Unit 1 requirements. Since the proposed changes would maintain the same level of protection as the current Unit 1 TS margins of safety would not be reduced.

The NRC staff believes that the proposed changes to the TS meet the criteria specified in 10 CFR 50.92(c) and, hence, proposes to determine that they involve no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination. The Commission will not normally make a final determination unless it receives a request for a hearing.

Written comments may be submitted by mail to the Regulatory Publications Branch, Division of Freedom of Information and Publications Services, Office of Administration and Resources Management, U.S. Nuclear Regulatory Commission, Washington, D. C. 20555, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room P-216, Phillips Building, 7920 Norfolk Avenue, Bethesda, Maryland, from 7:30 a.m. to 4:15 p.m. Copies of written comments received may be examined at the NRC Public Document Room, 2120 L Street, N.W., Washington, D.C. The filing of requests for hearing and petitions for leave to intervene are discussed below.

By \_\_\_\_\_, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written petition for leave to intervene. Requests for a hearing and petitions for leave to intervene shall be filed in accordance with the Commission's "Rule of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR §2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made a party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend

the petition without requesting leave of the Board up to fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than fifteen (15) days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene, which must include a list of the contentions which are sought to be litigated in the matter, and the bases for each contention set forth with reasonable specificity. Contentions shall be limited to matters within the scope of the amendment under consideration. A petitioner who fails to file such a supplement which satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Docketing and Service Branch, or may be delivered to the Commission's Public Document Room, 2120 L Street, N.W., Washington, D.C., by the above date. Where petitions are filed during the last ten (10) days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-800-325-6000 (in Missouri 1-800-342-6700). The Western Union operator should be given Datagram Identification Number 3737 and the following message addressed to David B. Matthews: Petitioner's name and telephone number; date petition was mailed; plant name; and publication date and page number of this FEDERAL REGISTER notice. A copy of the petition should also be sent to the Office of General Counsel, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, and to Mr. Arthur H. Dombey, Troutman,

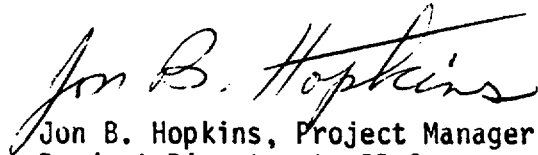
Sanders, Lockerman and Ashmore, Chandler Building, Suite 1400, 127 Peachtree Street, N.E., Atlanta, Georgia 30043, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, 2120 L Street, N.W., Washington, D. C. 20555, and at the Burke County Public Library, 412 Fourth Street, Waynesboro, Georgia 30830.

Dated at Rockville, Maryland, this 9th day of December 1988.

FOR THE NUCLEAR REGULATORY COMMISSION



Jon B. Hopkins, Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

Sanders, Lockerman and Ashmore, Chandler Building, Suite 1400, 127 Peachtree Street, N.E., Atlanta, Georgia 30043, attorney for the licensee.

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Dated at Rockville, Maryland, this 9th day of December 1988.

FOR THE NUCLEAR REGULATORY COMMISSION

*JS*

Jon B. Hopkins, Project Manager  
Project Directorate II-3  
Division of Reactor Projects I/II  
Office of Nuclear Reactor Regulation

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