

January 12, 1995

Mr. C. K. McCoy  
Vice President - Nuclear  
Vogtle Project  
Georgia Power Company  
P. O. Box 1295  
Birmingham, AL 35201

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C. Grimes 0-11 F23  
ACRS(4) T-2 E26  
D. Hood

SUBJECT: ISSUANCE OF AMENDMENTS - VOGTLE NUCLEAR GENERATING PLANT,  
UNITS 1 AND 2 (TAC NOS. M90273 AND 90274)

Dear Mr. McCoy:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 80 to Facility Operating License NPF-68 and Amendment No. 59 to Facility Operating License NPF-81 for the Vogtle Electric Generating Plant (VEGP), Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated August 16, 1994. They revise VEGP TS 3/4.7.1.1 and its Bases regarding the setpoint tolerance for the Main Steam Safety Valves.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Louis L. Wheeler, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

Enclosures:

- 1. Amendment No. 80 to NPF-68
- 2. Amendment No. 59 to NPF-81
- 3. Safety Evaluation

cc w/encl: See next page

DOCUMENT NAME: G:\VOGTLE\VOG90273.AMD

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

January 12, 1995

Mr. C. K. McCoy  
Vice President - Nuclear  
Vogtle Project  
Georgia Power Company  
P. O. Box 1295  
Birmingham, AL 35201

SUBJECT: ISSUANCE OF AMENDMENTS - VOGTLE ELECTRIC GENERATING PLANT,  
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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Louis L. Wheeler".

Louis L. Wheeler, Senior Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

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3. Safety Evaluation

cc w/encl: See next page

Mr. C. K. McCoy  
Georgia Power Company

Vogtle Electric Generating Plant

cc:

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80  
License No. NPF-68

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Facility Operating License No. NPF-68 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated August 16, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

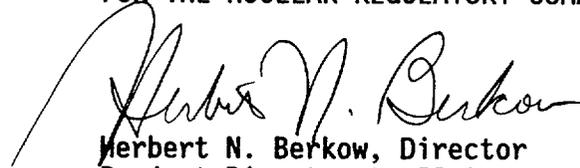
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-68 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 80 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: January 12, 1995



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59  
License No. NPF-81

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 2 (the facility) Facility Operating License No. NPF-81 filed by the Georgia Power Company, acting for itself, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the licensees), dated August 16, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-81 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 59, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. GPC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Technical Specification  
Changes

Date of Issuance: January 12, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

AND

TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. NPF-81

DOCKET NO. 50-425

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change. The unrevised overleaf pages are included for your convenience.

Remove Pages

Insert Pages

3/4 7-1  
3/4 7-2\*

3/4 7-1  
3/4 7-2\*

3/4 7-3  
3/4 7-4\*

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3/4 7-4\*

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B 3/4 7-2\*

B 3/4 7-1  
B 3/4 7-2\*

\* Overleaf pages containing no change

### 3/4.7 PLANT SYSTEMS

#### 3/4.7.1 TURBINE CYCLE

##### SAFETY VALVES

#### LIMITING CONDITION FOR OPERATION

---

3.7.1.1 All main steam line Code safety valves associated with each steam generator shall be OPERABLE with lift settings\* as specified in Table 3.7-2.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With four reactor coolant loops and associated steam generators in operation and with one or more main steam line Code safety valves inoperable, operation in MODES 1, 2, and 3 may proceed, provided that within 4 hours, either the inoperable valve is restored to OPERABLE status or the Power Range Neutron Flux High Trip Setpoint (NI-0041B&C, NI-0042B&C, NI-0043B&C, NI-0044B&C) is reduced per Table 3.7-1; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. The provisions of Specification 3.0.4 are not applicable.

#### SURVEILLANCE REQUIREMENTS

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4.7.1.1 No additional requirements other than those required by Specification 4.0.5.

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\* All valves tested must have "as-left" lift setpoints that are within  $\pm 1\%$  of the lift setting values in Table 3.7-2.

TABLE 3.7-1

MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT WITH  
INOPERABLE STEAM LINE SAFETY VALVES

<u>MAXIMUM NUMBER OF INOPERABLE SAFETY VALVES ON ANY OPERATING STEAM GENERATOR</u>	<u>MAXIMUM ALLOWABLE POWER RANGE NEUTRON FLUX HIGH SETPOINT (PERCENT OF RATED THERMAL POWER)</u>
1	87
2	65
3	43

TABLE 3.7-2

STEAM LINE SAFETY VALVES PER LOOP

<u>VALVE NUMBER</u>				<u>LIFT SETTING</u> <u>(+2%,-3%)*</u>	<u>ORIFICE SIZE</u>
SG-1	SG-2	SG-3	SG-4		
1. PSV 3001	PSV 3011	PSV 3021	PSV 3031	1185 psig	16.0 in <sup>2</sup> .
2. PSV 3002	PSV 3012	PSV 3022	PSV 3032	1200 psig	16.0 in <sup>2</sup> .
3. PSV 3003	PSV 3013	PSV 3023	PSV 3033	1210 psig	16.0 in <sup>2</sup> .
4. PSV 3004	PSV 3014	PSV 3024	PSV 3034	1220 psig	16.0 in <sup>2</sup> .
5. PSV 3005	PSV 3015	PSV 3025	PSV 3035	1235 psig	16.0 in <sup>2</sup> .

\*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

## PLANT SYSTEMS

### AUXILIARY FEEDWATER SYSTEM

#### LIMITING CONDITION FOR OPERATION

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3.7.1.2 At least three independent steam generator auxiliary feedwater pumps and associated flow paths shall be OPERABLE with:

- a. Two motor-driven auxiliary feedwater pumps, each capable of being powered from separate emergency busses, and
- b. One steam turbine-driven auxiliary feedwater pump capable of being powered from an OPERABLE steam supply system.

APPLICABILITY: MODES 1, 2, and 3.

#### ACTION:

- a. With one auxiliary feedwater pump inoperable, restore the required auxiliary feedwater pumps to OPERABLE status within 72 hours or be in at least HOT STANDBY within the next 6 hours and in HOT SHUTDOWN within the following 6 hours.
- b. With two auxiliary feedwater pumps inoperable, be in at least HOT STANDBY within 6 hours and in HOT SHUTDOWN within the following 6 hours.
- c. With three auxiliary feedwater pumps inoperable, immediately initiate corrective action to restore at least one auxiliary feedwater pump to OPERABLE status as soon as possible.

#### SURVEILLANCE REQUIREMENTS

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4.7.1.2.1 Each auxiliary feedwater pump shall be demonstrated OPERABLE:

- a. At least once per 31 days on a STAGGERED TEST BASIS by:
  - 1) Verifying that each motor-driven pump develops a discharge pressure of greater than or equal to 1605 psig at a flow of greater than or equal to 150 gpm (FI-15101, FI-15102);
  - 2) Verifying that the steam turbine-driven pump develops a discharge pressure of greater than or equal to 1675 psig at a flow of greater than or equal to 145 gpm (FI-15100) when the secondary steam supply pressure (PI-5105A, PI-5105B) is greater than 900 psig. The provisions of Specification 4.0.4 are not applicable for entry into MODE 3.

## 3/4.7 PLANT SYSTEM

### BASES

#### 3/4.7.1 TURBINE CYCLE

##### 3/4.7.1.1 SAFETY VALVES

The OPERABILITY of the main steam line Code safety valves ensures that the Secondary System pressure will be limited to within 110% (1304 psig) of its design pressure of 1185 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a Turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, 1974 Edition. The safety valves are tested in accordance with the requirements of Section XI of the ASME Code. The lift setting allowable values are consistent with the safety analysis. In the event a safety valve lifts outside of the tolerances specified in Table 3.7-2, the Section XI provisions of adjusting the setpoint and testing additional valves applies. When tested, the as-left setting will be within  $\pm 1\%$  of the specified set pressure. The total relieving capacity for all valves on all of the steam lines is 18,607,220 lbs/h which is 117% of the total secondary steam flow of  $15.92 \times 10^6$  lbs/h at 100% RATED THERMAL POWER. A minimum of two OPERABLE safety valves per steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Table 3.7-1.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in Secondary Coolant System steam flow and THERMAL POWER required by the reduced Reactor trip settings of the Power Range Neutron Flux channels. The Reactor Trip Setpoint reductions are derived on the following basis:

For four loop operation

$$SP = \frac{(X) - (Y)(V)}{X} \times (109)$$

Where:

- SP = Reduced Reactor Trip Setpoint in percent of RATED THERMAL POWER,
- V = Maximum number of inoperable safety valves per steam line,
- 109 = Power Range Neutron Flux-High Trip Setpoint for four loop operation,
- X = Total relieving capacity of all safety valves per steam line in lbs/hour, and
- Y = Maximum relieving capacity of any one safety valve in lbs/hour.

## PLANT SYSTEMS

### BASES

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#### 3/4.7.1.2 AUXILIARY FEEDWATER SYSTEM

The OPERABILITY of the Auxiliary Feedwater System ensures that the Reactor Coolant System can be cooled down to less than 350°F from normal operating conditions in the event of a total loss-of-offsite power.

The auxiliary feedwater system is capable of delivering a total feedwater flow of 510 gpm at a pressure of 1221 psig to the entrance of at least two steam generators while allowing for: 1) any possible spillage through the design worst case break of the main feedwater line; 2) the design worst case single failure and, 3) recirculation flow (applicable for turbine-driven auxiliary feedwater pump only). This capacity is sufficient to ensure that adequate feedwater flow is available to remove decay heat and reduce the Reactor Coolant System temperature to less than 350°F, at which point the Residual Heat Removal System may be placed in operation. Because it is not desirable to inject cold auxiliary feedwater into the steam generators during power generation, the pumps must be tested on miniflow (recirculation). The surveillance acceptance criteria are based on the miniflow testing configuration and are specified to ensure the above limits are met during injection to the steam generators.

#### 3/4.7.1.3 CONDENSATE STORAGE TANK

The OPERABILITY of the condensate storage tank with the minimum water volume ensures that sufficient water is available to maintain the RCS at HOT STANDBY conditions for 4 hours with steam discharge to the atmosphere concurrent with total loss-of-offsite power, followed by a cooldown to RHR initiation conditions. The contained water volume limit includes an allowance for water not usable because of tank discharge line location or other physical characteristics.

#### 3/4.7.1.4 SPECIFIC ACTIVITY

The limitations on Secondary Coolant System specific activity ensure that the resultant offsite radiation dose will be limited to a small fraction of 10 CFR Part 100 dose guideline values in the event of a steam line rupture. This dose also includes the effects of a coincident 0.35 gpm primary-to-secondary tube leak in the steam generator of the affected steam line. These values are consistent with the assumptions used in the safety analyses.

The identification of 95% of the gross specific activity by definition does not obligate VEGP into calculating E every time gross activity is determined.

#### 3/4.7.1.5 MAIN STEAM LINE ISOLATION VALVES

The OPERABILITY of the main steam line isolation valves and bypass valves ensures that no more than one steam generator will blow down in the event of a steam line rupture. This restriction is required to: (1) minimize the positive reactivity effects of the Reactor Coolant System cooldown associated with the blowdown, and (2) limit the pressure rise within containment in the event the steam line rupture occurs within containment. The OPERABILITY of the main steam isolation valves within the closure times of the Surveillance Requirements are consistent with the assumptions used in the safety analyses.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NPF-68  
AND AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NPF-81

GEORGIA POWER COMPANY

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-424 AND 50-425

1.0 INTRODUCTION

By letter August 16, 1994, Georgia Power Company, et al. (the licensee) submitted a request for changes to the Vogtle Electric Generating Plant (Vogtle or VEGP), Units 1 and 2 Technical Specifications (TS). The requested changes would revise TS 3/4.7.1.1 and its Bases to increase the main steam safety valve (MSSV) setpoint tolerance ranges from  $\pm 1\%$  to  $+2\%$ ,  $-3\%$  to accommodate setpoint drift that may occur with these valves during plant operation. The proposed changes also require that normal surveillance testing of the MSSVs, and any additional testing of the MSSVs if a setpoint tolerance is exceeded, be performed in accordance with the provisions of Section XI of the ASME Code. Additionally, following testing, the as-left lift setting of the MSSVs will be within  $\pm 1\%$  of the pressure specified in the TS. At Vogtle 1 and 2, there is a total of 20 MSSVs per Unit (i.e., five per main steam line), each set at increments which range from 1185 psig to 1235 psig.

2.0 EVALUATION

The increase in MSSV setpoint tolerance was reviewed with respect to the accident analyses presented in the VEGP Final Safety Analysis Report (FSAR). The licensee's submittal included the results of evaluations of the proposed change for Loss-of-Coolant-Accident (LOCA), non-LOCA and Steam Generator Tube Rupture (SGTR) events. The licensee concluded the proposed change will not adversely affect the pressure boundary integrity or safety function of the valves. The evaluation demonstrated that the acceptance criteria of the accident analyses continued to be met.

The licensee's submittal included the provision that the MSSVs will be tested in accordance with the requirements of Section XI of the ASME Code. In the event an MSSV lifts outside the setpoint tolerance values, the Section XI provisions for adjusting the setpoint and testing additional valves will apply. This condition is included in the proposed changes to the TS.

The licensee has determined that the proposed TS changes do not result in a significant reduction in the margin of safety. The limiting transient in each accident category has been analyzed to determine the effect of the change in the setpoint tolerances. Further, in order to prevent the setpoints from

drifting outside the +2%, -3% range, the licensee will continue to require MSSV setpoint tolerances to be restored to  $\pm 1\%$  when the as-found lift setting exceeds  $\pm 1\%$  prior to declaring the MSSVs operable. This will prevent excessive setpoint drift which would cause the peak system pressures to exceed the allowable limits.

The staff has reviewed the licensee's submittal and agrees with their conclusion that the analysis demonstrates the acceptability of the proposed TS changes. The proposed increase in the setpoint tolerances of the MSSVs has been shown to be acceptable for meeting the plant design basis. Also, for those occurrences where the as-found setpoints of MSSVs are in excess of  $\pm 1\%$ , resetting to within  $\pm 1\%$  of the nominal setpoint will be required prior to declaring them operable. In addition, the proposed changes to the TS are consistent with the requirements of the Improved Standard Technical Specifications found in NUREG-1431. Therefore, these proposed TS changes have no significant safety impact on the operation of Vogtle Units 1 and 2, and are acceptable.

### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Georgia State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 47168 dated September 14, 1994). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

### 5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Hammer  
L. Wheeler

Date: January 12, 1995